DCPP UNIT 1 ISI/IST PROGRAM PLAN CHANGE DESCRIPTION REVISION 7 ADDENDA 1

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Addenda 1 to Revision 7 of the DCPP Unit 1 Inservice Inspection and Testing Program Plan is issued as a controlled document in accordance with PG&E procedure AP E-4S7. All changed pages should be removed from the Program Plan binder and replaced with Addenda 1 to Revision 7.

The following technical changes are incorporated in Addenda 1 to Revision 7 and are identified by revision bars.

INSERVICE INSPECTION (ISI)

CHANGE	DESCRIPTION	JUSTIFICATION
Pumps Table 1.5 Page 3 of 5	Include Item B10.20 In Its Entirety	Inadvertent Omission From Rev. 6 to Rev. 7
Section 3.5 Tables 5.1, 5.2, and 5.3 Page 1 of 1	Changed Rounded Years To Actual Fractions 3, 7, 10 Yrs to 3 1/3, 6 2/3, 10 Yrs	These Inspection Periods Are Inconsistent With The Other Specified Periods In The Program Plan
Test Summary Table 5.3 Page 2 of 4	Changed IWD-5223 Test(a) To Required Test(b). This Is An Atmospheric Tank	This Test Is Conducted With The Surge Tank Filled To Normal Level
Table 5.3 Page 3 of 4	Test #15: Sheet #33 and 36 instead of 25 and 33. Test #16: Sheets 24, 25, 33, and 34 instead of 24 25 and 34	Typographical Correction Of Sheets #'s.
Table 5.3 Page 4 of 4	Include Test 23 In Its Entirety .	Inadvertent Omission From Rev. 6 to Rev. 7
Test Summary Table 5.4, Pages 2 Thru 9 of 9. Test Summary Table 5.2, Pages 9, 10, 11, 12, 13 of 13.	Clarify Basis, Alternate Testing and Test Schedule for Request For Relief #8. Revise Alternate Test Schedule For Request For Relief #9	NRC Request For Additional Information In Telephone Call

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DIABLO CANYON POWER PLANT-UNIT 1

TEN YEAR ASME SECT	EXAMINATION PROGRAM FION XI SYSTEMS-CLASS 1	REV. 7 Addenda 1		MAJOR ITEM: <u>PUMPS (B-K-1)(B-K-</u> TABLE: <u>1.5</u> PAGE <u>3A</u> of <u>5</u>				<u></u>
CATEGORY ITEM	COMPONENT OR SYSTEM	GENERAL IDENTIFICATION	NDE METH	TOTAL ITEMS	EXAMINATION AMOUNT & EXTENT	EXAM'N PERIOD	X TO DATE	EXAM RELIEF'S & REMARKS
	<u>Pumps</u> - Cont'd Reactor Coolant Pumps		<u></u>		Exam'n Area meets or exceeds req'ts of IWB_2500_15			
B-K-1	Support Members				(Typical Design)			
	Integrally Welded Attachments (To Pressure Bound.)							
B10.20	Welded Attachments Pump 1-2	WELDS #1, #2, #3 [1.5-1]	PT	3	1 Lug 100% 1 Lug 100% 1 Lug 100%	One Two Three	33 66 100	Welded attachment required on one pump. Category B-K-1 Footnote 1 Ref. B-J which specifies 1 loop.
B-K-2	Support Components							
B11.20	Support Components Pump 1-1	Support Components	VT-3	(1)		One	25	(¹) <u>B11.20</u> Items include all pump support members (support lugs, bolts, & column, lateral sup- port members)
	Support Components Pump 1-2		VT-3	(1)		Тwo	50	
	Support Components Pump 1-3	Support Components	VT-3	(1)		Three	75	
	Support Components Pump 1-4	Support Components	VT-3	(1)		Three	100	

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DIABLO CANYON POWER PLANT - UNIT 1 TEN YEAR EXAMINATION PROGRAM ASME SECTION XI SYSTEMS

TABLE 5.1, 5.2, & 5.3 REV. 7 ADDENDA 1 TEST SUMMARY FOOTNOTES SECTION 3.5A PAGE <u>1</u> OF <u>1</u>

FOOTNOTES:

- (1) Identifies the drawing sheet number of the ASME Code classification drawings, PG&E drawing 102028, Revision 21.
- (2) Identifies the applicable ASME B&PV Code Section XI pressure test requirement, as established by the 1977 Edition - Summer 1978 Addenda.
- (3) INS/FUNCT is the test pressure developed under the operating condition associated with normal system operation or a system/component functional test. LLRT is the local leak rate test per 10CFR50 App J.
- (4) I is one test per each inspection interval (10 YR) P is one test per each inspection period (40 months; 3 1/3, 6 2/3, and 10 years) R is one test per each refueling outage
- (5) P_0 is the nominal system operating pressure.

Psy is the safety or relief valve setting pressure.

 P_{g} is the design pressure of vapor or gas space above liquid level for which over pressure protection is provided by relief values.

<u>NOTE 1</u>: Piping pressurized during normal reactor operation and does not require test per IWC-5221. Normal reactor operation includes systems in operation during startup, operation, and Cooldown to shutdown conditions. (IWC-1220 footnote [2])



