## SIMULATOR SCENARIO #1

Appendix D Scenario Outline Form ES-D-1 Facility: Millstone Unit 2 Scenario No.: 1 Op-Test No.: ES16LI1 SRO Examiners: Operators: \_\_\_\_\_ ATC BOP Initial Conditions: 100% Power IC-139, No Equipment OOS, Ch-Y PZR Level in service. Turnover: 100% Power, steady state, no equipment OOS. 24E is aligned to 24C. Critical Tasks: 1. SPTA-5; Manually shutdown the reactor 2. Fully implement Once Thru Cooling prior to either SG reaching 32" Event No. Malf. No. **Event Description** Event Type\* 1 Ν N/A Start "B" TBCCW Pump, Secure "A" TBCCW Pmp. (BOP/S) 2 03A1A5S2 С Trip "A" CEDM Cool Fan. C04L-B11B (ATC/S) 3 Т RP10A Ch. "A" PZR Pressure fails low (TS) (ATC/S) 4 Т RX04A Ch-X PZR Level (LT110X) (non-selected) fails to 0% level. (TS) (ATC/S) 5 С FW01 Main Condenser Vacuum leak. (BOP/S) 6 R N/A Downpower due to vacuum leak. (All) RC11A, "A" RCP seizes and trips, TCBs fail to open (ATWS), manual Rx trip 7 Μ RP04A-D, pushbuttons fail, manually trip by opening MG set breakers. (All) RP27B FW33, С 8 ES01A, Rapid loss of condenser vacuum and failure of AFAS to trigger. (BOP/S) ES01B 9 FW36A, C (BOP/S) AFW pipe rupture at FW-44, inops both headers, results in LOAF. FW36B TS(S) (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Tar	get Quantitative Attributes (Per Scenario; See Section D.5.d)	Actual
1.	Total malfunctions (5–8)	6
2.	Malfunctions after EOP entry (1–2)	2
3.	Abnormal events (2–4)	4
4.	Major transients (1–2)	1
5.	EOPs entered/requiring substantive actions (1–2)	1
6.	EOP contingencies requiring substantive actions (0–2)	0
7.	Critical tasks (2–3)	3

## NRC 2016, Scenario 1 Summary:

The crew will take the shift with the unit at 100% power, steady state, with no equipment out of service (IC-30). TBCCW Pumps: "A" and "C" running, "B" in standby and ready to be started. The crew has been instructed to start the "B" TBCCW pump (Standby Equipment) and secure the "A" TBCCW pump.

**Event 1**: Upon taking the shift, the crew has been instructed to swap a running TBCCW with the Standby pump. The BOP will be directed to start the "B" TBCCW pump (Standby Equipment) and secure the "A" TBCCW pump, verifying no change in TBCCW flow. Once this is accomplished, Event 2 will be triggered.

**Event 2**: The "A" CEDM Cooling Fan will trip, triggering the CEDM Cooling Fan Trip annunciator. The crew will respond per ARP 2590C-082 and the ATC will be directed to start the standby ("B") CEDM cooling fan. Once this is accomplished, Event 3 will be triggered.

**Event 3**: The "A" Safety Channel of Pressurizer Pressure will fail low as a Tech. Spec. only event. The US should address the applicable Tech. Specs. and ARP, which directs all systems affected by the transmitter failure be "bypassed". Once the crew has bypassed Ch. "A" on RPS, ESAS and AFAS, Event 4 is triggered.

**Event 4**: The Non-Selected (Ch. "X") pressurizer level detector (LT-110X) will fail to 0%, causing all pressurizer heaters to trip. The crew will respond per ARP 2590B-215, de-select the failed channel and reset all pressurizer heaters. Once the crew has restored normal RCS pressure control, Event 5 will be triggered.

**Event 5**: Main Condenser Vacuum will begin to degrade. The crew should enter AOP 2574 for Loss Of Condenser Vacuum, and take the applicable actions to increase condenser air removal capacity. Once this proves ineffective, the US will enter AOP 2575, Rapid Downpower, and commence a plant shutdown.

**Event 6:** The crew will commence a plant shutdown using AOP 2575, Rapid Downpower, in an attempt to stabilize condenser vacuum by lower the energy load on the main condenser. The crew will insert CEAs to start the power reduction and then continue the downpower by boric acid injection into the RCS. This will be accomplished by aligning charging pump suction directly to the RWST (instead of the VCT). When the power change evaluation is completed, Event 7 will be triggered.

**Event 7:** "A" RCP will seize and trip, but the TCBs will fail to open (ATWS). The crew should recognize the ATWS and immediately trip the reactor manually by pressing the four TCB manual trip buttons. These will also fail to open the TCBs, requiring the opening of the MG Set supply breakers. The crew should then verify all CEAs are inserting (reactor trip successful), the main turbine has tripped and commence Standard Post Trip Actions per EOP 2525.

**Event 8:** During the performance of SPTA, Main Condenser vacuum will degrade rapidly, causing the loss of Main Feedwater Pumps and Condenser Steam Dumps. This will require the use of the Auxiliary Feedwater System to feed the S/Gs. The AFAS will fail to automatically start AFW flow to the S/Gs, requiring manual actuation of AFW flow. Once Aux. Feedwater is manually aligned to feed both S/Gs, Event 9 is triggered.

**Event 9:** Shortly after the feed flow has been established to the S/Gs using Auxiliary Feedwater, a rupture will occur on both sides of 2-FW-44 (normally open x-tie between AFW headers), resulting in the loss of both Auxiliary Feedwater headers. Once STPA are completed, and the LOAF is diagnosed, the crew will transition to EOP 2537, Loss Of All Feed, and discus using a Condensate Pump to feed the S/Gs. This will require a plant cooldown using the available SG inventory, to lower SG pressure below the shutoff head of the condensate pumps.

	INPUT SUMMARY						
Either IN	Either INPUT or VERIFY the following functions:						
ID Num	Description	Delay Time	Ramp Time	Event Time	Sev or Value	Final Value	Rel Order
	MALI	FUNCTIO	NS				
RP04A- RP04D	Failure of all four Manual Trip Buttons			N/A			0
RP27B	Failure of ALL RPS trip functions			N/A			0
ES01A, ES01B	AFAS Failure, both Facilities			N/A			0
C04L- B11B	F-13A Trip Alarm			E-2		ON	2
RP10A	Ch. "A" PZR Pressure fails low			E-3	100%		3
RX04A	Ch. "X" PZR Level Cont. fails low			E-4	0%		4
FW01	Main Condenser Vacuum Loss			E-5	0.5"		5
FW01	Main Condenser Vacuum Loss			E-6	0.3"		6
RC11A	"A" RCP Seizes and tirps			E-7			7
FW33	Rapid loss of Condenser Vacuum			E-30	100%		8
FW36A FW36B	Rupture Aux Feedwater, both headers			E-9	100%		9
REMOTE FUNCTIONS							
TPR02	"A" TBCCW Pump discharge valve			E-10	CLOSED		1
TPR02	"A" TBCCW Pump discharge valve			E-11	OPEN		1

The scenario will ended when the crew has recovered SG feed flow using a Condensate Pump, or at the Examiners discretion.

	OVERRIDES					
03A1A5 S2	Trip the "A" CEDM Cooling Fan, F-13A					2

Op-Test No	.: <u>ES16LI1</u>	Scenario No.:1 Event No.: _1
Event Desc	ription: Start tl	he "B" TBCCW Pump, Secure the "A" TBCCW Pump
Time	Position	Applicant's Actions or Behavior
		ew has been instructed to brief the starting of the Standby ("B") TBCCW A" TBCCW pump prior to taking the watch.
	ving steps are s for applicab	e from OP 2330B. OP procedure is marked up with "N/A" and Unit Supervisor ble steps.
	BOP/PEO	4.1.1 ESTABLISH communications between Operators at TBCCW pumps and Control Room.
		tablish communications with the control room as the PEO on station at the ed, local conditions appear ready for starting the "B" TBCCW pump.
	BOP/PEO	<ul> <li>4.1.2 To start standby TBCCW Pump, PERFORM the following: <ul> <li>a. PLACE selected "TBCCW PUMP" switch to "START" (C-06).</li> <li>b. CHECK the following for running pump: <ul> <li>No abnormal noise or vibration (local)</li> <li>Normal motor amperage (C-06)</li> <li>Maintains normal system pressure (C-06)</li> <li>Normal system flow as indicated on FI-6272 (TBCCW HX area)</li> </ul> </li> </ul></li></ul>
Simulator	Operator: Re	port as the PEO on station, "B" TBCCW pump is running no abnormal noise.
	BOP	<ul> <li>4.1.3 IF desired to stop 'A' TBCCW Pump, PERFORM the following: <ul> <li>a. CLOSE 2-TB-3A, "TBCCW PUMP 'A' DISCHARGE STOP" (TBCCW HX area).</li> <li>b. PLACE P-7A, 'A' "TBCCW PUMP" switch to "STOP" OR "PULL-TO-LOCK"(C-06).</li> <li>c. OPEN 2-TB-3A, "TBCCW PUMP 'A' DISCHARGE STOP" (TBCCW HX area).</li> </ul> </li> </ul>
		the PEO on station, operate TB-3A as directed. ED], Event-11 [TPR02 OPEN]
	BOP	<ul> <li>4.1.4 IF desired to stop 'B' TBCCW Pump, PERFORM the following: <ul> <li>a. CLOSE 2-TB-3B, "TBCCW PUMP 'B' DISCHARGE STOP" (TBCCW HX area).</li> <li>b. PLACE P-7B, 'B' "TBCCW PUMP" switch to "STOP" OR "PULL-TO-LOCK"(C-06).</li> <li>c. OPEN 2-TB-3B, "TBCCW PUMP 'B' DISCHARGE STOP" (TBCCW HX area).</li> </ul> </li> <li>Examiner Note: This step is N/A</li> </ul>
Examiner	BOP Note: When	<ul> <li>4.1.5 IF desired to stop 'C' TBCCW Pump, PERFORM the following: <ul> <li>a. CLOSE 2-TB-3C, "TBCCW PUMP 'C' DISCHARGE STOP" (TBCCW HX area).</li> <li>b. PLACE P-7C, 'C' "TBCCW PUMP" switch to "STOP" OR "PULL-TO-LOCK"(C-06).</li> <li>c. OPEN 2-TB-3C, "TBCCW PUMP 'C' DISCHARGE STOP" (TBCCW HX area).</li> </ul> </li> <li>Examiner Note: This step is N/A</li> <li>"B" TBCCW Pump has been started and the "A" TBCCW Pump has been</li> </ul>
secured, o	or at the lead	examiner's direction, proceed to Event #2, Trip of "A" CEDM Cooling Fan.

