

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 7901230145 DOC. DATE: 78/12/18 NOTARIZED: NO
 FACIL: 50-275 DIABLO CANYON #1, PACIFIC GAS & ELECTRIC CO.
 50-323 Diablo Canyon #2, Pacific Gas & Electric Co.
 AUTH. NAME AUTHOR AFFILIATION
 CRANE, P.A. PACIFIC GAS & ELEC
 RECIPIENT NAME RECIPIENT AFFILIATION
 STOLZ, J.F. ***LIGHT WATER REACTORS BRANCH 1

DOCKET #
 05000275
 05000323

SUBJECT: Responds to 780725 ltr & forwards results of component thermal analysis re steam line break accidents. Concludes qualification program provides safety margin for accidents.

DISTRIBUTION CODE: B0018 COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 8
 TITLE: PSAR/FSAR AMDTS AND RELATED CORRESPONDENCE.

NOTES: J HANCHETT w/1 CY ALL MATL.

	RECIPIENT ID CODE/NAME	COPIES LTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTR ENCL
ACTION:	05 PM <u>D ALLISON</u>	1 1	AD <u>VASSALLO</u>	1 0
	BC <u>LWR #1</u>	1 0	LA <u>LWR #1</u>	1 0
INTERNAL:	01 <u>REG FILE</u>	1 1	02 NRC PDR	1 1
	06 I & E	2 2	08 OPERA LIC BR	1 1
	09 GEOSCIEN BR	1 1	10 QAB	1 1
	11 MECH ENG BR	1 1	12 STRUC ENG BR	1 1
	13 MATL ENG BR	2 2	15 REAC SYS BR	1 1
	16 ANALYSIS BR	1 1	17 CORE PERF BR	1 1
	18 AUX SYS BR	1 1	19 CONTAIN SYS	1 1
	20 I & C SYS BR	1 1	21 POWER SYS BR	1 1
	22 AD SITE TECH	4 4	26 ACCDNT ANLYS	1 1
	27 EFFL TRT SYS	1 1	28 RAD ASMT BR	1 1
	29 KIRKWOOD	1 1	AD FOR ENG	1 0
	AD PLANT SYS	1 0	AD REAC SAFETY	1 0
	AD SITE ANLYSIS	1 0	DIRECTOR NRP	1 0
	MPA	1 0	OFLD	1 0
EXTERNAL:	03 LPDR	1 1	04 NSIC	1 1
	30 ACRS	16 16		

Ltr
 MOORE
 EPB #1 BC
 J JACKSON
 EPB #1 LA

JAN 24 1979

MA 4
 GD

TOTAL NUMBER OF COPIES REQUIRED: LTR 55 ENCL 45



1
2
3
4
5
6
7
8
9
10
11
12
13
14
15
16
17
18
19
20
21
22
23
24
25
26
27
28
29
30
31
32
33
34
35
36
37
38
39
40
41
42
43
44
45
46
47
48
49
50
51
52
53
54
55
56
57
58
59
60
61
62
63
64
65
66
67
68
69
70
71
72
73
74
75
76
77
78
79
80
81
82
83
84
85
86
87
88
89
90
91
92
93
94
95
96
97
98
99
100

PACIFIC GAS AND ELECTRIC COMPANY

PG&E + 77 BEALE STREET, 31ST FLOOR • SAN FRANCISCO, CALIFORNIA 94106 • (415) 781-4211

JOHN C. MORRISSEY
VICE PRESIDENT AND GENERAL COUNSEL

MALCOLM H. FURBUSH
ASSOCIATE GENERAL COUNSEL

CHARLES T. VAN DEUSEN
PHILIP A. CRANE, JR.
HENRY J. LAPLANTE
RICHARD A. CLARKE
JOHN B. GIBSON

ARTHUR L. HILLMAN, JR.
ROBERT OHLBACH
CHARLES W. THISSELL
ASSISTANT GENERAL COUNSEL

December 18, 1978

GILBERT L. HARRIS
GLENN WEST, JR.
DAN GRAYSON LUSBOCK
JACK F. PALMISTO
BERNARD J. DELLASANTA

EDWARD J. MCGANNEY
DANIEL E. DIBSON
JOSEPH L. KELLY
HOWARD V. JOLUS
JAMES C. LOBSON
SENIOR COUNSEL

JOSHUA BARILEY
DIANE BERGHAUSEN
STEVEN P. BURK
PAMELA CHAPPELLE
BRIAN S. DENTON
WILLIAM H. EDWARDS
JOSEPH B. ENGLERT, JR.
JOHN H. FEVE
PATRICK G. GOLDEN
ROBERT L. HARRIS
LEWIS H. KURTZ
ROBERT L. LINDBERG, JR.
RICHARD F. LOEKE
ROBERT S. MCLENNAN
RICHARD M. MOSS
ROBERT L. PETERS
ROBERT R. RICHETT
SHIRLEY A. SANDERSON
JUDITH SHAFER
DAVID J. WILKINSON
BRUCE R. WORTHINGTON