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DIAE TEN ASME	BLO CANYON POWER PLANT - UNIT 1 YEAR EXAMINATION SUMMARY E SECTION XI SYSTEMS - CLASS 2		TABLE 5.2 REV. 7			ADDENDA 1 TEST SUMMARY PAGE 9A of 13		
COM DES(PONENT OR PIPING SYSTEM CRIPTION	SHEET No.(1)	CODE REQUIREMENT(2)	REQUIRED TEST PRESS (PSIG) (3)	TEST FREQ(4)	CONTAINMENT PENETRATION	REMARKS (5)	-
						(X) From	footnote page 1 of 1	-
64.	REACTOR COOLANT PUMPS CCW SUPPLY & OIL COOLER RETURN CONTAINMENT PENETRATIONS, SPEC K2	29	IWC-5222 (a)	165	I		P _{SV} = 150	
65.	REACTOR COOLANT PUMPS CCW RETURN CONTAINMENT PENETRATION, LINE K17-1357-6	29	IWC-5222 (a)	3106	I	·	P _{SV} = 2485;	
66.	EXCESS LETDOWN HEAT EXCHANGER, SHELL SIDE	31	IWC-5222 (a)	165	I		P _{sv} = 150	ļ
67.	EXCESS LETDOWN HEAT EXCHANGER, SHELL SIDE	31	IWC-5222 (a)	165	I		P _{SV} = 150	;
68.	STEAM GEN. N ₂ SUPPLY HDR.	6	IWC-5221 (a)	INS/FUNCT	Ρ	52	LINE 1863. TESTED AT NORMAL OPERATING PRESSU	RE.
69.	LINES 4681, 4682, 4683, 4684, 4685, 4686	9	EXEMPT	NONE	NONE	59,80	NA-1130 (c). CAP SEALED	
70.	PZR. QUENCH TK. GAS ANALYZER	10	IWC-5221 (a)	LLRT@50PSI*	Ρ	76	LINE 1163, *REQUEST FOR RELIEF #8	ł
71.	PZR RELIEF TK N ₂ SUPPLY	10	IWC-5222 (a) I	NS/FUNCT@NOP*	I	52	LINE 1161, *REQUEST FOR RELIEF #8	1

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DIAE TEN ASME	BLO CANYON POWER PLANT - UNIT 1 YEAR EXAMINATION SUMMARY SECTION XI SYSTEMS - CLASS 2		TABL REV.	.E 5.2 7		ADDENDA 1	SYSTEM PRESSURE TEST SUMMARY PAGE 10A of 13	
COMPONENT OR PIPING SYSTEM DESCRIPTION		SHEET No.(1)	CODE REQUIREMENT(2)	REQUIRED TEST PRESS (PSIG) (3)	TEST FREQ(4)	CONTAINMENT PENETRATION	REMARKS (5)	
72.	PRI WTR SUP TO PZR RELIEF TK	10	IWC-5222 (a) I	NS/FUNCT@NOP*	I	52	LINE 3000. *REQUEST FOR RELIEF #8	
73.	RV'S DISCHARGE TO P.R.T.: CHG. PPS. SUCT. RV DISCH RHR HT EXCHR. 1 OUT. RV RHR HT EXCHR 2 OUT RV S.I. PPS. SUCT. RV OUT. CNT. SPRAY, PPS. DISCH. RV	10	IWC-5221 (a)	LLRT@50PSI*	Ρ	71	*REQUEST FOR RELIEF #8 LINE 1459 LINE 2003 LINE 2004 LINE 2061	
	OUT. P.R.T. S.I. PP. 1-2 DISCH. LINE RV SIS RV OUTLET HDR TO P.R.T. SIS RV OUTLET HDR TO P.R.T. SIS PP. 1-1 DISCH LINE RV SIS PPS. RECIRC. DISCH.					-	LINE 2518 LINE 2572 LINE 2998 LINE 2999 LINE 3851	
	LINE RV						LINE 3852	
74.	PZR DEADWEIGHT PRESS. GEN	10	NONE	NONE	NONE	76	LINE 478. ABANDONED IN PLACE	
75.	TANKS N ₂ SUPPLY HDR.	17	IWC-5222 (a) I	NS/FUNCT@NOP*	I	51	LINE 531. *REQUEST FOR RELIEF #8.	
76.	ACCUMS. SAMPLES HDR.	22	IWC-5221 (a) I	NS/FUNCT@NOP*	I*	59	LINE 1679. *REQUEST FOR RELIEF #8.	
77.	LINES 636, 2514, 2515, 637 4529, 4525	23	EXEMPT	NONE	NONE	76,59,78 52,80,59	NA-1130 (c). (NOTE PENETRATION RECEIVES LLRT)	















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DIAE TEN ASME	BLO CANYON POWER PLANT - UNIT 1 YEAR EXAMINATION SUMMARY SECTION XI SYSTEMS - CLASS 2	TABLE 5.2 REV. 7				ADDENDA 1	SYSTEM PRESSURE - TEST SUMMARY PAGE 12A of 13	
COMPONENT OR PIPING SYSTEM DESCRIPTION		SHEET No.(1)	CODE REQUIREMENT(2)	REQUIRED TEST PRESS (PSIG) (3)	TEST FREQ(4)	CONTAINMENT PENETRATION	REMARKS (5)	
88.	REACT. COOL. DRN. PPS. DISCH. HDR.	42	IWC-5221 (a) I	NS/FUNCT@NOP*	Ρ	50	LINE 3729. *REQUEST FOR RELIEF #8	
89.	CHPS AIR SUP. FANS 1 & 2 DISCH. EXT. AND PENETR. 83 INLET & AIR SUP.	43	IWC-5221 (a)	LLRT050PSI*	р	83	LINE 4382, 4386, 4387, 4388, 4389. *REQUEST FOR RELIEF #8	
90.	CHPS EXH AIR FILT. 2 INLET	43	IWC-5221 (a)	LLRT@50PSI*	P	81	LINE 4390. *REQUEST FOR RELIEF #8	
91.	CHPS EXH SYS. FLOW CONT. IN.	43	IWC-5221 (a)	LLRT@50PSI*	Ρ	57	LINE 4395. *REQUEST FOR RELIEF #8	
92.	CONTAINMENT VACUUM RELIEF	43	IWC-5221 (a)	LLRT@50PSI*	Ρ	63	LINE 647. *REQUEST FOR RELIEF #8	
93.	CONTAINMENT PURGE OUT	43	IWC-5221 (a)	LLRT@50PSI*	Ρ	62	LINE K2-48" (RCV-11 TO RCV-12) *REQUEST FOR RELIEF #8	
94.	CONTAINMENT PURGE IN	43	IWC-5221 (a)	LLRT@50PSI*	р	61	LINE K2-48" (FCV-661 TO FCV-660) *REQUEST FOR RELIEF #8	
95.	INCORE CHILLER WATER RTN.	43	IWC-5221 (a)	LLRT@50PSI*	Р	83	LINE 3936. *REQUEST FOR RELIEF #8	
96.	INCORE CHILLER WATER SUPPLY	43	IWC-5221 (a)	LLRT@50PSI*	P	82	LINE 3937. *REQUEST FOR RELIEF #8	
97.	CONTAINMENT AIR SAMPLE INLET	44	IWC-5222 (a)	LLRT@50PS1	P*	68	LINE 3837. *REQUEST FOR RELIEF #8	