Op-Test No.:	ES16LI1	Scenario No.:1_ Event No.: _2	
Event Descri	ption: "A" CEI	DM Cooler Fan Trip	
Time	Position Applicant's Actions or Behavior		
Simulator	Operator: W	hen directed, initiate Event #2, "A" CEDM Cooling Fan trip.	
	s Available: DM COOLER	FAN TRIP (C-04, BB-11).	
Examiner I	Note: The fol	llowing steps are from ARP 2590C-082.	
	ATC	<ol> <li>Start idle CEDM cooling fan (C-04).</li> <li>MONITOR the following:         <ul> <li>"A" CEDM fan air discharge temperature on computer point, "T8102"</li> <li>"B" CEDM fan air discharge temperature on computer point, "T8106"</li> <li>"C" CEDM fan air discharge temperature on computer point, "T8101"</li> <li>If all three CEDM cooling units fail, Go To OP 2204, "Load Changes", and PERFORM applicable actions to initiate a plant and reactor shutdown.</li> </ul> </li> <li>SUBMIT CR to Electrical Maintenance Department to investigate cause of fan trip.</li> </ol>	
	ked to invest is tripped on	igate and determine the status of "A" CEDM Fan breaker, report back that over current.	
	Examiner Note: When the CEDM fan trip has been mitigated or at lead examiner's direction, go to Event 3 Ch. "A" RCS Pressure failure.		

Op-Test No.:	ES16LI1	Scenario No.:1 Event No.: _3
Event Descri	ption: Failure	of Safety Ch. "A" PZR Pressure (low)
Time	Position	Applicant's Actions or Behavior
Simulator (	Operator: Wh	en directed, initiate Event 3, Failure of Ch. "A" RCS Pressure (PT-102A).
Indications	Available:	
<ul> <li>PZR PR</li> <li>TM-LP 1</li> <li>RPS PR</li> <li>PZR PR</li> <li>ESAS C</li> </ul>	ES LO BLOC TRIP CH A (C E TRIP (C-04 ESS LO LO A COMPONENT	
		AUTOMATIC FUNCTIONS 1. If 2 RPS channels actuate, reactor trips.
	ATC	<ol> <li>NOTE</li> <li>1. TMLP Trip may be manually bypassed when power is less than 1 x 10-4%. However, if power increases above 1 x 10-4%, bypass is automatically removed.</li> <li>2. Pressurizer pressure is an input to ATWS logic circuitry.</li> </ol>
	ATC	<ol> <li><u>IF</u> reactor trips, Go To EOP 2525, "Standard Post Trip Actions" and PERFORM necessary corrective actions.</li> <li>OBSERVE channel "A" pressurizer pressure indication and TMLP setpoint and COMPARE to other safety channel indications (C-03, PPC, ESAS).</li> <li>OBSERVE channel "A" TH and TC instruments indicating properly for present conditions (C-03, PPC, RPS).</li> <li><u>IF</u> pressurizer pressure or RCS temperature is abnormal AND no automatic reactor trip has occurred, manually TRIP reactor and Go To EOP 2525, "Standard Post Trip Actions."</li> <li>Examiner Note: Applicant observes that Ch. "A" PZR Pressure failed low by comparing the four Pressure Safety Channels on C-02/3 PZR mimic.</li> </ol>
Simulator C	Operator: If d	ispatched as a PEO to check indication on C-21, report PI-102A is failed low.

Time	Position	Applicant's Actions or Behavior
Event Descri	ption: Failure	of Safety Ch. "A" PZR Pressure (low)
Op-Test No.:	ES16LI1	Scenario No.:1_ Event No.: _3

ATC/BOP	<ul> <li>5. <u>IF</u> alarm is due to any instrument malfunction, PERFORM the following:</li> <li>5.1. <u>IF</u> pressure instrument malfunctioned, OBTAIN necessary keys and PERFORM applicable actions to bypass the following channel "A" pressurizer pressure outputs:</li> <li>TMLP Trip (RPS)</li> <li>High Pressurizer Pressure Trip (RPS)</li> <li>ESAS pressurizer pressure bistable</li> <li>ATWS (C-100)</li> </ul> Examiner Note: The US may chose to have either the ATC or the BOP carry out the above step to bypass the affected safety channels.
ATC/SRO	<ul> <li>5.2. <u>IF</u> temperature instrument malfunctioned, OBTAIN necessary keys and PERFORM applicable actions to bypass the following channel "A" temperature outputs:</li> <li>TMLP Trip (RPS)</li> <li>High Power Trip (RPS)</li> <li>Local Power Density Trip (RPS)</li> <li>5.3. Refer To the following Technical Specifications LCOs and DETERMINE applicability: <ul> <li>3.3.1.1, Table 3.3-1</li> <li>3.3.2.1, Table 3.3-3</li> <li>3.3.3.5, Table 3.3-9 (PI-102A only)</li> <li>3.3.3.8, Table 3.3-11 (PI-102B only)</li> </ul> </li> <li>5.4. Refer To the following TRM LCOs and DETERMINE applicability: <ul> <li>3.3.1.1</li> </ul> </li> </ul>
RO or BOP	At ESAS Sensor cabinet "A", PRESS SIAS/CIAS keyswitch is turned from OPER to INHIBIT

Op-Test No.	: <u>ES16LI1</u>	Scenario No.:1 Event No.: _3
Event Descri	iption: Failure	of Safety Ch. "A" PZR Pressure (low)
Time	Position	Applicant's Actions or Behavior
	1	
		Review Tech. Specs. and TRM:
		<b>LCO 3.3.1.1 (RPS):</b> As a minimum, the reactor protective instrumentation channels and bypasses of Table 3.3-1 shall be OPERABLE.
		<b><u>APPLICABILITY</u></b> : As shown in Table 3.3-1.
		ACTION: As shown in Table 3.3-1.
		Per TS Table 3.3-1: FUNCTIONAL UNIT #4 Pressurizer Pressure - High; Total Number of Channels = 4, Minimum Channels Operable = 3, App. Modes = 1&2, Action = 2
		<u>Action 2</u> - With the number of OPERABLE channels one less than the Total Number of Channels, operation may continue provided the following conditions are satisfied:
		<b>a</b> . The inoperable channel is placed in either the bypassed or tripped condition within 1 hour. The inoperable channel shall either be restored to OPERABLE status, or placed in the tripped condition, within 48 hours.
		b. Within 1 hour, all functional units receiving an input from the inoperable channel are also declared inoperable, and the appropriate actions are taken for the affected functional units.
	SRO	c. The Minimum Channels OPERABLE requirement is met; however, one additional channel may be removed from service for up to 48 hours, provided one of the inoperable channels is placed in the tripped condition.
		<b>Examiner Note:</b> SRO should note TSAS 3.3.1, Action 2 applies and is presently being met by the actions taken per the ARP to bypass the affected channels.
		LCO 3.3.2.1 (ESAS): The engineered safety feature actuation system instrumentation

**APPLICABILITY**: As shown in Table 3.3-3.

## ACTION:

a. With an engineered safety feature actuation system instrumentation channel trip setpoint less conservative than the value shown in the Allowable Values column of Table 3.3-4, either adjust the trip setpoint to be consistent with the value specified in the Trip Setpoint column of Table 3.3-4 within 2 hours or declare the channel inoperable and take the ACTION shown in Table 3.3-3.

channels and bypasses shown in Table 3.3-3 shall be OPERABLE with their trip setpoints

set consistent with the values shown in the Trip Setpoint column of Table 3.3-4.

b. With an engineered safety feature actuation system instrumentation channel inoperable, take the ACTION shown in Table 3.3-3.

Per TS Table 3.3-3: FUNCTIONAL UNIT #1c, Pressurizer Pressure - Low; Total Number of Channels = 4, Minimum Channels Operable = 3, App. Modes = 1,2,3a, Action = 2

Time	Position	Applicant's Actions or Behavior	
Event Descri	ption: Failure	of Safety Ch. "A" PZR Pressure (low)	
Op-Test No.:	ES16LI1	_ Scenario No.: <u>1</u> Event No.: <u>3</u>	

		<ul> <li>Action 2 - With the number of OPERABLE channels one less than the Total Number of Channels, operation may continue provided the following conditions are satisfied:</li> <li>a. The inoperable channel is placed in either the bypassed or tripped condition within 1 hour. The inoperable channel shall either be restored to OPERABLE status, or placed in the tripped condition, within 48 hours.</li> </ul>
		b. Within 1 hour, all functional units receiving an input from the inoperable channel are also declared inoperable, and the appropriate actions are taken for the affected functional units.
		c. The Minimum Channels OPERABLE requirement is met; however, one additional channel may be removed from service for up to 48 hours, provided one of the inoperable channels is placed in the tripped condition.
		<b>Examiner Note:</b> SRO should note TSAS 3.3.2.1b, Action 2 applies and is presently being met by the actions taken to bypass the affected channels.
		<b>TS 3.3.3.5 (PI-102A - High Range):</b> The remote shutdown monitoring instrumentation channels shown in Table 3.3-9 shall be OPERABLE with readouts displayed external to the control room.
		APPLICABILITY: MODES 1, 2 and 3.
	SRO	ACTION:
		With the number of OPERABLE remote shutdown monitoring instrumentation channels less than required by Table 3.3-9, either:
		a. Restore the inoperable channel to OPERABLE status within 7 days, or
		b. Be in HOT SHUTDOWN within the next 24 hours.
		<b>Examiner Note:</b> SRO should note that one of the two channels of PZR Pressure on C-21 is <i>not</i> OPERABLE. However, Table 3.3-9 only requires <u>one</u> channel be OPERABLE, which there still is (PI-102B). Therefore, the TS is still met
		TS 3.3.3.8 (Acc. Monitoring): SRO should note that this TS is Not Applicable.
		<ul> <li>TRM 3.3.1.1.1: All pressurizer high pressure reactor protection channels shall be FUNCTIONAL.</li> <li><u>APPLICABILITY</u>: In accordance with Technical Specification LCO 3.3.1.1 Applicability.</li> </ul>
		ACTION: Restore any nonfunctional pressurizer high pressure reactor protection channel to FUNCTIONAL status within 30 days of placing the channel in the tripped condition, or be in MODE 3 within the next 6 hours with the failed channel in the bypassed condition.
		<b>Examiner Note:</b> SRO should note that the Action Requirements of this TRM applies while the instrument channel is not OPERABLE per TS 3.3.1.1.
Examiner Note	: When th	ne instrument failure has been evaluated and mitigated, or at the lead

Examiner Note: When the instrument failure has been evaluated and mitigated, or at the lead examiner's direction, proceed to Event 4, Ch. "X" PZR Level failure.

