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 SCHUYLER,J.O. Pacific Gas & Electric Co.
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 KNIGHTON,G.W. Licensing Branch 3

SUBJECT: Forwards Rev 0 to "Stress Analysis of Two Piping Sys & Supports,Diablo Canyon Nuclear Power Plant,Unit 1," in response to SSER 20 open Item 18, "Piping Analysis Rept." Rept shows that design criteria me for all evaluations.

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J. O. SCHUYLER
VICE PRESIDENT
NUCLEAR POWER GENERATION

February 3, 1984

PGandE Letter No.: DCL-84-037

Mr. George W. Knighton, Chief
Licensing Branch No. 3
Division of Licensing
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Re: Docket No. 50-275, OL-DPR-76
Diablo Canyon Unit 1
SSER 20 Open Item 18
Piping Analysis Report

Dear Mr. Knighton:

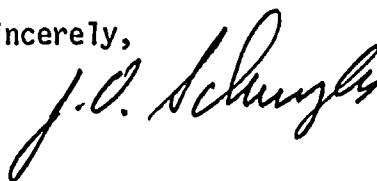
In SSER 20 Open Item 18, the Staff required that calculations for selected piping systems should be repeated with revised support configurations and current loadings to verify that piping and supports satisfy corresponding design criteria. The Staff required that this confirmatory analysis work be performed by an independent party.

Robert L. Cloud and Associates, Inc. (RLCA) performed the confirmatory analysis work on two piping systems selected by the Staff (Analyses 8-102 and 8-111). Enclosed is the RLCA final report, "Stress Analysis of Two Piping Systems and Supports, Diablo Canyon Nuclear Power Plant, Unit 1, Revision 0." The report shows that Diablo Canyon Project (DCP) design criteria are met for all the evaluations performed.

It should be noted that in the PGandE Phase I Final Report, the limiting load case for Analysis 8-102 was improperly identified as DDE (Table 2.2.1-3 page 2.2.1-26, Rev. 8). The correct limiting load case is thermal. The maximum stress ratio of 0.98 shown in the Final Report for Analysis 8-102 is the correct value for the thermal load case.

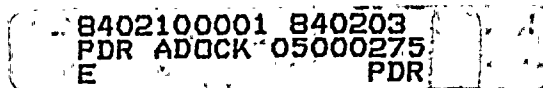
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Sincerely,



Enclosure

cc: D. G. Eisenhut
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February 3, 1984

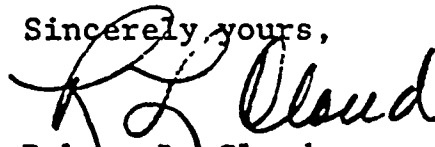
Mr. G. H. Moore
Project Engineer - Unit 1
Pacific Gas and Electric Company
Post Office Box 3965
San Francisco, CA 94119

Dear Mr. Moore:

Attached is an errata sheet for the RLCA Final Report, "Stress Analysis of Two Piping Systems and Supports...", Rev. 0. These corrections do not represent any content change, but are meant to present a more concise, understandable report.