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DIAB TEN ASME	DIABLO CANYON POWER PLANT - UNIT 1TEN YEAR EXAMINATION SUMMARYTABLE 5.2ASME SECTION XI SYSTEMS - CLASS 2REV. 7					ADDENDA 1	SYSTEM PRESSURE - TEST SUMMARY PAGE 13A of 13	
COMPONENT OR PIPING SYSTEM DESCRIPTION		SHEET No.(1)	CODE REQUIREMENT(2)	REQUIRED TEST PRESS (PSIG) (3)	TEST FREQ(4)	CONTAINMENT PENETRATION	REMARKS (5)	
98.	CONTAINMENT AIR SAMPLE RETURN	44	IWC-5222 (a)	LLRT@50PS1*	p*	69	LINE 3838. *REQUEST FOR I RELIEF #8	
99.	POST-LOCA SAMP. CNT. AIR. RTN.	44	IWC-5221 (a)	LLRT@50PSI*	P	82	LINE 5190. *REQUEST FOR RELIEF #8	
100.	POST-LOCA SAMP. CNT. AIR. SUP.	44	IWC-5221 (a)	LLRT@50PSI*	Ρ	82	LINE S-3/8" (FCV-698 TO FCV-699) *REQUEST FOR RELIEF #8	
101.	HYD. MON. CEL. 82 CNT. AIR SUP.	44	IWC-5221 (a)	LLRT@50PSI*	Ρ	52	LINE 4633. *REQUEST FOR RELIEF #8	
102.	HYD. MON. CEL. 82 CNT. AIR RTN.	44	IWC-5221 (a)	LLRT@50PSI*	Ρ	52	LINE 4634. *REQUEST FOR RELIEF #8	
103.	HYD. MON. CEL. 83 CNT. AIR SUP.	44	IWC-5221 (a)	LLRT@50PSI*	P	78	LINE 4635. *REQUEST FOR RELIEF #8	
104.	HYD. MON. CEL. 83 CNT. AIR RTN.	44	IWC-5221 (a)	LLRT@50PSI*	P	78	LINE 4636. *REQUEST FOR RELIEF #8	
105.	SERVICE AIR PENETR. HDR.	45	IWC-5221 (a)	LLRT@50PSI*	P	56	LINE 3941. *REQUEST FOR RELIEF #8	
106.	INSIDE CNT. INSTR. AIR SUP. HDR.	45	IWC-5222 (a) II	NS/FUNCT@NOP*	I	54	LINE 3242. *REQUEST FOR RELIEF #8	
107.	CNT. INST. AIR SUP. FCV-584 BP	45	IWC-5222 (a) II	NS/FUNCT@NOP*	Ī	54	LINE 4353. *REQUEST FOR RELIEF #8	

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DIABLO CANYON POWER PLANT-UNIT 1

DIA TEN ASME	BLO CANYON POWER PLANT - UNIT 1 YEAR EXAMINATION SUMMARY SECTION XI SYSTEMS - CLASS 3		TABLE REV.	5.3 7	SYSTEM PRESSURE ADDENDA 1 TEST SUMMARY PAGE 2A of 4		
COMPONENT OR PIPING SYSTEM DESCRIPTION		SHEET No.(1)	SHEET CODE No.(1) REQUIREMENT(2)	REQUIRED TEST PRESS (PSIG) (3)	TEST FREQ(4)	REMARKS (5)	
						(X) From footnote page 1 of 1	
7.	COMPONENT COOLING WATER PUMPS	25	IWD-5223 (a)	165	Ι	$P_{SV} = 150$	
			IWD-5221/5222	INS/FUNCT	Р		
8.	COMPONENT COOLING WATER	25	IWD-5223 (a)	165	I	$P_{SV} = 150$	
	newi Exchangers (Shell Side)		IWD-5221/5222	INS/FUNCT	Р		
9.	COMPONENT COOLING WATER SURGE TANK	25	IWD-5223 (b)	TANK FILLED	I	TEST CONDUCTED WITH SURGE TANK FILLED TO NORMAL LEVEL	
			IWD-5221/5222	INS/FUNCT	Р		
10.	COMPONENT COOLING WATER	25,32	IWD-5223 (a)	165	I	P _{SV} = 150	
	HEADERS A, B & C, SUPPLY & RETURN PIPING FROM COM- PONENTS WITH THE EXCEPTION OF ITEM NUMBER 11		IWD-5221/5222	INS/FUNCT	Ρ		
11.	REACTOR COOLANT PUMP	29	IWD-5223 (a)	3106	I	P _{SV} = 2485	
	& SUPPLY FROM CHECK VALVES, SPEC K17		IWD-5221/5222	INS/FUNCT	Ρ		
12.	CONDENSATE STORAGE TANK	33	IWD-5223 (b)	TANK FILLED	I		
	AND UNISULADLE FIFING		IWD-5221/5222	INS/FUNCT	Р		

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DIABLO CANYON POWER PLANT-UNIT 1