Op-Test No.:	ES16LI1	Scenario No.:1 Event No.: _4
Event Descri	ption: : Failure	e of Control Ch. "X" PZR Level (low)
Time	Position	Applicant's Actions or Behavior
Simulator C Level to 0%		nen directed, initiate Event 4, Failure of the Non-Selected Channel ("X") of PZR
	SSURIZER C	CH X LEVEL LO LO (C-02/3, C-38) CH X LEVEL HI/LO (C-02/3, A-38)
		Iowing steps are from ARP 2590B-215, PRESSURIZER CH X LEVEL LO LO. SURIZER CH X LEVEL HI/LO (ARP 2590B-213) are used, skip to the next page. AUTOMATIC FUNCTIONS
	ATC	1. <u>IF</u> "SEL SW" is in "X+Y" position, <i>all</i> heaters de-energize.
	ATC	<ul> <li><u>CORRECTIVE ACTIONS</u></li> <li>1. OBSERVE actual level on pressurizer level recorder, LR110, pressurizer level controllers (C03) and PPC.</li> <li>2. <u>IF</u> annunciator is <i>not</i> valid, SHIFT pressurizer level control to channel "Y." 2.1. SHIFT pressurizer heater control "SEL SW" to channel Y.</li> <li>2.2. As necessary, RESET the following Pressurizer heater breakers: <ul> <li>"PROP HTR GROUP 1"</li> <li>"BACKUP HTRS GROUP 2"</li> <li>"BACKUP HTRS GROUP 2"</li> <li>"BACKUP HTRS GROUP 3"</li> <li>"BACKUP HTRS GROUP 4"</li> </ul> </li> <li>Examiner Note: Because this is an <i>instrument</i> failure of the Non-Controlling channel, and does not reflect an actual change in PZR level, the only Effect is the PZR Heater Breakers tripping open. Therefore, steps 3 – 8 of this ARP are not applicable.</li> </ul> <li>9. <u>IF</u> alarm was caused by channel X malfunctioning, SUBMIT Trouble Report to I&amp;C Department.</li>

Time Position	Applicant's Actions or Behavior	
Event Description: : Failure of Control Ch. "X" PZR Level (low)		
Op-Test No.: <u>ES16LI1</u>	Scenario No.:1_ Event No.: _4	

Examiner Note: The following steps are from ARP 2590B-213, PRESSURIZER CH X LEVEL HI/LO.			
	ATC	<ul> <li><u>AUTOMATIC FUNCTIONS</u></li> <li>1. <u>IF</u> level is high: <ul> <li><i>All</i> backup heater energize</li> <li><i>All</i> proportional heaters go to maximum output.</li> <li>Backup charging pumps stop.</li> <li><u>If</u> level is low, backup signal is generated to start <i>both</i> backup charging pumps.</li> </ul> </li> </ul>	
		CORRECTIVE ACTIONS CAUTION While restoring Pressurizer level, Pressurizer pressure must be closely monitored to prevent exceeding DNB limits, or effects on reactivity due to pressure changes.	
	ATC	<b>NOTE</b> Actual Pressurizer level response should track with pressure response. 1. OBSERVE Pressurizer levels on all channels (C-03). 2. OBSERVE Pressurizer pressure on all channels (C-03).	
		<b>NOTE</b> Failure of the non-controlling Channel X low, will de-energize pressure heaters if heater "SEL SW" is in "X+Y" position.	

Time	Position	Applicant's Actions or Behavior	
Event Descrip	otion: : Failure	e of Control Ch. "X" PZR Level (low)	
Op-Test No.:	ES16LI1	_ Scenario No.: <u>1</u> Event No.: <u>4</u>	

	3. <u>IF</u> L110X is the noncontrolling channel, and a controller or transmitter failure is indicated in the low direction, PERFORM the following:
	3.1. SHIFT pressurizer heater control "SEL SW" to "Y"
	3.2. As necessary, RESET the following Pressurizer heater breakers:
	"PROP HTR GROUP 1"
	"PROP HTR GROUP 2"
ATC	BACKUP HTRS GROUP 1"
	BACKUP HTRS GROUP 2"
	BACKUP HTRS GROUP 3"
	BACKUP HTRS GROUP 4"
	3.3. IF desired, COMMENCE forcing Pressurizer sprays.
	3.4. Go To Step [13] <b>{Note: a typographical error exists in the procedure</b> at this step, in that the words "NO TAG" appear in place of "13".}
ATC	<ul> <li>13. IF alarm was caused by a controller or transmitter malfunction, DETERMINE appropriate channel, and SUBMIT Trouble Report to Instrumentation &amp; Control Department.</li> <li>14. To determine ACTION Statement requirements, Refer To the following LCOs: <ul> <li>TS 3.2.6 (for DNB)</li> <li>TS 3.3.5 (for HSD or C-21)</li> </ul> </li> </ul>
	<ul> <li>TS 3.3.3.8 (for Acc Monitoring)</li> </ul>
	<ul> <li>TS 3.4.4 (LT110X Only)</li> <li>TRM 7.1.4 (LT110X Only)</li> </ul>

Op-Test No.:	ES16LI1	Scenario No.:1_ Event No.: _4		
Event Descri	Event Description: Failure of Control Ch. "X" PZR Level (low)			
Time	e Position Applicant's Actions or Behavior			
		Review Technical Specifications:		
		<b>LCO 3.2.6 (DNB):</b> The DNB margin shall be preserved by maintaining the cold leg temperature, pressurizer pressure, reactor coolant flow rate, and AXIAL SHAPE INDEX within the limits specified in the CORE OPERATING LIMITS REPORT.		
		ACTION:		
		With any of the above parameters exceeding its specified limits, restore the parameter to within its above specified limits within 2 hours or reduce THERMAL POWER to $\leq 5\%$ of RATED THERMAL POWER within the next 4 hours.		
		Examiner Note: Impacted only if RCS pressure dropped below 2225 psia.		
		<b>TS 3.3.3.5 (HSD):</b> The remote shutdown monitoring instrumentation channels shown in Table 3.3-9 shall be OPERABLE with readouts displayed external to the control room.		
		APPLICABILITY: MODES 1, 2 and 3.		
		ACTION:		
		With the number of OPERABLE remote shutdown monitoring instrumentation channels less than required by Table 3.3-9, either:		
		c. Restore the inoperable channel to OPERABLE status within 7 days, or		
		d. Be in HOT SHUTDOWN within the next 24 hours.		
	SRO	<b>Examiner Note:</b> The SRO should note that one of the two channels of PZR Level on C-21 is <i>not</i> OPERABLE. However, Table 3.3-9 only requires <u>one</u> channel be OPERABLE, which there still is (L110Y). Therefore, the TS is still met.		
		TS 3.3.3.8 (Acc. Monitoring): SRO should note that this TS is Not Applicable.		
		<b>TS 3.4.4 (PZR):</b> The pressurizer shall be OPERABLE with:		
		a. Pressurizer water level $\leq 70\%$ , and		
		b. At least two groups of pressurizer heaters each having a capacity of at least 130 kW.		
		APPLICABILITY: MODES 1, 2 and 3.		
		ACTION:		
		a. With only one group of pressurizer heaters OPERABLE, restore at least two groups to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 12 hours.		
		b. With the pressurizer otherwise inoperable, be in at least HOT STANDBY with the reactor trip breakers open within 6 hours and in HOT SHUTDOWN within the following 6 hours.		
		<b>Examiner Note:</b> The SRO should note the need to log into TSAS "a" at the approximate time the Proportional Heater breakers tripped, and log out at the approximate time they were reclosed.		

Time	Position	Applicant's Actions or Behavior
Event Descri	ption: : Failure	e of Control Ch. "X" PZR Level (low)
Op-Test No.:	ES16LI1	Scenario No.:1 Event No.: _4

		<b>TRM 7.1.4 (App R Safe Shutdown):</b> The Appendix R Safe Shutdown Related (ARSR) equipment listed in the TRM Table 7.1.4-1 shall be FUNCTIONAL.				
		APPLICA	BILITY: MO	DDES 1, 2, 3, and 4.		
		<b>Examiner Note:</b> SRO should note the need to reference the TRM when time allows, as the requirements for these actions are not applicable unless the component is not functional for 14 days.				
		ACTION:				
	SRO	With an ARSR component listed in the above referenced component table nonfunctional (unable to meet its intended Appendix R shutdown function), take the ACTION as specified in the above table under Compensatory Measures.				
		Examiner Note: Only applicable part of the Table 7.1.4-1 is included.				
		TR Item	Comp ID	FUNCTIONALITY Description	Compensatory Measures if Component Not Restored in 14 days	
		E	LT-110X	Loop LT-110X must be FUNCTIONAL from Control Room.	With loop LT-110X nonfunctional from Control Room, perform ACTIONS b.1, b.2 for fire area R-2, R-10, and R-15.	
			be discusse		Measures, contained on TRM page scretion, or following the completion of	
					eclosed and the SRO has finished ner's direction, proceed to Event 5,	

Main Condenser Vacuum Leak.

Op-Test No.:	Op-Test No.: <u>ES16LI1</u> Scenario No.: <u>1</u> Event No.: <u>5, 6</u>				
Event Description: Main Condenser Vacuum Leak					
Time	Position	Applicant's Actions or Behavior			

Simulator Operator: When directed, initiate Event 5, Main Condenser Vacuum Leak.

Indications:

- "COND VACUUM LO" {PPC, C-06/7, A-37}
- Condensate pressure indicator and recorder show an unexplained rise in condenser pressure. {PPC, C-05}
- Unexplained drop in electric megawatts.

Examiner Note: The following steps are from ARP 2590E-185, "COND VACUUM LO", C-06/7, A-37 annunciator.

<u>A</u>	UTOMATIC FUNTIONS
1.	. None
<u><u>c</u></u>	CORRECTIVE ACTIONS
1.	<ul> <li>IF condenser pressure rise is due to slow fouling of the condenser due to seasonal changes, Refer To OP 2204, "Load Changes," and REDUCE Reactor power and turbine load to clear "COND VACUUM LO" annunciator.</li> </ul>
SRO/BOP 2	<ul> <li><u>IF</u> degraded condenser vacuum is being directed during performance of a power ramp, PERFORM the following:</li> </ul>
	2.1. NOTIFY personnel controlling condenser pressure of the alarm, the value and trend of condenser pressure.
	2.2. DIRECT personnel controlling condenser pressure to recover vacuum to the applicable control band.
3.	. IF steps 1. or 2. recover condenser vacuum, EXIT this ARP.
4	. Go To AOP 2574, "Loss of Condenser Vacuum."

Examiner Note: Operators should enter AOP 2574, Loss of Condenser Vacuum, prior to beginning a plant downpower, as there was no reason given to suspect condenser fouling.

Examiner Note: The following steps are from AOP 2574, Loss of Condenser Vacuum.

Op-Test No.: <u>ES16LI1</u> Scenario No.: <u>1</u> Event No.: <u>5, 6</u>			
Event Descri	Event Description: Main Condenser Vacuum Leak		
Time	Position	Applicant's Actions or Behavior	

1	
SRO	<ul> <li>3.1 IF ANY of the following conditions exist:</li> <li>Reactor power and turbine load is lowered to 30% power <u>AND</u> condenser pressure is still greater than 5 inches of mercury absolute</li> <li>Condenser pressure is approaching trip setpoint (7.5 inches of mercury absolute) <u>AND</u> reactor power is greater than or equal to 15%</li> <li>PERFORM the following: <ul> <li>Manually TRIP the reactor and turbine.</li> <li>Go To EOP 2525, "Standard Post Trip Actions."</li> </ul> </li> </ul>
	NOTE
	When power is less than 15% AND level "1" bistable light clears (not lit), on at least 3 RPS channels, the turbine trip is inhibited and turbine trip will not result in an automatic reactor trip.
	3.2 IF condenser pressure is approaching trip setpoint (7.5 inches of mercury absolute) AND reactor power is less than 15%, with turbine trip bypassed, manually TRIP the turbine.
	<ul> <li>3.3 IF condenser pressure is rising to 10 inches of mercury absolute, PERFORM the following:</li> <li>a. ENSURE reactor power less than 3%.</li> <li>b. Refer To OP 2322, "Auxiliary Feedwater System," and INITIATE AFW flow to SGs.</li> <li>c. TRIP running SGFPs.</li> </ul>
	3.4 IF, at any time, efforts to restore vacuum are not successful, Refer To AOP 2575, "Rapid Downpower," and LOWER reactor power and turbine load at the maximum attainable rate until pressure stabilizes.
	3.5 NOTIFY ISO New England of the loss or imminent loss of unit.
BOP	<ul> <li>4.1 VERIFY performance of condenser air removal as follows: <ul> <li>a. VERIFY F55A or F55B operating</li> <li>b. VERIFY condenser air removal fan discharge path is aligned per ONE of the following: <ul> <li>EB-55 AND EB-56, condenser air removal to Millstone stack, are open</li> <li>EB-57, condenser air removal to Unit #2 stack, is open</li> <li>C. VERIFY the operating SJAE steam supply pressure is 200- 220 psig.</li> <li>d. Refer To OP 2329, "Condenser Air Removal," and VERIFY both sets of SJAE's in service.</li> </ul> </li> </ul></li></ul>