J. PETER BAUMGARTNER
ROBERT L. BERDON
LEIGH S. CARRIOY
HEATHER S. CISHA
MICHAEL G. DESMARAIS
GARY P. EVINAS
DONALD EISENBERG
DAVID C. JILBERT
PETER W. MANGSCHEN
JUAN M. JAYO
F. DONALD LAUPHEIMER
MERCK E. LIPSON
HARRY W. LONG, JR.
RICHARD L. MEIS
DOUGLAS A. DOLESBY
J. MICHAEL REIDENBACH
JOE E. SANDOZ
SUE ANN LEVIN SCHIFF
JACK W. SHUCK
SHIRLEY A. WOOD

ATTORNEYS

Mr. John F. Stolz, Chief
Light Water Reactors Branch No. 1
Division of Project Management
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

REGULATORY DOCKET FILE COPY

Re: Docket No. 50-275-OL
Docket No. 50-323-OL
Diablo Canyon Units 1 and 2

Dear Mr. Stolz:

Your letter dated July 25, 1978 requested additional information on the Diablo Canyon main steam line break (MSLB) analysis. Our letter of November 2, 1978 stated in part that we expect to satisfactorily qualify all safety related electrical equipment for a new MSLB analysis by performing component thermal analysis.

We have performed the component thermal analysis based upon the calculations made by the NRC for a typical containment. Attachment 1 summarizes review of MSLB using heat transfer approach. Figure 1 shows containment temperature with respect to time. Attachment 2 is a sample calculation of instrument thermal response to containment environment. Attachment 3 lists the qualification temperatures for Class IE equipment with potential for exposure to a main steam line break.

As indicated in the attachments, the calculated instruments and equipment temperatures are less than what would be expected from a design basis loss-of-coolant accident.

We conclude that the qualification program provides adequate margin for steam line break accidents.

7901230145

Boor
SE
1/71



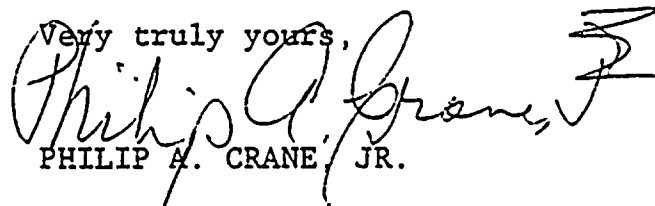
Mr. John F. Stolz

-2-

December 18, 1978

Kindly acknowledge receipt of the above material on the enclosed copy of this letter and return it to me in the enclosed addressed envelope.

Very truly yours,

A handwritten signature in cursive script, reading "Philip A. Crane, Jr.", with a large, stylized flourish at the end.

PHILIP A. CRANE, JR.

Enclosures

CC w/enc.: Service List



1950-1951
1952-1953
1954-1955
1956-1957
1958-1959
1960-1961
1962-1963
1964-1965
1966-1967
1968-1969
1970-1971
1972-1973
1974-1975
1976-1977
1978-1979
1980-1981
1982-1983
1984-1985
1986-1987
1988-1989
1990-1991
1992-1993
1994-1995
1996-1997
1998-1999
2000-2001
2002-2003
2004-2005
2006-2007
2008-2009
2010-2011
2012-2013
2014-2015
2016-2017
2018-2019
2020-2021
2022-2023
2024-2025