DIA TEN ASMI	BLO CANYON POWER PLANT - UNIT 1 YEAR EXAMINATION SUMMARY SECTION XI SYSTEMS - CLASS 3	······	TABLE REV.	5.3 7	SYSTEM PRESSURE ADDENDA 1 TEST.SUMMARY PAGE <u>3A</u> of <u>4</u>	
COMPONENT OR PIPING SYSTEM DESCRIPTION		SHEET No.(1)	CODE REQUIREMENT(2)	REQUIRED TEST PRESS (PSIG) (3)	TEST FREQ(4)	REMARKS (5)
						(X) From footnote page 1 of 1
13.	FIRE WATER TRANSFER TANK	33	IWD-5223 (b)	TANK FILLED	Ι	
			IWD-5221/5222	INS/FUNCT	P	
14.	MAKEUP WATER TRANSFER PUMPS AND SUCTION PIPING	33,36	IWD-5223 (b)	TANK FILLED	Ι	REQUEST FOR RELIEF #1; NO SAFETY VALVE IN SYSTEM - TESTED AT NORMAL OPERATING
			IWD-5221/5222	INS/FUNCT	Ρ	CUNUTITIONS; CUNUENSATE STURAGE TARK
15.	MU WATER TRANSFER PUMPS DISCHARGE PIPING	33,36	IWD-5223 (a)	INS/FUNCT	I	NO SAFETY VALVE IN SYSTEM - TESTED AT
			IWD-5221/5222	INS/FUNCT	Ρ	NORMAL OPERATING CONDITIONS
16.	CCW AND SPENT FUEL PIT	24,25 33,34	IWD-5223 (a)	INS/FUNCT	Ι	NO SAFETY VALVE IN SYSTEM - TESTED AT
	PIPING		IWD-5221/5222	INS/FUNCT	Р	NUKMAL UPERALING CUNUITIONS
17.	AUXILIARY SALTWATER PUMP SUPPLY TO CCW HEAT	35	IWD-5223 (a)	INS/FUNCT	1	REQUEST FOR RELIEF #4; TESTED AT MAX PUMP DISCH PRESSURE
			IWD-5221/5222	INS/FUNCT	[°] Р	REQUEST FOR RELIEF #4
18.	COMPONENT COOLING WATER	35 🦿	IWD-5223 (a)	INS/FUNCT	I	TESTED AT MAX PUMP DISCH PRESSURE
	NEAT EXCHANGERS TUBE STUE		IWD-5221/5222	INS/FUNCT \sim	P	
19.	CCW HEAT EXCHANGERS	35	IWD-5223 (c)	NONE REQ'D	-	OPEN-ENDED DISCHARGE TO OCEAN
	SALIWATER OUTLET PIPING		IWD-5221/5222	INS/FUNCT	Ρ	REQUEST FOR RELIEF #4

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DIABLO CANYON POWER PLANT-UNIT 1

DIAB TEN ASME	LO CANYON POWER PLANT - UNIT 1 YEAR EXAMINATION SUMMARY SECTION XI SYSTEMS - CLASS 3		TABLE 5.3 REV. 7			SYSTEM PRESSURE ADDENDA 1 TEST SUMMARY PAGE4A of4		
COMPONENT OR PIPING SYSTEM DESCRIPTION		SHEETCODEREQUIREDTESTNo.(1)REQUIREMENT(2)TESTPRESSFREQ(4)(PSIG)(3)		REMARKS (5)				
						(X) From footnote page 1 of 1		
20.	DIESEL ENGINE JACKET WATER COOLING SYSTEM	37	IWD-5223 (a)	7.5	I	$P_{SV} = 6$		
			IWD-5221/5222	INS/FUNCT	Р			
21.	SPENT FUEL POOL PUMPS AND SUCTION PIPING	24	IWD-5223 (a)	POOL FILLED	I	REQUEST FOR RELIEF #1: NO SAFETY		
			IWD-5221/5222	INS/FUNCT	P	OPERATING CONDITIONS; SPENT FUEL POOL		
22.	SPENT FUEL POOL PUMPS	24	IWD-5223 (a)	INS/FUNCT	I	NO SAFETY VALVE IN SYSTEM - TESTED		
	HEAT EXCHR. TO SPENT FUEL POOL		IWD-5221/5222	INS/FUNCT	Ρ	AT NORMAL OPERATING CONDITIONS.		
23.	RAW WATER STORAGE RESERVOIR T AUXILIARY FEEDWATER PUMPS SUCTION	0 3,36 37,38 39	IWD-5223(b)&(c IWD-5221/5222)INS/FUNCT INS/FUNCT	I P	NO SAFETY VALVE IN SYSTEM - TESTED AT NORMAL OPERATING CONDITIONS. (RAW WATER STORAGE RESERVOIR)		

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DIABLO CANYON POWER PLANT - UNIT 1 TEN YEAR EXAMINATION SUMMARY ASME SECTION XI SYSTEMS		REQUES REV. 7	T FOR RELIEF F	ROM CODE REQUIREMENTS	ADDENDA 1 SYSTEM PRESSURE TABLE 5.4A TEST SUMMARY PAGE 1 of 9		
	SYSTEM OR COMPONENT	CODE	SUM.	BASIS FOR REQUESTING	TESTING IN LIEU	TESTING	
NU.	DESCRIPTION	CLASS	NO.	RELIEF	OF REQUIREMENTS	SCHEDULE	
1	CENTRIFUGAL TYPE PUMPS - VARIOUS	2	19 24 53	PUMP MECHANICAL SEALS WILL NOT PERMIT HYDRO	ESTABLISH HYDRO BOUNDARY IN	TESTING TO BE SPECIFIED	
		3	55 4 14	SUCTION PIPING TEST PRESSURE.	1980 ASME SECTION XI IWA- 5224 (d).	FREQUENCY IN THE SUMMARY.	
2	DELETED						
3	DELETED						
4	EMBEDDED AUXILIARY SALTWATER PIPING	3 17 19	17 19	THE MAJORITY OF ASW PIPING IS UNDERGROUND AND NOT ACCESSIBLE FOR INSPECTION AND/OR TESTING. PRESERVICE TESTING WAS NOT REQUIRED BY CONSTRUC- TION CODE.	VISUAL LEAK TEST OF EXPOSED POR- TIONS OF SYSTEM DURING SYSTEM PRESSURE TESTS AS SPECIFIED IN THE SUMMARY	EACH INSPECTION INTERVAL	
					PERIODIC PUMP TESTING WILL VERIFY UNIM- PAIRED FLOW THROUGH THE INACCESSIBLE PORTIONS OF THE SYSTEM.	TESTING TO BE AS SPECIFIE IN THE PUMP INSERVICE TESTING PROGRAM.	
5	CLASS 2 SAFETY INJEC- TION, RESIDUAL HEAT REMOVAL,RCP SEAL INJECTION CHARGING AND BORON INJ PIPING	2	15 23 26 34 38 39 49 50 52	SOME OF THE PIPING IS NOT ISOLABLE FROM THE CLASS 1 PIPING.	THE UNISOLABLE PORTIONS WILL BE TESTED AT TEST PRESSURES DETERMINED BY THE CLASS 1 REQUIREMENTS.	TESTING TO BE AS SPECIFIED UNDER TEST FREQUENCY IN THE SUMMARY.	