Op-Test No.: <u>ES16LI1</u> Scenario No.: <u>1</u> Event No.: <u>5, 6</u>				
Event Description: Main Condenser Vacuum Leak				
Time	Position	Applicant's Actions or Behavior		

r	-	
		NOTE
		Use of the mechanical vacuum pumps may cause a rise in backpressure and mask the original leak and may cause the East condenser pit sump to overflow.
		<ul> <li>4.2 IF condenser backpressure is greater than 4 inches Hg absolute, PERFORM the following: <ul> <li>a. START mechanical vacuum pumps "A" and "B."</li> <li>b. VERIFY local vacuum gage is 27 inches Hg or greater.</li> <li>c. OPEN 2- AR- 11, "MECHANICAL VACUUM PUMP COMBINED SUCTION".</li> <li>d. OPEN 2- AR- 12A, "PUMP 'A' SUCTION STOP".</li> <li>e. OPEN 2- AR- 12B, "PUMP 'B' SUCTION STOP".</li> <li>f. IF condenser air removal fan, F55A, is available, THEN PERFORM the following: <ol> <li>START condenser air removal fan, F55B.</li> <li>ENSURE EB- 171, MAKE- UP DMPR", is closed.</li> </ol> </li> </ul></li></ul>
If BOP dire pumps hav	cts a PEO to ve been align	align the mechanical vacuum pumps, wait a couple minutes and then say the ed per Step 4.2 (do NOT actually perform the alignment). ower FW01 to 0.3" ***
Examiner Note: At some point the Unit Supervisor should determine that efforts to restore vacuum are not being successful, enter AOP 2575, "Rapid Downpower", and direct a reactor and turbine shutdown in an attempt to lower the main condenser heat loading.		
Examiner Note: The following steps are from AOP 2575, Rapid Downpower, Section 3.0 Rapid Downpower.		
	SRO	Enters AOP 2575, Rapid Downpower

Op-Test No.: <u>ES16LI1</u> Scenario No.: <u>1</u> Event No.: <u>5, 6</u>				
Event Description: Main Condenser Vacuum Leak				
Time	Position	Applicant's Actions or Behavior		

SRO	<ul> <li>3.1 PERFORM focus brief on the following:</li> <li>REACTOR TRIP CRITERIA <ul> <li>Parameters associated with automatic reactor or turbine trips are challenged</li> <li>RCS T cold <i>not</i> within 10°F of temperature program and efforts to regain control are unsuccessful</li> </ul> </li> <li>RCS TEMPERATURE CONTROL <ul> <li>RCS T cold to be maintained within 10°F of Attachment 5, "Temperature vs. Power program" using Attachment 10, "Main Turbine Load Set Control."</li> <li>To avoid uncontrolled cooldowns or power transients, sudden changes in RCS temperature or boron concentration should be avoided.</li> </ul> </li> <li>3.2 REQUEST SM/STA to Refer To Attachment 8, "Required Notifications," and PERFORM notifications.</li> </ul>
ATC	3.3 INITIATE forcing pressurizer sprays.
	CAUTION In the case of a dropped CEA, rod motion is <i>not</i> used to initiate downpower.
ATC	3.4 IF <i>not</i> downpowering due to a dropped rod, INSERT Group 7 CEAs $10 \pm 2$ steps to initiate downpower.
BOP	3.5 Using the "Load Speed Control" switch, REDUCE turbine load to maintain Tc on program (+/-2 deg).
SRO	3.6 Refer To PPC or Reactor Engineering Curve and Data Book and OBTAIN reactivity plan for the initial reactor power condition and desired load reduction.
Examiner Note:	The crew should refer to Reactivity Plan for downpower parameters.

Op-Test No.: <u>ES16LI1</u> Scenario No.: <u>1</u> Event No.: <u>5, 6</u>				
Event Description: Main Condenser Vacuum Leak				
Time	Position	Applicant's Actions or Behavior		

		NOTE Attachment 10 "Approximate Load Demand vs. Reactor Power," can be used to correlate the desired power level to a turbine load demand setpoint.
	BOP	3.7 Refer To Attachment 9, "Main Turbine Load Set Control," REDUCE turbine load and MAINTAIN Tc on program (+/-2 deg).
Examiner N Load Set C		owing steps are from AOP 2575 Rapid Downpower Attachment 9 Main Turbine
	BOP	CAUTION Operation of the "Load/Speed CONTROL" switch will change turbine NOTE Steps provided in this attachment are dependent on plant conditions
	BOP	<ol> <li><u>IF</u> desired to commence or modify a turbine load ramp, PERFORM the following (HMI "Load" screen):         <ul> <li><u>IF</u> previous ramp has stopped, SELECT "Load Hold."</li> <li>SELECT "Load Setpt" and ENTER desired value.</li> <li>SELECT "Rate setpt" and ENTER desired value.</li> <li><u>WHEN</u> ready to commence load reduction, SELECT "Load Resume."</li> </ul> </li> </ol>

Op-Test No.: <u>ES16LI1</u> Scenario No.: <u>1</u> Event No.: <u>5, 6</u>			
Event Description: Main Condenser Vacuum Leak			
Time	Position	Applicant's Actions or Behavior	

	BOP	<ul> <li>2. <u>IF</u> desired to adjust the "Load Ramp Rate," PERFORM <i>any</i> of the following:</li> <li>SELECT "Rate setpt" and ENTER new value.</li> <li>SELECT "5% / hour," <u>OR</u> "10% / hour," <u>OR</u> "20% / hour."</li> <li>SELECT "Raise" or "Lower" (0.25% / hour change).</li> </ul>
		a. IF Tavg and Tc are high off program, PERFORM the following:
		a. SELECT "Load Hold" to stop ramp.
		<ul> <li><u>WHEN</u> Tavg and Tc are trending back to program, SELECT "Load Resume."</li> </ul>
		b. IF Tavg and Tc are low off program, PERFORM the following:
		a. JOG the "Load/Speed CONTROL" switch to "Lower."
		<ul> <li><u>WHEN</u> Tavg and Tc are back on program, SELECT Load Setpt" and ENTER desired value.</li> </ul>
		c. IF desired, Go To Step 1 and RESUME turbine load ramp.
		c. IF desired load has been reached SELECT "Load Hold."
		Examiner Note: operator should select x load setpoint, x load rate. Program band for Tavg and Tc is x (+/- 2 deg for Tc).
Examiner N Downpowe		owing steps are from AOP 2575 Rapid Downpower Section 3.0 Rapid
	ATC	3.8 Based on required rate of downpower, START additional charging pumps as necessary and balance charging and letdown.
1		

Op-Test No.: <u>ES16LI1</u> Scenario No.: <u>1</u> Event No.: <u>5, 6</u>			
Event Description: Main Condenser Vacuum Leak			
Time	Position	Applicant's Actions or Behavior	

ATC	<ul> <li>3.9 IF desired to borate from the RWST (preferred method)</li> <li>PERFORM the following: <ul> <li>a. ENSURE at least one charging pump operating.</li> <li>b. ENSURE CH-196, VCT makeup bypass, closed.</li> <li>c. ENSURE CH-504, RWST to charging suction, open.</li> <li>d. OPEN CH-192, RWST isolation.</li> <li>e. CLOSE CH-501, VCT outlet isolation.</li> <li>f. CHECK charging flow at desired rate.</li> <li>g. Go To step 3.11</li> </ul> </li> <li>Examiner Note: Crew should borate from the RWST.</li> </ul>
 O/ATC/ BOP	<ul> <li>3.11 During the downpower, Refer To Attachment 1, "Rapid Downpower Parameters," and MAINTAIN parameters as specified throughout downpower:</li> <li>Examiner note: Attachment 1 Rapid Downpower Parameters:</li> <li>Condensate and heater drain flows and pressures: sufficient to maintain adequate SGFP suction pressure</li> <li>FRV D/P: greater than 40 psid</li> <li>Turbine load: responding to changes in load demand, with control valves operating together</li> <li>Steam generator levels 55 to 70%.</li> <li>MSR parameters tracking together</li> <li>Turbine Generator MVARs: as specified by CONVEX</li> <li>Reactor power: being monitored using delta T power indication</li> <li>ASI: In accordance with reactivity plan or within 0.01 of ESI or per COLR.</li> <li>CEA position: greater than PDIL</li> <li>Tc: less than or equal to 549 deg</li> <li>Pressurizer pressure: between 2,225 and 2,300 psia (DNB margin)</li> </ul>

Op-Test No.: <u>ES16LI1</u> Scenario No.: <u>1</u> Event No.: <u>5, 6</u>				
Event Description: Main Condenser Vacuum Leak				
Time	Position	Applicant's Actions or Behavior		

SRO/A	тс
	NOTE
	<ol> <li>Xenon rate of change should be considered when terminating boration.</li> </ol>
	<ol> <li>During rapid downpower, the PPC calorimetric may be inaccurate due to SG level transients. The most accurate available indication of reactor power is RPS delta T power.</li> </ol>
Examiner Note: Onc Event 7, RCP trip an	e power has dropped at least 5%, or at the lead examiner's direction, proceed to d ATWS

Time	Position	Applicant's Actions or Behavior		
Event Descri	Event Description: RCP trip, ATWS and total loss of condenser vacuum.			
Op-Test No.:	Op-Test No.: <u>ES16LI1</u> Scenario No.: <u>1</u> Event No.: <u>7, 8</u>			

Simulator (	Operator: Wh	en directed, initiate Event 7 & 8, "A" RCP seize and trip, ATWS, Loss of MFW.
• "RC	P A MOTOR <sup>.</sup> P LO SPEED	TRIP" {C-02/3, AA-17} TRIP CH A" {C-04, AA-4} RIP CH *" {* = All four Channels, A – D; C-04: CA-2, CB-2, DA-2, DB-2}
reactor will	•	rator should attempt to trip the reactor using the manual pushbuttons. The ause the pushbuttons fail. An operator will open the CEDMS output breakers ally.
output break	kers immediat	Ily Shutdown the reactor. The reactor must be manually tripped using the CEDM ely (within approximately 1 minute) when an automatic reactor trip fails and/or the NOT work. (CT-1/SPTA-5)
		outtons attempted:s opened (reactor trip):
	ATC	<ul> <li>Reports alarms on "A" RCP, RCP tripped</li> <li>Reactor failure to auto trip, tripping the reactor manually by push buttons</li> <li>Failure of TCBs to manually open, <b>opening MG Set supply breakers (CT)</b></li> <li>Reactor trip successful, CEAs inserting.</li> </ul>
	SRO	Acknowledge need to trip, directs (or acknowledges) reactor trip
Examiner N to improve		owing steps are from EOP 2525, Standard Post Trip Actions, modified slightly
	ATC	<ul> <li>Determine Status of Reactivity Control – Reactor Trip</li> <li>1. DETERMINE that Reactivity Control acceptance criteria are met for the reactor by performing ALL of the following steps: <ul> <li>CHECK that all CEAs are fully inserted.</li> <li>CHECK that reactor power is dropping.</li> <li>CHECK that SUR is negative.</li> </ul> </li> </ul>

