NRR-PMDAPEm Resource

From: Williams, Shawn
Sent: Monday, December 05, 2016 9:51 AM
To: 'gkmcelro@southernco.com'
Cc: Coleman, Jamie Marquess
Subject: Joseph M. Farley, Units 1 and 2 – Acceptance of Requested Licensing Action regarding TS 5.5.17, Extension of Type A and Type C Leak Rate Test Frequencies (CAC Nos. MF8844and MF8845)

Dear Mr. McElroy,

By letter dated November 15, 2016 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML16320A540), the Southern Nuclear Operating Company, Inc., (SNC) submitted an amendment request to revise the Joseph M. Farley Nuclear Plant, Unit 1 and Unit 2, Technical Specifications. The November 22, 2016, application replaced in its entirety the October 4, 2016 application (ADAMS Accession No. ML16280A294). Specifically, the proposed change is a request to revise TS 5.5.17 "Containment Leakage Rate Testing Program" to allow the following:

- Increase in the existing Type A integrated leakage rate test (ILRT) program test interval from 10 years to 15 years in accordance with Nuclear Energy Institute (NEI) Topical Report NEI 94-01, Revision 3-A and the conditions and limitations specified in NEI 94-01, Revision 2-A.
- Adopt an extension of the containment isolation valve (CIV) leakage testing (Type C) frequency from the 60 months currently permitted by 1 O CFR 50, Appendix J, Option B, to a 75-month frequency for Type C leakage rate testing of selected components, in accordance with NEI 94-01, Revision 3-A.
- Adopt the use of American National Standards Institute/American Nuclear Society (ANSI/ANS) 56.8-2002, Containment System Leakage Testing Requirements.
- Adopt a more conservative grace interval of 9 months, for Type A, Type B and Type C leakage tests in accordance with NEI 94-01, Revision 3-A.

The purpose of this letter is to provide the results of the U.S. Nuclear Regulatory Commission (NRC) staff's acceptance review. The acceptance review was performed to determine if there is sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical review. The acceptance review is also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

Consistent with Section 50.90 of Title 10 of the *Code of Federal Regulations* (10 CFR), an amendment to the license (including the technical specifications) must fully describe the changes requested, and following as far as applicable, the form prescribed for original applications. Section 50.34 of 10 CFR addresses the content of technical information required. This section stipulates that the submittal address the design and operating characteristics, unusual or novel design features, and principal safety considerations.

The NRC staff has reviewed your application and concluded that it does provide technical information in sufficient detail to enable the NRC staff to complete its detailed technical review and make an independent assessment regarding the acceptability of the proposed amendment in terms of regulatory requirements and the protection of public health and safety and the environment. The application made numerous references to updating the Probabilistic Risk Assessment (PRA) model to credit the Generation III RCP [Reactor Coolant Pump] seals after Topical Report PWROG-14001-P/NP, Revision 1, "PRA Model for the Westinghouse Shutdown Seal" (Topical Report) is approved. However, at this time, the Topical Report is not yet approved, therefore the NRC will review the application without consideration of the Topical Report. Given the lesser scope and depth of the acceptance review as compared to the detailed technical review, there may be instances in which issues that impact the NRC staff's ability to complete the detailed technical review are

identified despite completion of an adequate acceptance review. You will be advised of any further information needed to support the NRC staff's detailed technical review by separate correspondence.

Based on the information provided in your submittal, the NRC staff has estimated that this licensing request will take approximately 490 hours to complete. The NRC staff expects to complete this review in approximately 12 months. If there are emergent complexities or challenges in our review that would cause changes to the initial forecasted completion date or significant changes in the forecasted hours, the reasons for the changes, along with the new estimates, will be communicated during the routine interactions with the assigned project manager. These estimates are based on the NRC staff's initial review of the application and they could change, due to several factors including requests for additional information, unanticipated addition of scope to the review, and review by NRC advisory committees or hearing-related activities. Additional delay may occur if the submittal is provided to the NRC in advance or in parallel with industry program initiatives or pilot applications.

If you have any questions, please contact me at (301) 415-1009.

Shawn Williams, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-348 and 50-364

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