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DIABLO CANYON POWER PLANT-UNIT 1

DIABLO CANYON POWER PLANT - UNIT 1 TEN YEAR EXAMINATION SUMMARY ASME SECTION XI SYSTEMS		REQUEST FO REV. 7	OR RELIEF FROM	A CODE REQUIREMENTS T	DDENDA 1 SYSTEM PRE ABLE 5.4A TEST SUMMA PAGE 2	SSURE RY of 9
NO.	SYSTEM OR COMPONENT DESCRIPTION	CODE CLASS	SUM. ITEM NO.	BASIS FOR REQUESTING RELIEF	TESTING IN LIEU OF REQUIREMENTS	TESTING SCHEDULE
6.	RCP SEAL RETURN PIPING FROM 8141A,B,C,D, TO THE RCP SEALS	2	12	PIPING IS NOT ISOLABLE FROM THE RCP SEALS AND CANNOT BE INCLUDED IN THE HYDROSTATIC PRESSUR TEST BOUNDARY. THE PIP ING IS PRESSURIZED DURI NORMAL REACTOR OPERATIO AND ANY LEAKAGE WOULD B DETECTED DURING OPERATI	NONE E NG N E ON.	NONE
7.	RHR PUMP SUCTION FROM CONTAINMENT SUMPS	2	41	PIPING IS OPEN ENDED TO THE CONTAINMENT AND CAN NOT BE ISOLATED TO PERF PRESSURE TESTS.	NONE	NONE
8.	NON-SAFETY RELATED SYSTEMS CONTAINMENT PENETRATIONS EXCEPT REFUELING FUEL TRANSFER TUBE.	2	VARIOUS	SEE PAGES 3 THRU 9 OF OF THIS TABLE.		
9.	FUEL TRANSFER TUBE TO REFUEL CANAL	2	78	NON-SAFETY RELATED SYSTEM, AS ABOVE. LINE IS INACCESSIBLE, ENCASED IN CONCRETE.	TEST OF INTEGRITY OF O-RING SEAL AT FLANGE	EVERY 3 1/3 YEARS

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DIABLO CANYON POWER PLANT - UNIT 1 TEN YEAR EXAMINATION SUMMARY ASME SECTION XI SYSTEMS REQUEST FOR RELIEF FROM CODE REQUIREMENTS REQUEST FOR RELIEF #8 REV. 7 ADDENDA 1 SYSTEM PRESSURE TABLE 5.4A TEST SUMMARY PAGE 3 of 9

A comparison of the requirements of visual examination VT-2 versus the requirements of 10 CFR APP. J local leak rate testing follows.

VISUAL EXAMINATION, VT-2

IWA-2212 Visual Examination General Requirements

- (a) The VT-2 visual examination shall be conducted to locate evidence of leakage from pressure retaining components, or abnormal leakage from components with or without leakage collection systems as required during the conduct of system pressure or functional test.
- (b) The VT-2 visual examination shall be conducted in accordance with IWA-5241 and IWA-5242 (see below).
 - NOTE: IWA-5241 and IWA-5242 clearly connotes liquid testing although a pneumatic test may be substituted by IWA-5211(e).

IWA-5241 Noninsulated Components

- (a) The visual examination, VT-2, shall be conducted by examining the accessible external exposed surfaces of pressure retaining components for evidence of leakage.
- (b) For components whose external surfaces are inaccessible for direct visual examination, VT-2, only the examination of surrounding area, including floor areas or equipment surfaces located underneath the components, for evidence of leakage shall be required.

IWA-5242 Insulated Components

- (a) The visual examination, VT-2, may be conducted without the removal of insulation by examining the accessible and exposed surfaces and joints of the insulation. Essentially vertical surfaces of insulation need only be examined at the lowest elevation where leakage may be detectable. Essentially horizontal surfaces of insulation shall be examined at each insulation joint.
- (b) For components whose external insulation surfaces are inaccessible for direct examination, only the examination of surrounding area, including floor areas or equipment surfaces located underneath the components, for evidence of leakage, or other areas to which such leakage may be channeled, shall be required.
- (c) Discoloration or residue on surfaces examined shall be given particular attention to detect evidence of boric acid accumulations from borated reactor coolant leakage.

NOTE: There is no requirement to remove insulation and a safety caution is invoked against removing insulation.

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DIABLO CANYON POWER PLANT-UNIT 1

DIABLO CANYON POWER PLANT - UNIT 1	REQUEST FOR RELIEF FROM CODE REQUIREMENTS	ADDENDA 1 SYSTEM PRESSURE
TEN YEAR EXAMINATION SUMMARY	REQUEST FOR RELIEF #8	TABLE 5.4A TEST SUMMARY
ASME SECTION XI SYSTEMS	REV. 7	PAGE of

10CFR50 APP. LOCAL LEAK RATE TESTING (TYPE C)

Type "C" test methods are similar to those described in ASME Section XI IWV-3424, except that the method, pressure, Acceptance Criteria, and schedule is specified and is in more detail.