Op-Test No.:	ES16LI1	Scenario No.:1_ Event No.: _7, 8		
Event Description: RCP trip, ATWS and total loss of condenser vacuum.				
Time	Position	Applicant's Actions or Behavior		

	Determine Status of Reactivity Control – Turbine Trip
BOP	<ul> <li>2. DETERMINE that Reactivity Control acceptance criteria are met for the turbine by performing ALL of the following steps : <ul> <li>a. CHECK that the main turbine is tripped by BOTH of the following:</li> <li>ALL main stop valves are closed.</li> <li>Generator megawatts indicate zero.</li> <li>Turbine speed is lowering.</li> </ul> </li> </ul>
	<ul> <li>b. <u>IF</u> 15G-2XI-4, motor operated disconnect, is closed, CHECK that the main Generator output breakers 8T and 9T are open.</li> </ul>
BOP	<ul> <li>Determine Status of Maintenance of Vital Auxiliaries</li> <li>3. DETERMINE that Maintenance of Vital Auxiliaries acceptance criteria are met by performing ALL of the following steps: <ul> <li>a. CHECK that ALL Facility 1 and 2 electrical buses are energized:</li> <li>6.9kV Electrical Buses 25A, 25B</li> <li>4.16kV Non-Vital Electrical Buses 24A, 24B</li> <li>4.16vV Vital Electrical Buses 24C, 24D</li> <li>Vital DC Buses 201A, 201B, DV-10, DV-20</li> <li>Vital AC Instrument Buses VA-10, VA-20</li> </ul> </li> <li>b. CHECK that BOTH facilities of service water are operating.</li> <li>c. CHECK that BOTH facilities of RBCCW are operating with service water cooling.</li> </ul>

Op-Test No.:	ES16LI1_	Scenario No.: <u>1</u> Event No.: <u>7, 8</u>	
Event Description: RCP trip, ATWS and total loss of condenser vacuum.			
Time	Position	Applicant's Actions or Behavior	

ATC	Determine Status of RCS Inventory Control
110	<ol> <li>DETERMINE that RCS Inventory Control acceptance criteria are met by performing ALL of the following:</li> </ol>
	a. CHECK that <b>BOTH</b> of the following conditions exist:
	Pressurizer level is 20 to 80%
	<ul> <li>Pressurizer level is trending to 35 to 70%</li> </ul>
	a.1 IF the Pressurizer Level Control System is not operating properly in automatic, RESTORE and MAINTAIN pressurizer level 35 to 70% by performing ANY of the following:
	1) OPERATE the Pressurizer Level Control System.
	2) Manually OPERATE charging and letdown.
	b. CHECK that RCS subcooling is greater than or equal to $30^\circ$ F
	Determine Status of RCS Pressure Control
ATC	5. DETERMINE RCS Pressure Control acceptance criteria are met by <b>BOTH</b> of the following:
	CHECK that pressurizer pressure is 1900 to 2350 psia.
	<ul> <li>CHECK that pressurizer pressure is trending to 2225 to 2300 psia.</li> </ul>
	Determine Status of Core Heat Removal
	<ol> <li>DETERMINE that Core Heat Removal acceptance criteria are met by performing ALL of the following:</li> </ol>
ATC	a. CHECK that at least one RCP is operating and that loop delta T is less than 10 °F
	b. CHECK that Th subcooling is greater than or equal to 30°F.

Op-Test No.	ES16LI1	Scenario No.:1 Event No.: _7, 8
Event Descri	ption: RCP tri	ip, ATWS and total loss of condenser vacuum.
Time	Position	Applicant's Actions or Behavior
		Determine Status of RCS Heat Removal
		7. DETERMINE that RCS Heat Removal acceptance criteria are met by ALL of

a.	CHECK that at least one steam generator has <b>BOTH</b> of the following conditions met:

• Level is 10 to 80%.

BOP

- Main feedwater or TWO auxiliary feedwater pumps are operating to restore level 40 to 70%.
- b. CHECK that RCS Tc is being maintained between 530 °F to 535"F.
  c. CHECK that **BOTH** steam generators pressure are 880 to 920 psia.

Examiner Note: Once the BOP has established AFW flow to each SG, or at the lead examiner's direction, proceed to Event 9, Loss Of All Feedwater

Time	Position	Applicant's Actions or Behavior			
Event Descri	Event Description: Rupture of Auxiliary Feedwater Headers resulting in Loss Of All Feedwater flow				
Op-Test No.:	ES16LI1	Scenario No.:1 Event No.: _9			

Examiner Note	e: The AF	W rupture is simulated as a severe break in the <u>body</u> of FW-44, the cross- vo AFW headers and closing the valve will have no effect on the loss of the
		P should report the loss of all feed to the SGs, due to loss of Aux. Feedwater tion to an event specific EOP.
	BOP	<ul> <li>7. a. RNO <ul> <li>a.1 RESTORE level to 40 to 70% in at least one steam generator using ANY of the following:</li> <li>Motor- driven auxiliary feedwater pump.</li> <li>TDAFW Pump. Refer To Appendix 6, "TDAFW Pump Normal Startup."</li> </ul> </li> </ul>

closing FW-44 will have any effect on the leak.

<u>After</u> Aux. Feed flow is established, if the rupture is <u>not</u> recognized by the crew and an operator is <u>not</u> dispatched, report as the Turbine Building Watch the above conditions.

Op-rest No.: <u>ESTOLIT</u> Scenario No.: <u>I</u> Event No.: <u>9</u>	Op-Test No.: _	ES16LI1	Scenario No.:	1	Event No.: 9	)
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Time Position Applicant's Actions or Behavior
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ATC	Determine Status of Containment Isolation
	8. DETERMINE that Containment Isolation acceptance criteria are met by <b>ALL</b> of the following:
	a. CHECK that containment pressure is less than 1.0 psig.
	b. CHECK that <b>NONE</b> of the following primary plant radiation monitors have an unexplained alarm or indicate an unexplained rise in activity:
	Radiation Monitors Inside Containment
	RM-7890, Personnel Access Area RM-7891, Ctmt Refuel Floor Area RM-8240, High Range RM-8241, High Range RM-8123 A and B, Ctmt Atmosphere RM-8262 A and B, Ctmt Atmosphere
	c. CHECK that NONE of the following steam plant radiation monitors have an unexplained alarm or indicate an unexplained rise in activity:
	Steam Plant Radiation Monitors
	RM-5099, Steam Jet Air Ejector RM-4262, SG Blowdown RM-4299A and B, Main Steam Line 1 RM-4299C, Main Steam Line 2
	Determine Status of Containment Temperature and Pressure Control
	<ol><li>DETERMINE that Containment Temperature and Pressure Control acceptance criteria are met by <b>BOTH</b> of the following steps:</li></ol>
ATC	<ul> <li>CHECK that containment temperature is less than 120°F. (PPC or avg. of Points 5 and 6)</li> </ul>
	b. CHECK that containment pressure is less than 1.0psig.
	10. PERFORM the following:
	a. DIAGNOSE the event. Refer To Appendix 1, "Diagnostic Flowchart."
SRO	<ul> <li>INITIATE Appendix 4, Attachment 4A "Reactor Trip Subsequent Actions."</li> </ul>
	c. Go To the appropriate EOP

Op-Test No.:	ES16LI1	Scenario No.: 1	Event No.: 9
op 10001100			

Time Position	Applicant's Actions or Behavior
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	ATC/BOP	<ul> <li>{Step 10.b above}</li> <li>Perform Appendix 4, Attachment 4A "Reactor Trip Subsequent Actions".</li> <li>Examiner Note: EOP Appendix 4, Attachment 4A "Reactor Trip Subsequent Actions." are attached to guide.</li> </ul>
Examiner I diagnose t		t Supervisor refers to EOP 2541 Appendix 1, Diagnostic Flowchart to
	SRO	Enters EOP 2537, Loss Of All Feed.
within the Safety Fun	ORP or select ction. Asteri	lowing steps are from EOP 2537 Loss Of All Feedwater. Asterisked steps, ted FRPs being implemented, may be brought forward to restore or preserve a sked steps are "Continuously Applicable," and may be performed out of order complished once.
	0.50	*1. CONFIRM diagnosis of a Loss of All Feedwater by performing the following.
	SRO	Examiner Note: SRO checks EOP 2537-001 LOAF Safety Function Status Checks and confirms that all Safety Criteria are satisfied.
	SRO	*2. CLASSIFY the event. Refer To MP-26-EPI-FAP06, "Classification and PARs" IF classification requires RCS sampling, Refer To Appendix 46, "Sampling for EAL Determination" and DIRECT Chemistry as required.
	SRO	<ul> <li>*3. PERFORM ALL of the following:</li> <li>OPEN the placekeeper and ENTER the EOP entry time.</li> <li>ENSURE the master alarm silence switch is in "NORMAL".</li> </ul>
	ATC	<ul> <li>*4. PERFORM the following:</li> <li>a. STOP ALL RCPs.</li> <li>b. PLACE HIC- 4165, steam dump TAVG controller, in manual AND closed.</li> <li>c. PLACE the following pressurizer spray valve controllers in manual and CLOSE the valves:</li> <li>HIC- 100E</li> <li>HIC- 100F</li> </ul>

Op-Test No.:	ES16LI1	Scenario No.:	1	Event No.:	9
					-

Time Position Applicant's Actions or Behavior	
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BOP	<ul> <li>*5. PERFORM ALL of the following to conserve steam generator inventory:</li> <li>a. ENSURE MS- 220A, blowdown isolation valve is closed.</li> <li>b. ENSURE MS- 2206, blowdown isolation valve is closed.</li> <li>c. CLOSE BOTH steam generator sample isolation valves:</li> <li>MS- 191A</li> <li>MS- 1918</li> </ul>			
	NOTE OTC should be initiated prior to ONE steam generator wide range level reaching 70 inches, AND ONE steam generator wide range level reaching 165 inches if ANY of the following exist: 1. Main or auxiliary feedwater is <i>not</i> expected to be restored. 2. Less than TWO trains of HPSI, PORVs and ADVs are available. 3. Less than THREE charging pumps are available. *6. CHECK for adequate RCS heat removal via the steam generators by BOTH of the following: • BOTH steam generator wide range levels are greater than 70 inches			
BOP	<ul> <li>RCS Tc stable or controlled within 5°F or less</li> <li>6.1 Contingency Actions:</li> <li>IF steam generator level is not restoring AND ANY of the following conditions exists: <ul> <li>ONE steam generator wide range level less than or equal to 70 inches AND the REMAINING steam generator wide range level is less than or equal to 165 inches</li> <li>RCS TC rises uncontrollably by 5°F or more</li> </ul> </li> </ul>			
	<ul> <li>ESTABLISH heat removal via once- through- cooling by performing ALL of the following: <ol> <li>ENSURE ALL proportional heaters are tripped</li> <li>ENSURE ALL backup heaters in "PULL- TO- LOCK".</li> <li>IF main condenser is available, THEN OPEN ALL steam dump valves.</li> <li>OPEN BOTH ADVs.</li> <li>ENSURE SIAS actuated.</li> <li>ENSURE BOTH HPSI pumps have started.</li> <li>ENSURE that ALL HPSI loop injection valves are open.</li> <li>ENSURE that ALL available charging pumps are running.</li> </ol> </li> </ul>			
	<ol> <li>ENSURE that <b>BOTH</b> PORV block valves are open.</li> <li>WHEN at least <b>ONE</b> HPSI pump has started, THEN OPEN <b>BOTH</b> PORVs. (Key # 187)</li> <li>Go To EOP 2540, "Functional Recovery."</li> </ol>			