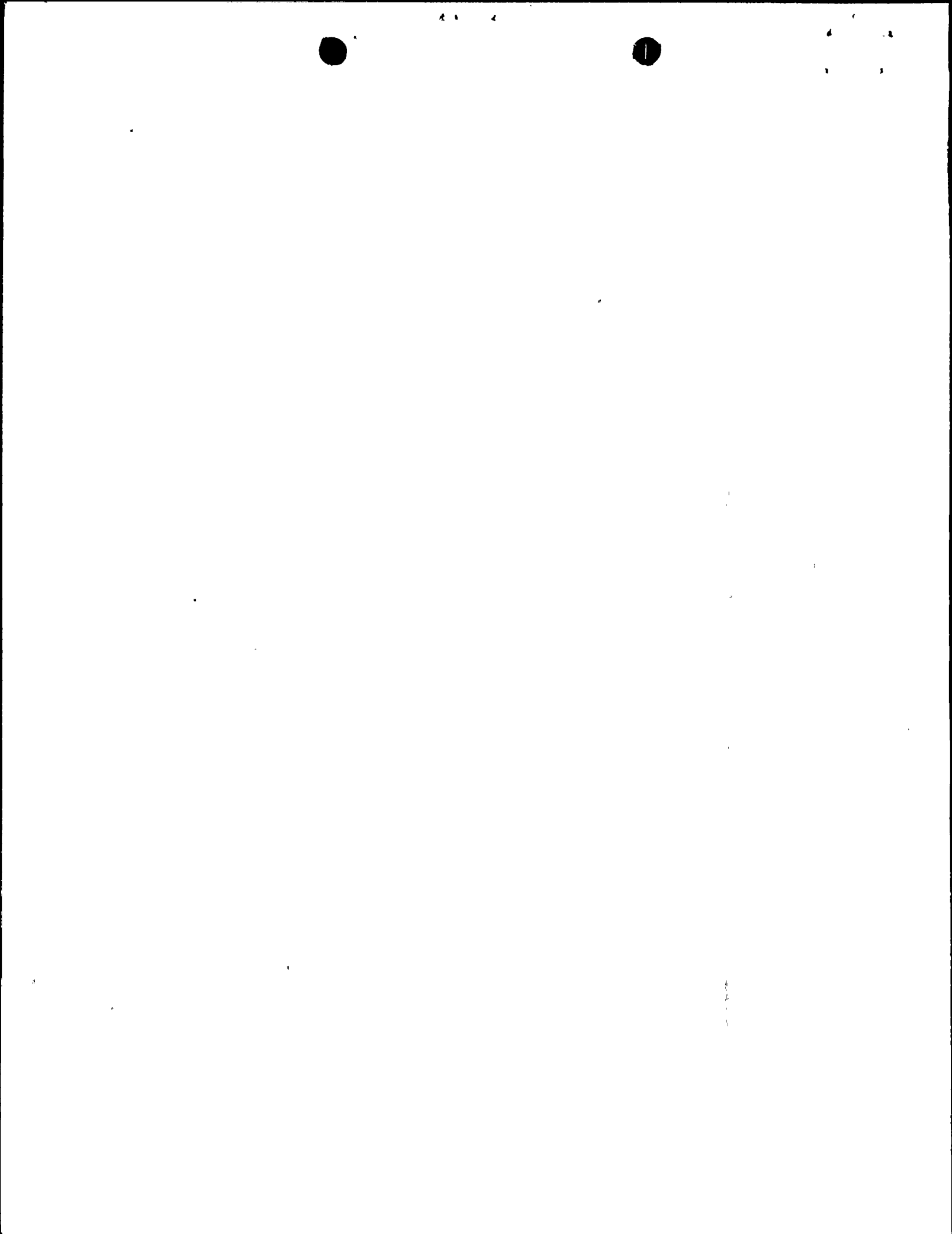
If you have any questions concerning the final report or the errata sheet, please contact me.

Sincerely yours,



Robert L. Cloud
RLCA

RLC/am
enclosure



Errata Sheet

Stress Analysis of Two Piping Systems and Supports, Diablo Canyon Nuclear Power Plant, Unit 1

1. Page iii: Table No. 5-3 should reference page 5-8,
not 5-7.
2. Page 2-1: Piping Analysis Program 8-102 is primarily
part of the Chemical Volume and Control System,
in addition to being part of the Residual Heat
Removal System and the Safety Injection System.
3. Page 4-8: Under Eq. 11 description, in definition of M_A ,
should read...joint thrust, not trust.

Under Eq. 12 description, term should read
 $\frac{PDo}{4t_n}$, not $\frac{PDo}{4t_t}$.
4. Page 5-5: In note 1, should read...Bulletin 107.
5. Page 5-7: Should be labeled page 5-8.
6. Page 5-8: Should be labeled page 5-7.