IWV-3424 Seat Leakage Measurement

Valve seat leakage may be determined by one of the following:

- (a) Draining the line, closing the valve, bringing one side to test pressure, and measuring leakage through a downstream tell-tale connection, or
- (b) By measuring the feed rate required to maintain pressure between two valves or between two seats of a gate valve, provided the total apparent leak rate is charged to the valve or gate valve seat being tested, and that the conditions required by IWV-3423 (valve pressure test direction and exceptions) are satisfied.

DISCUSSION

LLRT is an instrumented test measuring leakage (ΔP) throughout the entire test boundary, including the penetration valves and the pipe segment inside the penetration itself. The VT-2 examination only checks the accessible outside surface of the pipe (or floors underneath) for evidence of leakage that is visually apparent.

<u>NOTE:</u> Although the ILRT examination demonstrates the collective integrity of the entire containment structure credit is not requested for these type "A" test which are performed at least three times each ten year interval.



PG&E				DIABLO	CANYON POWER F	PLANT-UNIT 1	
DIABLO CANYON POWER PLANT - UNIT 1 TEN YEAR EXAMINATION SUMMARY	REQUEST	FOR RELIEF	FROM CODE	REQUIREMENTS	ADDENDA 1	SYSTEM PRESSURE	
ASME SECTION XI SYSTEMS	REV. 7	FOR RELIEF	πο -		INDLE 5.4/	PAGE <u>5</u> of _	9

JUSTIFICATION

The lines on pages 7 through 9 are classified as ASME code class 2 solely because they penetrate containment and constitute part of the containment pressure boundary. They have no other safety related function. These lines can be considered in three categories:

- 1. Those that operate at 50 psi or less (See Pages 7 and 8). The scheduled Appendix J local leak rate tests (LLRT) at 50 psi, which are conducted at least once every 3 1/3 years, fully demonstrate all aspects of the components safety related containment integrity function. PG&E requests to substitute the LLRT for the VT-2 examination in these cases. If excessive leakage is discovered during the LLRT additional testing will be performed as necessary to identify the location of the leakage. This testing may be visual examination following application of leak detection fluid (snoop), extension of the test boundary to eliminate boundary valve leakage, use of ultrasonic leak detection equipment to detect valve seat leakage or other means. Appropriate repairs in accordance with ASME Section XI requirements will be made, and the LLRT will be reperformed until successful test results are obtained.
- 2. Those that are not normally in service, but may see operating pressures greater than 50 psi (See Page 9) during the occasions they are in use. These lines would nominally be required to have a system functional test at normal operating pressure once each 3 1/3 years. The concern is that since these lines can see pressures greater than 50 psi, a leak may develop at this higher pressure which would subsequently degrade the containment integrity function. For lines 1679, 3001 and 3729, which also receive the Appendix J LLRT tests at least once every 3 1/3 years. PG&E believes that a VT-2 examination during a system functional test once each 10 years adequately demonstrates system integrity at the higher operating pressure, while maintaining the plant's ALARA goals. PG&E therefore requests that the time period of the system functional test for these lines be increased from 3 1/3 to 10 years. The remainder of lines in this category (3935, 749, 2993, 4663, 3941) should be tested with the Appendix J LLRT each 3 1/3 years. The specific reasons for which PG&E requests to substitute LLRT for periodic system functional tests follow:
 - Line 3935: This line is isolated during all operating modes, and has never been in operation. Any existing leaks would be identified by LLRT and the line's integrity would not be challanged by system pressure until the next test.
 - Line 749: This line is seldom operated, and when in service normally sees only 27 psi, although the design maximum is 63 psi. The LLRT at 50 psi guarantees line integrity at nearly double its normal pressure.
 - Line 2993: This line operates only in mode 6, and its normal pressure is only 7 psi, although the design maximum is 54 psi. The LLRT at 50 psi guarantees its integrity at over seven times its normal pressure.
 - Line 4663: This system normally operates at less than 50 psi, although the design maximum is 80 psi. Again the LLRT assures line integrity; and the code examination would be based on the lower normal operating pressure.

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DIABLO CANYON POWER PLANT-UNIT 1

DIABLO CANYON POWER PLANT - UNIT 1REQUEST FOR RELIEF FROM CODE REQUIREMENTSADDENDA 1SYSTEM PRESSURETEN YEAR EXAMINATION SUMMARYREQUEST FOR RELIEF #8TABLE 5.4A TEST SUMMARYASME SECTION XI SYSTEMSREV. 7PAGE 6 of 9

Line 3941: This line is not in service in modes 1-4 and has strict administrative controls for its use. Containment integrity is not applicable when operating in modes 5-6. A pressure test using water could cause the following problems:

- 1.) Potential plant trip due to moisture in instrument air lines if any leaks by or does not get removed.
- 2.) Put water into the supply air to safety related equipment thereby damaging components.
- 3.) Get into the service air line and cause corrosion.

The LLRT is as effective for this system as the VT-2 examination, considering the limitations to VT-2 and normally isolated condition of the line.

- 3. Those that are normally in service at a pressure greater than 50 psi (See Page 9). These lines would nominally be required to have a hydrostatic test at 1.1 times the relief valve setpoint for that portion of the system, once each 10 years. Since all of the lines in this category are supply lines into containment, failure of these systems could result in;
 - a. leakage of the process fluid/gas into containment due to the greater system pressure than containment maximum pressure.
 - b. leakage of the process fluid outside containment, again due to the greater system pressure.

Neither of these conditions would result in containment leakage to the environment. Also, since the maximum pressure required for the safety related containment integrity function is less than 50 psi, the normal system pressure effectively constitutes a hydrostatic test for these lines. Furthermore, these lines again receive the Appendix J LLRT tests at least once each 3 1/3 years. PG&E therefore, requests that the system pressure requirement for these lines be reduced from 1.1 times the relief valve setpoint to the normal operating pressure of the system.

Additionally for line 531, a pneumatic hydrostatic test at 1100 psi (RV-290 setpoint is 1000 psi) would create the potential for serious injury to examination personnel. Since the line is continuously pressurized with 900 psi nitrogen, leakage would be continuously noticeable due to loss of inventory.