Op-Test No.:	ES16LI1	Scenario No.:1_ Event No.: _9			
Event Descri	Event Description: Rupture of Auxiliary Feedwater Headers resulting in Loss Of All Feedwater flow				
Time	Position	Applicant's Actions or Behavior			
		-Through-Cooling (OTC) shall be fully implemented (both ADVs open, both ains of HPSI in operation) prior to either SG level lowering to less than 32			
•	-	(CT-2/ OP 2260, "EOP User's Guide", Attachment 7, Operator Actions #2]			
Stea	erator Level v am Generator am Generator				
condensate	e pumps were	may decide to use the Once-Through-Cooling success path if all three e secured in error or he feels conditions warrant. In that instance, RNO 6.1 is e complete, the crew will transition to EOP 2540 and the scenario is complete.			
		RNO (CONTINGENCY ACTIONS)			
		6.1 IF steam generator level is not restoring AND ANY of the following conditions exists:			
	ATC	<ul> <li>ONE steam generator wide range level less than or equal to 70 inches AND the REMAINING steam generator wide range level is less than or equal to 165 inches</li> <li>RCS TC rises uncontrollably by 5°F or more ESTABLISH heat removal via once- through- cooling by performing ALL of the following:</li> </ul>			
		6.1 ENSURE ALL proportional heaters are tripped.			
		<ul> <li>a. ENSURE ALL backup heaters in "PULL- TO- LOCK".</li> <li>b. IF main condenser is available, THEN OPEN ALL steam dump valves.</li> <li>c. OPEN BOTH ADVs.</li> <li>d. ENSURE SIAS actuated.</li> </ul>			
		e. ENSURE BOTH HPSI pumps have started.			
		RNO (CONTINGENCY ACTIONS)         6.1 (continued)			
		f. ENSURE that ALL HPSI loop injection valves are open.			
	470	g. ENSURE that ALL available charging pumps are running.			
	ATC	<ul> <li>h. ENSURE that BOTH PORV block valves are open.</li> <li>i. WHEN at least ONE HPSI pump has started, THEN OPEN BOTH PORVs. (Key # 187)</li> </ul>			
		j. Go To EOP 2540, "Functional Recovery."			
		ew uses the Once-Through-Cooling success path, once it is implemented and P 2540, the remainder of EOP 2537 is N/A. Go to Page 36 for EOP 2540 steps.			

Op-Test No.:	ES16LI1	Scenario No.:	1 Event	No.: 9
• p . • • • • • •			<u> </u>	

Time F	Position	Applicant's Actions or Behavior
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	*7. WHEN feedwater source becomes available, THEN RESTORE feedwater to the affected steam generators as follows:
	a. CHECK steam generator level is less than 33%.
	a.1 FEED affected steam generators at any desired flow rate to restore and maintain level within 40% to 70%.
BOP	<ul> <li>b. FEED each affected steam generator by raising feedwater flow rate in increments of 50 gpm within BOTH of the following limits: <ul> <li>Flow limited to wi thin the capacity of the available feedwater source</li> <li>Maximum flow rate of less than, or equal to 650 gpm, (325 klbm/hr).</li> </ul> </li> <li>c. WHEN ANY of the following conditions are met: <ul> <li>Steam generator shows a rising trend</li> <li>Feedwater flow rate has been established within the capacity of the available feedwater source, with a maximum flow rate of less than, or equal to 650 gpm, (325 klbm/hr).</li> <li>Steam generator level is greater than 33%</li> </ul> </li> </ul>
	<u>THEN</u> FEED affected steam generators at any desired flow rate, within the capacity of the available feedwater source, to restore and maintain level within 40% to 70%.
BOP	<ul> <li>*8. <u>IF</u> a main feedwater line break is indicated, ISOLATE the affected portion of the main feedwater system by performing the following: <ul> <li>a. PLACE applicable main feed isolation air assisted check valve, to "CLOSE":</li> <li>FW- 5A</li> <li>FW- 5B</li> </ul> </li> <li>b. ENSURE applicable main feedwater block valve, is closed: <ul> <li>FW- 42A</li> <li>FW- 42B</li> </ul> </li> <li>c. CLOSE applicable main feedwater regulating bypass valve: <ul> <li>LIC- 5215</li> <li>LIC- 5216</li> </ul> </li> <li>d. <u>IF</u> leak is in common section of feedwater piping, SECURE BOTH main feedwater pumps.</li> <li>e. ESTABLISH feedwater to the unaffected header.</li> <li>f. SECURE steaming the steam generator with feedwater isolated.</li> </ul>
BOP	<ul> <li>*9. <u>IF</u> a auxiliary feedwater line break is indicated, ISOLATE the affected portion of the auxiliary feedwater system by performing the following:</li> <li>a. CLOSE FW - 44, auxiliary feedwater cross- connect valve.</li> <li>b. STOP ANY auxiliary feedwater pumps on the side with the affected header.</li> <li>c. ESTABLISH feedwater to the unaffected header.</li> <li>d. SECURE steaming the steam generator with feedwater isolated.</li> </ul>

Op-Test No.: <u>I</u>	ES16LI1	Scenario No.: _	1	Event No.: 9	-	
Event Description	n: Rupture o	of Auxiliary Fee	dwat	er Headers res	ulting in Loss Of All F	eedwater flow

Time	Position	Applicant's Actions or Behavior

## Examiner Note: The simulated AFW rupture cannot be isolated; therefore, all AFW pumps must be secured.

	SRO	<ul> <li>*10. IF offsite power has been lost, OR the condenser is <i>not</i> available, PERFORM the following: <ul> <li>a. CLOSE BOTH MSIVs:</li> <li>MS - 64A</li> <li>MS- 648</li> </ul> </li> <li>b. ENSURE BOTH MSIV bypass valves are closed: <ul> <li>MS - 65A</li> <li>MS - 658</li> <li>c. OPEN AR- 17, condenser vacuum breaker.</li> </ul> </li> </ul>
	SRO	<ul> <li>*11. RESTORE feedwater flow to at least ONE steam generator using the motor- driven AFW pumps as follows:</li> <li>Examiner Note: Step 11 is N/A due to the actions taken in Step 9 to isolate the AFW rupture.</li> </ul>
	SRO	*12. IF auxiliary feedwater flow is restored, Go To step 23.
	onto	Examiner Note: Step 12 is N/A
	SRO	*13. RESTORE feedwater flow to at least one steam generator using the TDAFW pump as follows
		Examiner Note: Step 13 is N/A
	SRO	*14. IF auxiliary feedwater flow is restored, Go To step 23.
	ono	Examiner Note: Step 14 is N/A
	SRO	*15. START ONE SG feedwater pump by performing the following for the pump to be started:
		Examiner Note: Step 15 is N/A due to the loss of condenser vacuum.
	SRO	*16. IF SG feedwater flow is restored, Go To step 23.
	300	Examiner Note: Step 16 is N/A
Examiner N understood		O may quickly proceed to Step 17 once the nature of the feedwater loss is
	BOP	*17. ENSURE at least ONE condensate pump is running.

Op-Test No.:	ES16LI1	Scenario No.: 1	Event No.: 9

Time	Position	Applicant's Actions or Behavior

	BOP	<ul> <li>*18. ESTABLISH a flowpath from the hotwell to at least ONE steam generator as follows: <ul> <li>a. ENSURE BOTH SGFP discharge valves are open:</li> <li>FW- 38A</li> <li>FW- 38B</li> </ul> </li> <li>b. CLOSE BOTH SGFP "MIN FLOW RECIRC": <ul> <li>FIC- 5237</li> <li>FIC- 5240</li> </ul> </li> <li>c. OPEN CNM-2, CPF bypass valve.</li> <li>d. OPEN BOTH main feed reg bypass valves: <ul> <li>LIC-5215</li> <li>LIC-5216</li> </ul> </li> <li>e. ENSURE BOTH main feed isolation air assisted check valves are open: <ul> <li>FW-5A</li> <li>FW-5B</li> </ul> </li> </ul>		
	BOP	*19. <u>IF SIAS is <i>not</i> present AND SIAS Block is permitted, THEN BLOCK the automatic initiation as the cooldown and depressurization proceeds.</u>		
Examiner N	lote: The cre	ew should block SIAS as the event does not warrant its actuation.		
	BOP	*20. <u>IF MSI is <i>not</i> present AND MSI Block is permitted</u> , THEN BLOCK the automatic initiation as the cool down and depressurization proceeds.		
Examiner N	lote: The cre	ew should block MSI as its actuation will delay feed flow restoration.		
	BOP	*21. <u>IF</u> a flowpath from the hotwell to a steam generator is established, THEN DEPRESSURIZE at least ONE steam generator at the maximum controllable rate, until adequate feed flow is obtained from the condensate pump to restore steam generator level.		
	ATC	*22. COMMENCE emergency boration. Refer To Appendix 3, "Emergency Boration."		
	BOP	*23. <u>IF</u> feed flow is restored, RESTORE and MAINTAIN 40 to 70% level in at least one steam generator.		
Scenario Termination: When crew has restored feedwater flow to one SG, or at the lead examiner's direction, the scenario is complete.				

Op-Test No.:	ES16LI1	Scenario No.:1_ Event No.: _9
Event Descri	ption: Rupture	e of Auxiliary Feedwater Headers resulting in Loss Of All Feedwater flow
Time	Position	Applicant's Actions or Behavior
		is initiated in EOP 2537, Loss Of All Feedwater, once initiated, the US will Functional Recovery.
		from EOP 2540, Functional Recovery.
Indications	):	
		Leg Temperature
	SRO	1. CLASSIFY the event. Refer To MP-26-EPI-FAP06, "Classification and PARs"
	0110	<ul> <li><u>IF</u> classification requires RCS sampling, Refer To Appendix 46, "Sampling for EAL Determination" and DIRECT Chemistry as required.</li> </ul>
		ied as an Alert/C-1; Equipment Failure, EA1, ATWS w/ Successful Manual Trip.
Also, Alert	/C-1; RCS Bai	rrier, RCB2 Loss (RCS Subcooling < 30 °F)
		2. PERFORM ALL of the following:
	SRO	<ul> <li>OPEN the Safety Function Tracking Page and ENTER the EOP entry time.</li> <li>ENSURE the master alarm silence switch is in "NORMAL".</li> </ul>
		3. IF pressurizer pressure is less than 1714 psia AND SIAS has initiated,
	ATC	<ul> <li>PERFORM the following: <ul> <li>a. ENSURE ONE RCP in each loop is stopped.</li> <li>b. PLACE associated pressurizer spray valve controller RC-100E or RC-100F in manual and CLOSE the valve.</li> <li>c. <u>IF</u> pressurizer pressure lowers to less than the minimum RCP NPSH limit, PERFORM the following: <ol> <li>STOP ALL RCPs.</li> <li>PLACE TIC-4165, steam dump TAVG controller, in manual and closed.</li> <li>PLACE pressurizer spray valve controllers RC-100E and RC-100F in manual and CLOSE the valves.</li> </ol> </li> </ul></li></ul>

Op-Test No.:	ES16L11	Scenario No.: 1	Event No.: 9
op 1001100.			