Environmental Temperature Qualifications
For
Containment Instrumentation

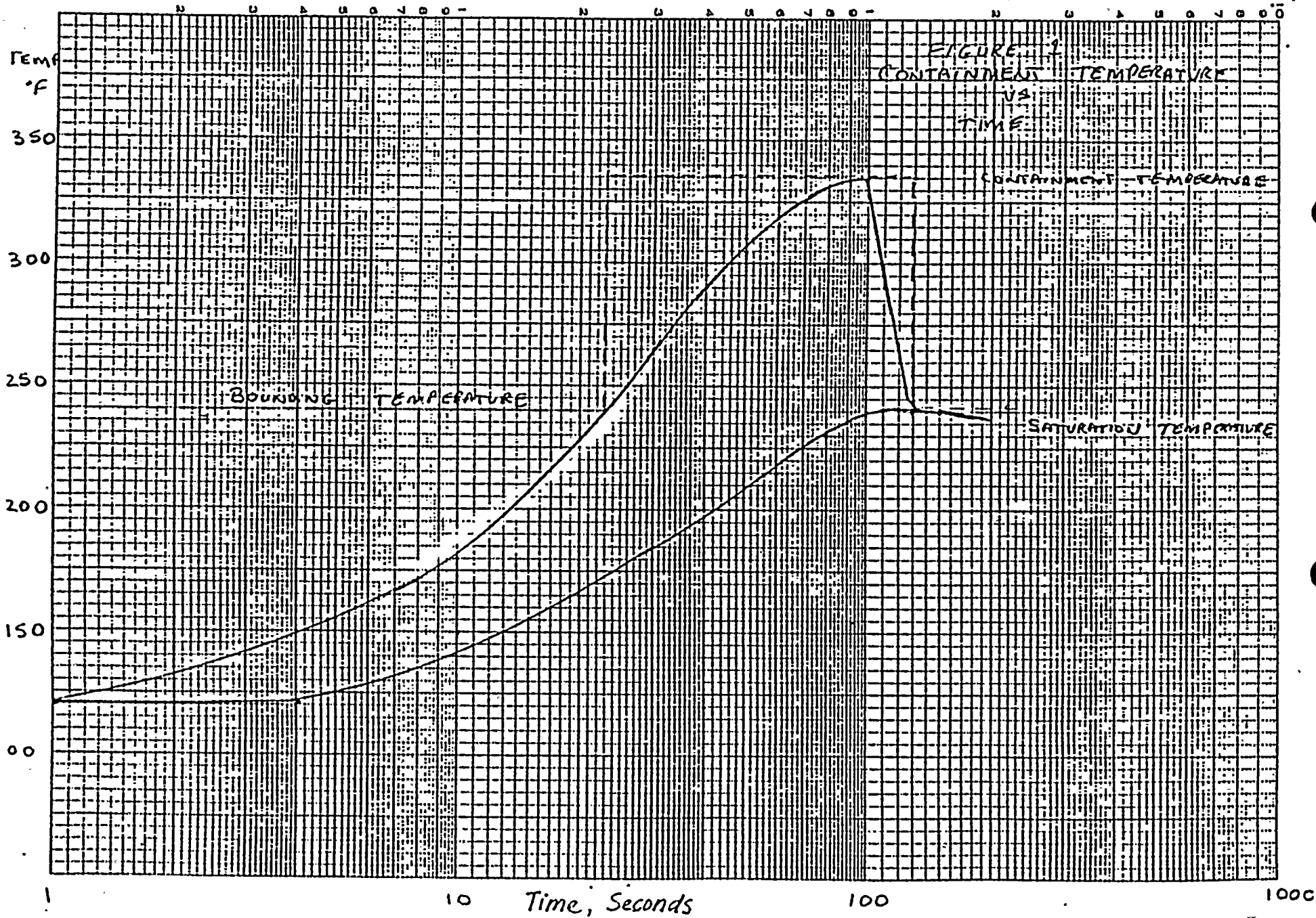
We have reviewed the environmental conditions (mainly temperature) that equipment listed in the table of Environment Qualifications of Class IE Equipment, submitted in a letter on May 3, 1978, would be subject to in the event of a main steam line break.

The instrument transmitters listed are protected more than those at most other reactor installations because of the protective cabinets used in the Diablo Canyon design. Although the instrument cabinets were provided for other purposes, they do afford insulation from the general containment environment in the event of a super-heated steam environment.

For purposes of calculating the environmental temperatures of the instrument transmitters and equipment, parameters were taken from a typical NRC Staff calculated containment atmosphere. The Staff's calculational results were obtained from an internal memorandum dated February 24, 1978, "Containment Environmental Qualification Best Estimates Evaluation for Main Steam Line Break Analysis." The basic parameters as used are contained in Figure 1. The assumptions and a sample calculation are shown in Attachment 2.

Using conservative heat transfer assumptions, calculations show that the peak surface temperature of the instrument transmitters would be about 252°F. This is less than the saturation temperatures resulting from a design basis LOCA. Using more realistic, best estimates, heat transfer coefficients would result in a peak temperature of about 230°F which is less than the saturation temperature of the steam environment. With regard to equipment in the containment, the peak temperatures were calculated to be 262°F. We have, therefore, concluded that the qualification program for the design basis LOCA environment provides adequate margin for a steam line break temperature excursion.