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DIAB TEN ASME	LO CANYON POWER PLANT - UNIT] YEAR EXAMINATION SUMMARY SECTION XI SYSTEMS	RFQUEST FO MAXIMUM OP REQUEST FO REV. 7	FQUEST FOR RELIEF FROM CODE REQUIREMENTS_ AXIMUM OPERATING PRESSURE < 50# EQUEST FOR RELIEF #8 EV. 7							ADDENDA 1 TABLE 5.4A	YSTEM PRESSURE TEST SUMMARY PAGE 7 OF9	
TEST NO.	SYSTEM OR COMPONENT DESCRIPTION	-	LINE	SIZE	FLUID	NORH PRESS	MAX PRESS	OPERAT Y/N	CODE REQ SF/HYD ¹	TESTING IN LIEU OF REQUIREMENTS	TESTING SCHEDULE	
70	PZR QUENCH TK GAS ANALYZER		1163	3/8	N2	20	SÖ	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS	
73	SI PP 1-2 DISCH LINE RV		2572	1	WTR	0	3	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS	
73	CNT SPRAY PPS DISCH RV OUT PRT		2518	1	WTR	0	9	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS	
73	CHG PPS SUCT RV DISCH		1459	1	WTR	Q	15	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS	
73	SIS RV OUTLET HDR TO PRT		2999	4	WTR	3	15	н	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS	
73	SIS RV OUTLET HDR TO PRT		2998	4	WTR	3	15	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS	
73	SI PPS SUCT RV OUT		2061	1	WTR	0	14	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS	
73	RHR HT EXCHR 2 OUT RV		2004	3	WTR	0	37	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS	
73	RHR HT EXCHR 1 OUT RV		2003	3	WTR	0	42	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS	
73	SIS PP 1-1 DISCH LINE RV		3851	1	WTR	0	12	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS	
73	SIS PPS RECIRC DISCH LINE RV		3852	1	WTR	0	10	N	SF	LLRT 🖗 50 PSI	EA. 3 1/3 YEARS	
86	REACT COOL DRN TK VENT		525	3/4	GAS	1.5	25	Y	HYD	LLRT 🛛 50 PSI	EA. 3 1/3 YEARS	
87	REACT COOL DRN TK GAS ANAL		526	1/2	AIR	1.5	25	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS	
89	CHPS AIR SUP FANS 162 DISCH. PE	NE 83 IN.	4382	4	AIR	3	20	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS	
89	CHPS AIR SUP FANS 182 DISCH. PE	NE 83 IN.	4386	4	AIR	3	20	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS	
89	CHPS AIR SUP FANS 1&2 DISCH. PE	NE 83 IN.	4388	4	AIR	3	20	N	SF	LLRT 🖶 50 PSI	EA. 3 1/3 YEARS	
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1 SF = SYSTEM FUNCTIONAL @ NOP @ 3 1/3 YRS.

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HYD = HYDRO @ 1.1 X PSV @ 10 YRS.



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PG&	E				D	TABLO CA	NYON POWER	PLAN	IT-UNIT 1	
DIA	BLO CANYON POWER PLANT - UNIT 1 REQUEST	FOR RELI	F FROM	CODE RE	OUTREMEN	15			ADDENDA 1 SI	STEM PRESSURE
TEN ASM	YEAR EXAMINATION SUMMARY MAXIMUM	OPERATING	G PRESS	URE < 50)≉				TABLE 5.4A T	EST SUMMARY
	REV. 7	FUK KELI	1 40						P/	40E 8 01 9
		· · · · ·					CODE	5		······································
TES	SYSTEM OR COMPONENT	1 7415	C 1 7F	CUUD	NORM	HAX	OPERAT REQ	.v.n1	TESTING IN LIEU	TESTING
<u></u>	ULSCRIPTION		<u>, , , , , , , , , , , , , , , , , , , </u>	FLUID	PRL35	PRESS		110.	UF REQUIREMENTS	SCHEDULE
89	CHPS AIR SUP FANS 182 DISCH. PENE 83 IN.	4389	۵	AJR	3	2(1	N SF	:	LLRT @ 50 PSI	EA. 3 1/3 YEARS
89	CHPS AIR SUP FANS 182 DISCH. PENE 83 IN.	4387	4	AIR	3	20	N SF	•	LLRT @ 50 PSI	EA. 3 1/3 YEARS
90	CHPS EXH AIR FILT 2 INLET	4390	4	AIR	5	20	N SF		LLRT @ 50 PSI	EA. 3 1/3 YEARS
91	CHPS EXH AIR FILT 2 INLET	4395	4	H2	0	20	N SF	-	LLRT @ 50 PSI	EA. 3 1/3 YEARS
97	CONTAINMENT VACUUM RELIEF	647	17	AIR	0	18	N SF	•	LLRT @ 50 PSI	EA. 3 1/3 YEARS
93	CONTAINMENT PURGE OUT	K2	48	AIR	< 50	< 5(1	N SF	•	LLRT 🕈 50 PSI	EA. 3 1/3 YEARS
94	CONTAINMENT PURGE IN	K2	48	AIR	< 50	< 50	N SF		LLRT @ 50 PSI	EA. 3 1/3 YEARS
95	INCORE CHILLER WATER RTN	3936	2	WTR	25	25	N SF		LLRT @ 50 PSI	EA. 3 1/3 YEARS
96	INCORE CHILLER WATER SUPPLY	3937	2	WTR	25	25	N SF		LLRT @ 50 PSI	EA. 3 1/3 YEARS
97	CONTAINMENT AIR SAMPLE INLET	3837	1	AIR	ָ 3	14.7	Y HY	Ď	LLRT @ 50 PSI	EA. 3 1/3 YEARS
98	CONTAINMENT AIR SAMPLE RETURN	3838	1	AIR	0	20	Y HY	D	LLRT @ 50 PSI	EA. 3 1/3 YEARS
99	POST-LOCA SAMP CNT AIR RTN	5190	3/8	AIR	< 50	< 50	N SF		LLRT @ 50 PSI	EA. 3 1/3 YEARS
109	POST-LOCA SAMP CNT AIR SUP	5.38	3/8	AIR	< 50	< 5(1	N SF		LLRT 🛛 50 PSI	EA. 3 1/3 YEARS
101	HYD MON CEL 82 CNT AIR SUP	4633	3/8	AIR	< 50	< 50	N SF		LLRT @ 50 PSI	EA. 3 1/3 YEARS
102	HYD MON CEL 82 CNT AIR SUP	4634	3/8	AIR	< 50	< 50	N SF		LLRT @ 50 PSI	EA. 3 1/3 YEARS
103	HYD MON CEL 83 CNT AIR SUP	4635	3/8	AIR	< 50	< 50	N SF		LLRT @ 50 PSI	EA. 3 1/3 YEARS
104 1 sf	HYD MON CEL 83 CNT AIR RTN = System Functional @ Nop @ 3 1/3 yrs.	4636 HYD =	3/8 Hydro ø	AIR 1.1 X P	< 50 SV @ 10	< 50 YRS.	N SF		LLRT @ 50 PSI	EA. 3 1/3 YEARS