Event Description: Rupture of Auxiliary Feedwater Headers resulting in Loss Of All Feedwater flow

Time	Position	Applicant's Actions or Behavior

[]	
BOP CUE: When directed	<ul> <li>4. SAMPLE steam generators that are available for RCS heat removal as follows: <ul> <li>a. CHECK "B" train RBCCW in service.</li> <li>b. ENSURE 2-RB-210 "Degasifier Effluent Cooler Return Outlet" is open.</li> <li>c. OPEN appropriate steam generator sample valves: <ul> <li>MS-191A</li> <li>MS-191B</li> </ul> </li> <li>d. DIRECT Chemistry to perform ALL of the following: <ul> <li>Sample ANY steam generator that is available for RCS Heat Removal</li> <li>Frisk the samples</li> <li>Report frisk results</li> <li>Analyze samples for boron and activity</li> </ul> </li> <li>e. <u>WHEN</u> Chemistry reports that samples have been taken, PERFORM the following: <ul> <li>CLOSE the steam generator sample valves</li> <li>IF SIAS has actuated, <u>AND</u> no other sampling is in progress, CLOSE 2-RB-210, "Degasifier Effluent Cooler Return Outlet"</li> </ul> </li> </ul> </li> <li>to sample Steam Generators, respond 20 minutes later that samples have been</li> </ul>
BOP	isk results show all background levels.         5. PLACE the hydrogen analyzers in service. Refer To Appendix 19, "Hydrogen Analyzer Operation."
SRO	NOTE         If the Safety Function Status Checklist is <i>not</i> satisfied for the selected success path, the US may commence the operator actions for safety functions which are <i>not</i> met based on Safety Function hierarchy. The remaining Safety Functions should be prioritized as time permits.         6. IDENTIEX success paths to be used to satisfy each safety function using BOTH
SRO	<ul> <li>6. IDENTIFY success paths to be used to satisfy each safety function using <b>BOTH</b> of the following: <ul> <li>Resource Assessment Trees</li> <li>Safety Function Tracking Page</li> </ul> </li> </ul>

Op-Test No.:	ES16LI1	Scenario No.: 1	Event No.: 9

Event Description: Rupture of Auxiliary Feedwater Headers resulting in Loss Of All Feedwater flow

Time	Position	Applicant's Actions or Behavior

		7. PRIORITIZE s	afety functio	ns to be addresse	ed first based o	on ALL of	the		
		following:							
		a. Safety	functions wh	nich do <i>not</i> meet t	he Safety Fund	ction Statu	is Checklist		
		for the se	lected succe	ess path.					
	SRO	b. Safety	functions for	which the equipn	nent to suppor	t the succ	ess path is		
	0110	<i>not</i> opera	•						
		-		which success p					
		•	d. Safety functions for which success path two has been selected.						
		-		which success p					
		Board Operators	•	Resource Assess	ment Trees ar	nd query p	lant status		
to determine	e the correct F	unctional Procedu							
		4.0 SAFETY FU	NCTION ST	ATUS CHECKLIS	T				
		SAFETY FUNCT	ION TRACK		EOP ENTRY	( TIME			
	SRO	Safety Function	Suc	cess Path	Procedure	SFSC Met	Priority		
			RC-1	CEA Insertion		Y			
		Reactivity	RC-2	Boration CVCS	EOP 2540A		4		
		Control	RC-3	Boration SI			-		
		Maintenance of Vital DC Power	MVA-DC-1	Battery Chargers/ Station Batteries	EOP 2540B	Y	5		
			MVA-AC-1	RSST		Y			
		Maintenance of	MVA-AC-2	EDG	EOP 2540B		6		
		Vital AC Power	MVA-AC-3	BUS 34A/34B					
		RCS Inventory	IC-1	CVCS	EOP		2		
		Control	IC-2	Safety Injection	2540C1	Y	2		
			PC-1	Subcooled					
		RCS Pressure	PC-2	Saturated	EOP	Y	3		
		Control	PC-3	PORVs	2540C2				
			HR-1	SI no operating					
	SRO	SRO Removal	HR-2	SI operating	EOP 2540D		1		
			HR-3	O-T-C		Y			
		Containment Isolation	CI-1	Automatic / Manual	EOP 2540E	Y	7		

Op-Test No.:	ES16LI1	Scenario No.:	1	Event No.: 9	
op 100011011_	<u></u>	00011411011011			_

Event Description: Rupture of Auxiliary Feedwater Headers resulting in Loss Of All Feedwater flow

Time Position Applicant's Actions or Behavior	Time
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SRO	Containment Temperature and Pressure Control	CTPC-1	CARs (Normal)	EOP 2540F	Y	<b>8</b> May be #4 if CTMT >4.42psi
360		CTPC-2	CARs (Emerg)		May be <b>Y</b> if >4.42psi	
SRO	8. DIRECT the S satisfied for chose		•	ion Status Che	ecklist Crite	eria are
	9. PERFORM operator actions for chosen success paths based on priority assigned.					
	should transition hich time the sce		-	Recovery of H	leat Remo	oval,

Time	Position	Applicant's Actions or Behavior	
Event Descri	ption: EOP 2	541, Followup Actions, Appendix 4A, Reactor Trip Subsequent Actions	
Op-Test No.:	ES16LI1	Scenario No.:1_ Event No.: _7	

Subsequent Actions.	<ol> <li>IF charging pumps suction is aligned to the VCT, <u>THEN</u> CHECK VCT level is between 72% to 86%:         <ul> <li>a. IF VCT level is less than 72%, THEN ALIGN charging pump suction to RWST as follows:</li> <li>OPEN CH- 192, RWST isolation.</li> <li>ENSURE CH- 504, RWST to charging suction is open.</li> <li>CLOSE CH- 501, VCT outlet isolation.</li> <li>ENSURE CH- 196, VCT makeup bypass is closed.</li> <li>b. IF VCT level is greater than 88%, THEN PLACE CH- 500, letdown divert handswitch, to the "RWS" position, and divert as required to maintain VCT level 72% to 86%.</li> </ul> </li> </ol>
ATC	<ul> <li>2. TCOA: <u>IF</u> SIAS actuated, <u>THEN</u> ENSURE <b>ONE</b> complete facility of CRAC operating, in RECIRC mode, as follows: (C25A/B)</li> <li>Facility 1 <ul> <li>HV- 203A, Fan F- 21A exhaust damper open</li> <li>Fan F- 21A, supply fan running</li> <li>HV- 206A, Fan F- 31A exhaust damper open</li> <li>Fan F- 31A, exhaust fan running</li> <li>HV- 212A, Fan F- 32A exhaust damper, open</li> <li>Fan F- 32A, filter fan, running</li> <li>HV- 202, minimum fresh air damper, closed</li> <li>HV- 208, exhaust air damper, closed</li> </ul> </li> </ul>
	<ul> <li>Facility 2</li> <li>HV- 203B, Fan F- 21B exhaust damper open</li> <li>Fan F- 21B, supply fan running</li> <li>HV- 206B, Fan F- 31B exhaust damper open</li> <li>Fan F- 31B, exhaust fan running</li> <li>HV- 212B, Fan F- 32B exhaust damper, open</li> <li>Fan F- 32B, filter fan, running</li> <li>HV- 495, fresh air damper, closed</li> <li>HV- 496, exhaust air damper, closed</li> <li>HV- 497, cable vault exhaust damper, closed</li> </ul>

Op-Test No.:	ES16LI1	Scenario No.:	1	Event No.:	7
					<u> </u>

Event Description: EOP 2541, Followup Actions, Appendix 4A, Reactor Trip Subsequent Actions

Time	Position	Applicant's Actions or Behavior
TIME	FUSICION	Applicant's Actions of Denavior

	3. TCOA: <u>IF</u> SIAS not actuated, <u>THEN</u> CHECK ONE facility of CRAC operating, in NORMAL mode, as follows: (C25A/B)
ATC	<ul> <li>Facility 1</li> <li>HV- 203A, Fan F- 21A exhaust damper is open</li> <li>Fan F- 21A, supply fan running</li> <li>HV- 206A, Fan F- 31A exhaust damper open</li> <li>Fan F- 31A, exhaust fan running</li> </ul>
	<ul> <li>Facility 2</li> <li>HV- 203B, Fan F- 21B exhaust damper open</li> <li>Fan F- 21B, supply fan running</li> <li>HV- 206B, Fan F- 31B exhaust damper open</li> <li>Fan F- 31B, exhaust fan running</li> </ul>
ATC	<ul> <li>4. <u>IF</u> charging pumps suction aligned to the RWST <u>AND</u> boration not required, <u>THEN</u> RESTORE charging pump suction to VCT as follows: <ul> <li>a. CHECK BOTH of the following:</li> <li>1) VCT level between 72% and 86%</li> <li>2) VCT pressure greater than 15 psig</li> <li>b. CHECK letdown is in service.</li> <li>c. OPEN CH- 501, VCT outlet isolation.</li> <li>d. CLOSE CH- 192, RWST isolation.</li> </ul> </li> </ul>
BOP	5. CHECK instrument air pressure greater than 90 psig and stable.
BOP	<ol> <li>IF AFAS has actuated, WHEN BOTH steam generators are restored to greater than 33%, THEN PERFORM the following:         <ul> <li>a. PLACE the following switches in "M" (Manual) and ADJUST to obtain desired flow (C- 05):                 <ul></ul></li></ul></li></ol>
	once the Arw rupture is discovered and the SRO directs Arw be secured.