Sample Temperature Calculations

Assumptions

1. Material is steel flat plate geometry
2. Use infinite thermal conductivity -- no insulation effect of paint
3. Instrument wall thickness of $\frac{1}{4}$ inch

Heat Balance

$$WC_p \frac{dT}{d\theta} = hA(T_c - T)$$

Where:

- W = weight of instrument
 C_p = specific heat
 T = temperature
 \ominus = time
 h = convection heat transfer coefficient
 A = heat transfer area

Subscripts:

- C = temperature of containment atmosphere
 sat = saturation temperature of steam
 in = initial wall temperature

Referring to temperature response figure, it is assumed that the heat transfer process is in two distinct parts; that of steam condensing for the first 23 seconds followed by a heat up due to superheated steam in the second time period. In the first time period a high condensing heat transfer coefficient of 400 BTU/hr ft²OF was assumed and in the second time period a high steam heat transfer coefficient of 5 BTU/hr ft²OF was chosen

Performing calculation for 1 ft² area for first 23 seconds:

- W = 10.4
 C_p = 0.11
 h = 400
 T_{sat} = 240
 T_{in} = 120



$$T_{23} = T_{\text{sat}} - (T_{\text{sat}} - T_{\text{in}}) e^{-\frac{hA}{WC_p} \Theta}$$

$$T_{23} = 227$$

Or for ease of calculation it was assumed that at 23 seconds

$$T_{23} = 240$$

Calculating second time period to 130 seconds:

$$\begin{aligned} h &= 5 \\ T_{\text{in}} &= 240 \\ T_c &= 335 \\ \text{and } T_{130} &= 252 \end{aligned}$$



0

1
2
3
4
5
6
7
8
9
10
11
12
13
14
15
16
17
18
19
20
21
22
23
24
25
26
27
28
29
30
31
32
33
34
35
36
37
38
39
40
41
42
43
44
45
46
47
48
49
50
51
52
53
54
55
56
57
58
59
60
61
62
63
64
65
66
67
68
69
70
71
72
73
74
75
76
77
78
79
80
81
82
83
84
85
86
87
88
89
90
91
92
93
94
95
96
97
98
99
100

**QUALIFICATION TEMPERATURES
 FOR
 CLASS IE EQUIPMENT
 WITH
 POTENTIAL FOR EXPOSURE TO A MAIN STEAM LINE BREAK**

Sheet 1 of 3

Class IE Equipment Inside Containment - Subject to LOCA

<u>Equipment</u>	<u>Manufacturer</u>	<u>Type (Model No.)</u>	<u>Qualification Temperature</u>
1. Pressure and Differential Transmitters			
a. Pressurizer Pressure*	Rosemount	1152	350°F
b. Pressurizer Level	ITT Barton	764	320°F
c. Containment Sump Level	ITT Barton	764	320°F
d. Reactor Coolant System Wide Range Pressure	ITT Barton	763	320°F
e. Narrow Range Steam Generator Level	ITT Barton	764	320°F
f. Steam Flow*	Rosemount	1152	350°F
g. Sensor for Containment Pressure	Barton	351	320°F
2. Resistance Temperature Detector			
a. Reactor Coolant System Temperature	Sostman	118348-1	320°F
3. Valve Motor Operators			
	Limitorque	SHB-0, 00, 000	340°F
4. Containment Fan Cooler			
	Westinghouse	300/100 h.p.	326°F

*Required for Initiation Only

Attachment 3



0

2

1957-1958

<u>Equipment</u>	<u>Manufacturer</u>	<u>Type (Model No.)</u>	<u>Qualification Temperature</u>
5. Electrical Penetrations	General Electric	NS02/03/04	340°F
6. Electrical Cables	Continental	Silicon/Silicon	392°F
	Boston	Silicon/Hypalon	340°F*
	Raychem	Stilan	554°F
	Okonite	Tefzel	346°F
	Boston	Silicon Glass Briad/ Kapton/Hypalon	
7. Electrical Terminations	Raychem	Sealed Splice	357°F
8. Stem Mounted Limit Switches	Manco	EA180	340°F
9. Containment Isolation Solenoid Valves	ASCO	8300 8302 8316* 8321*	See FSAR Paragraph 3.11.3-3

*Special valves with all plastic parts replaced with stainless steel or brass parts to withstand higher temperatures.



***Note for Qualification of Boston Cable**

The Boston "Silicon Glass Braid/Kapton/Hypalon" insulated wires were tested in an autoclave at 302°F at 50 psig of saturated steam for 2 hours and 48 minutes. Insulation resistance was measured every 15 minutes during this period and found to be satisfactory at all times.

The insulation system of this conductor consists of silicon glass braid, 3 mils of Kapton tape, 6 mils of asbestos tape and an overall jacket of Hypalon. The silicon glass braid, Kapton and asbestos tapes are all capable of maintaining their properties at temperatures of 392°F. Hypalon, as an insulation for other conductors, has been qualified to 340°F.