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CANYON POWER PLANT-UNIT 1

DJAB TEN ASME	LO CANYON POWER PLANT - UNIT 1 R YEAR EXAMINATION SUMMARY M SECTION XI SYSTEMS R R	REQUEST FOR RELIN MAXIMUM OPERATING REQUEST FOR RELIN REV. 7	F FROM PRESS F #8	CODE RE URE > 5	QUIREMEN 0#	ADDENDA 1 SYSTEM PRESSURE TABLE 5.4A TEST SUMMARY PAGE 9_ of 9				
TEST NO.	SYSTEM OR COMPONENT DESCRIPTION	LINE	SIZE	FLUID	NORM PRESS	MAX PRESS (PSV)	OPERAT Y/H	CODE REQ SF/HYD ¹	TESTING IN LIEU OF REQUIREMENTS	TESTING SCHEDULE
71	PZR RELIEF TK N2 SUPPLY	1161	3/4	N2	108	150	Y	нур	SF @ NOP	EA. 10 YEARS
72	PRI WTR SUP TO PZR RELIEF TK	3000	3	WTR	128	140 (N/A)*	Y	HYD	SF @ NOP	EA. 10 YEARS
75	ACCUM TANKS N2 SUPPLY HDR	531	1	N2	900	1000 (1000)	¥	HYÐ	SF @ NOP	EA. 10 YEARS
76	ACCUM SAMPLE HDR	1679	3/8	WTR	650	656	N	SF	SF @ NOP	EA. 10 YEARS
79	AUX STEAM CONT PENETRATION	3935	2	STH	105+	110+	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS
80	FIREWATER SUPPLY BEFORE CONT	986	4	WTR	151	210 (N/A)+	Y	HYD	SF @ NOP	EA. 10 YEARS
81	CONT STR SUMP PPS DISCH	749	2	WTR	27	63	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS
82	REFUELING CANAL WATER INLET	3001	4	WTR	70	130	N	SF	SF @ NOP	EA. 10 YEARS
83	REFUELING CANAL WATER RETURN	2993	4	WTR	7	54	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS
84	REACT COOL DRN TK N2 SUPPLY	527	3/4	N2	108	165 (N/A)*	Y	HYD	SF @ NOP	EA. 10 YEARS
85	POST LOCA SAMPLE & R & CAVITY SUM	P 4663	1/4	WTR	< 50	80	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS
88	REACT COOL DRN PPS DISCH HDR	3729	2.5	WTR	80	145	N	SF	SF @ NOP	EA. 3 1/3 YEARS
105	SERVICE AIR PENE HDR	3941	3	AIR	100	130	N	SF	LLRT @ 50 PSI	EA. 3 1/3 YEARS
106	INSIDE CNT INSTR AIR SUP HDR	3242	2	AIR	100	110	Y	HYD	SF @ NOP	EA. 10 YEARS
107	CNT INST AIR SUP FCV-584 BP	4353	1.5	AIR	100	(110) 110 (110)	Y	HYD	SF @ NOP	EA. 10 YEARS
1 _{SF}	= SYSTEM FUNCTIONAL @ NOP @ 3 1/	3 YRS. + N	O RELI	EF VALVE	IN SYST	EM	+	LINE HAS	S NEVER BEEN IN SEE	RVICE
HY	D = HYDRO @ 1.1 X PSV @ 10 YRS.	T	EST @ :	1.1 X NO	P					

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PACIFIC GAS AND ELECTRIC COMPANY Diablo Canyon Power Plant ISI/NDE Department 420.9

Change to Diablo Canyon Unit <u>2</u> Inspection and Testing Program Plan

Date: 9/7/89

Diablo Canyon Unit _ 2 ISI and IST Program Plan Receipt Acknowledgement

M

You have been issued a Controlled Copy of the Diablo Canyon ISI and IST Program Plan. Here is Addenda 1 to Revision 7 of the plan. Please revise your controlled copy as follows:

CHANGE INSTRUCTIONS

Remove the following pages from the Program Plan binder and replace with Addenda 1.

Section 3.5 Tables 5.1, 5.2, and 5.3 Page 1 of 1 Test Summary Table 5.2 Pages 9,10,11,12, 13 of 13 Test Summary Table 5.3 Pages 2,3, and 4 of 4 Test Summary Table 5.4 Replace in its entirety.



For questions or assistance please phone the Document Control office, PG&E extension 691-4500, Pacific Bell phone (805) 595-4500.

After revising your copy of the Program Plan in accordance with these instructions, place the attached change description in the front of the manual behind the controlled copy numbered titled page.

Please acknowledge receipt of this revision by signing below and returning this form to:

Pacific Gas & Electric Co. Nuclear Regulatory Affairs 77 Beale/333 Market/A6093 San Francisco, CA 94106

Date: Receipt of Controlled Copy Addenda 1 to Revision # 4 of the Diablo Canyon Unit 2 ISI and IST Program Plan is hereby acknowledged.

RECIPIENT SIGNATURE



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