Op-Test No.:	ES16LI1	Scenario No.:	1	Event No.: 7

Event Description: EOP 2541, Followup Actions, Appendix 4A, Reactor Trip Subsequent Actions

Time	Position

Applicant's Actions or Behavior

BOP	<ul> <li>7. CHECK Main Condenser is available, as indicated by ALL of the following: <ul> <li>At least ONE MSIV open</li> <li>Condenser vacuum better than 15 inches HG - ABS (0 to 15 inches)</li> <li>At least ONE condensate pump operating</li> <li>At least ONE Circ Water pump operating</li> </ul> </li> <li>RNO <ul> <li>7.1 IF Main Condenser is not available, PERFORM the following:</li> <li>CLOSE BOTH MSIVs.</li> <li>ENSURE BOTH MSIV bypass valves are closed.</li> <li>OPEN AR-17, condenser vacuum breaker.</li> </ul> </li> <li>Examiner Note: Not available due to the loss of condenser vacuum.</li> </ul>
ВОР	8. OPEN HD-106, subcooling valve.
ВОР	9. ENSURE BOTH heater drain pumps stopped.
BOP	<ul> <li>10. IE MFW is supplying feed to the steam generators, <u>THEN</u> PERFORM the following: <ul> <li>a. ENSURE that only ONE main feedwater pump is operating.</li> <li>b. ENSURE that BOTH main feed block valves are closed: <ol> <li>FW- 42A</li> <li>FW- 42B</li> </ol> </li> <li>c. ADJUST the operating main feedwater pump pressure to 50 to 150 psi greater than SG pressure.</li> <li>d. ENSURE BOTH main feed reg bypass valves are throttled to control SG level: <ol> <li>LIC- 5215</li> <li>LIC- 5216</li> </ol> </li> <li>e. IE Main Feedwater Pump A is secured, <u>THEN</u> CLOSE the following: <ol> <li>FW- 38A, main feedwater pump discharge valve</li> <li>FIC- 5237, main feedwater pump mini flow recirc valve</li> </ol> </li> <li>f. IE Main Feedwater Pump B is secured, <u>THEN</u> CLOSE the following: <ol> <li>FW- 38B, main feedwater pump mini flow recirc valve</li> <li>FIC- 5240, main feedwater pump discharge valve</li> <li>FIC- 5240, main feedwater pump mini flow recirc valve</li> </ol> </li> </ul></li></ul>
BOP	<ul> <li>11. <u>IF</u> BOTH MFW pumps are secured, <u>THEN</u> PERFORM the following:</li> <li>a. CLOSE BOTH main feedwater pump mini flow recirc valves.</li> <li>FIC- 5237</li> <li>FIC- 5240</li> </ul>

Time	Position	Applicant's Actions or Behavior				
Event Descri	Event Description: EOP 2541, Followup Actions, Appendix 4A, Reactor Trip Subsequent Actions					
Op-Test No.:	Op-Test No.: <u>ES16LI1</u> Scenario No.: <u>1</u> Event No.: <u>7</u>					

BOP 12. <u>IF</u> 25A OR 25B is energized, <u>THEN</u> ALIGN condensate pumps as follows: a. ENSURE <b>ONE</b> pump is running. b. ENSURE <b>ONE</b> pump is in "PULL TO LOCK." c. ENSURE <b>ONE</b> pump is in "AUTO."					
Examiner Note: it is important that at least one condensate pump remain running. If all three are mistakenly secured, the only success path available will be Once-Through-Cooling.					
Examiner Note: End of Attachment 4- A					

## SIMULATOR SCENARIO #2

Appendix D

Scenario Outline

Form ES-D-1

Facility: Mills	tone Unit 2		Scenario No.: 2	2	Op-Test No.: ES16LI2	
Examiners:		Operato	ors: _ -	SRO ATC BOP		
Initial Conditi service.	ons: 90% Po	wer IC-140,	No Equipment OOS	6, Ch-Y	' PZR Level in service, Fac. 2 CRAC in	
Turnover: 90 OP2204 to 10		non building	in, no equipment C	)OS. 24	4E is aligned to 24C. Raise Power IAW	
Critical Tasks	S:					
	<ol> <li>SGTR-6; Manually establish the minimum design Safety Injection System flow.</li> <li>SGTR-5: Isolate the affected SG.</li> </ol>					
Event No.	Malf. No.	Event Type*		I	Event Description	
1	N/A	R, N (ATC/S) (BOP/S)	Raise Reactor Pov	wer to 1	100%.	
2	RP19C	I, TS (ATC/S)	'C' RPS Lower NI	Fails lo	ow.	
3	CW02D	C (BOP/S)	"D" Traveling Scre	en D/P	high, requires securing "D" Circ. Pump.	
4	RM01P CH08D	I, TS (ATC/S)	CRAC Radiation N Fan, F-32B, Trips		, RM-9799B, fails high and "B" CRAC Filter	
5	SG01A	C, TS (BOP/S)	SGTL in #1 SG (T	S).		
6	N/A	R (All)	Down power due t	o SGTI	L.	
7	SG02B	M (All)	SG Tube Rupture.	Manu	al plant trip.	
8	ES03J / SI05A	C (ATC/S)	"C" HPSI pump fai (100%)	ils to sta	art on SIAS. "A" HPSI pump is degraded	
* (N)orma	al, (R)eactivity,	(I)nstrument, (0	C)omponent, (M)ajor			

Tar	get Quantitative Attributes (Per Scenario; See Section D.5.d)	Actual
1.	Total malfunctions (5–8)	6
2.	Malfunctions after EOP entry (1–2)	1
3.	Abnormal events (2–4)	4
4.	Major transients (1–2)	1
5.	EOPs entered/requiring substantive actions (1-2)	1
6.	EOP contingencies requiring substantive actions (0–2)	0
7.	Critical tasks (2–3)	3

## NRC 2016, Scenario 2 Summary:

The crew will take the shift with the unit at 90% Xenon slowly building in, with no equipment out of service (IC-30). The crew will begin the shift by raising power to 100%.

**Event 1**: The crew takes the shift and begins the power ascension to 100% power. Xenon concentration will be slowly rising requiring the Crew to initiate a dilution or reduce Turbine load to maintain RCS temperature. The Crew will be referring to OP 2204, OP 2302A, OP 2304C and OP 2208 to dilute, with draw CEAs and raise Turbine load.

**Event 2**: After the dilutions to raise power and at the discretion of the Examiners "C" RPS Nuclear Instrument fails requiring ARP actions to bypass the affected Reactor Trip Modules on Channel "C" and the U.S. will enter the applicable Tech. Spec.

**Event 3**: At the discretion of the Examiners the malfunction for the "D" Traveling Screen DP will be initiated causing the Traveling Screen for "D" Water box to ramp in above high setpoint requiring the securing of "D" Circ. Pump. The Crew will enter AOP 2517 for Circulating Water Malfunction and take actions to cross-tie "C" and "D" Water boxes by closing the "D" Water box inlet valve and place the tripped Circ Pump handswitch in PTL and lastly the Crew will verify Condenser vacuum <4.5" Hg.

**Event 4**: At the discretion of the Examiners, the Control Room Air Conditioning (CRAC) Radiation Monitor will fail high, causing the ventilation system to shift into recirculation mode. Five seconds after starting, the "B" Filter Fan, F32B, will trip due to a broken belt. Per ARP 2590A-159, the crew should verify proper operation of the CRAC system and note the loss of the filter fan. This will require the crew to secure Facility 2 CRAC and ensure Facility 1 CRAC is operating as required per ARP 2590A-159. The US will enter TSAS 3.7.6.1a for an inoperable train of CRAC, TSAS 3.3.1 for the failed rad monitor, and call Maintenance/Work Planning for the needed equipment repairs.

**Event 5**: At the discretion of the Examiners the malfunction for a S/G tube leak is inserted. The Crew will be alerted to a SGTL by C06/07 Alarm for N-16 HIGH and carry out the actions for the ARP and will enter AOP 2569 SGTL. The Crew will verify Reactor Trip Criteria is not exceeded but the Tech. Spec. of 75 gpd will be exceeded requiring a Reactor down power. The Crew will transition to AOP 2575 Rapid Downpower.

**Event 6**: The crew will enter AOP 2575 Rapid Downpower. The first action for the rapid down power will require the Crew to force PZR sprays then insert Group 7 Rods 10 steps while reducing Turbine load to maintain RCS  $T_{COLD}$ . The Crew will then align for boration from the RWST requiring the ATC operator to start an additional Charging pump if not already started and the BOP to setup the Turbine HMI to lower load to maintain RCS  $T_{COLD}$ .

**Event 7**: During the Rapid Downpower at the discretion of the Examiner an S/G tube rupture will be inserted requiring the Crew to verify actual rupture using plant parameters and the Main Steam Line Hi alarm and direct a manually trip the Reactor.

**Event 8**: After the completion of EOP 2525 SPTA the crew will enter EOP 2534 SGTR and upon SIAS the "C" HPSI pump will fail to start and the "A" HPSI will be fully degraded requiring the ATC to manually start the "C" HPSI pump and or start the "B" HPSI pump to meet SI flow criteria.

INPUT SUMMARY									
Either INP	UT or VERIFY the following functions:								
ID Num	Description	Delay	Ramp	Event	Sev or	Final	Rel		
		Time	Time	Time	Value	Value	Order		
MALFUNCTIONS									
	"C" Channel Power Range RPS fail			E-2	0%	0%	2		
	"D" Traveling Screen D/P		180 sec	E-3	65%	65%	3		
	RM-9799A CNTRL RM Radmon			E-4	100%	100%	4		
	"B" CRAC Filter Fan, F32B, trip	5 sec		E-4	N/A	N/A	4		
SG01A	#1 S/G tube leak		120 sec	E-5	50%	50%	5		
	#1 S/G tube rupture		60 sec	E-7	15%	15%	7		
ES03J	"C" HPSI start fail on SIAS			E-30	N/A	N/A	8		
SI05A	"A" HPSI pump degradation		60 sec	E-30	100%	100%	8		
		E FUNC	TIONS						
	"B" Screen Wash Pump Start			E-10		START	3		
CWR07A	"A" Screens to Fast			E-10		FAST	3		
	"B" Screens to Fast			E-10		FAST	3		
CWR07C	"C" Screens to Fast			E-10		FAST	3		
	"D" Screens to Fast			E-10		FAST	3		
CWR07D	"D" Screens to Stop			E-11		STOP	3		
	OVERRIDES								

Op-Test No.: <u>ES16LI2</u> Scenario No.: <u>2</u> Event No.: <u>1</u>				
Event Description: Raise Power to 100%				
Time Position Applicant's Actions or Behavior				

 Examiner Note: The crew has been instructed to brief the up power prior to taking the watch. The The following steps are from OP 2204 Load Changes. OP 2204 Load Changes procedure is marked up with "N/A" and Unit Supervisor signatures for applicable steps.

 Up power in accordance with OP 2204 and Reactivity Plan.

 Method: dilution and CEAs

 Rate: 15%/hour

 Crew will dilute to the charging pump suction and raise power to ~100%.

 Turbine load will be increased to maintain RCS Tavg on program.

 Examiner Note: When reactor power is 5% higher than initial power or at the lead examiner's direction, proceed to Event #2, Trip of "C" Channel of NI power instrument.

		Scenario No.: 2 Event No.: 2
Event Descri	ption: "C" RI	PS Lower NI Failure (low)
Time	Position	Applicant's Actions or Behavior
Simulator	Operator: W	hen directed, initiate Event #2, "C" RPS Lower NI Fails Low.
<ul> <li>NIS</li> <li>RX</li> <li>TM-</li> <li>LCL</li> </ul>	POWER ΔT ( LP TRIP CH . PWR DENS	DEVIATION HI (C-04, BA-12). CH DEVIATION (C-04, AA-8). C (C-04, DA-3) E TRIP CH C (C-04, DA-5) NIS (C-04, CA-18)
		llowing steps are from ARP 2590C-089.
Examiner I	Note: Crew r	nay or may not secure dilution in progress from Event #1.           AUTOMATIC FUNCTIONS
		<ol> <li>None         CORRECTIVE ACTIONS     </li> <li>1. To determine the cause of alarm, OBSERVE the following:         <ul> <li>ASI (C-04, PPC, RPS)</li> <li>Linear power channel indications (C-04, PPC, RPS)</li> </ul> </li> </ol>
		NOTE
	ATC	<ul> <li>This alarm may be indicative of one or more of the following:</li> <li>One RPS linear power channel (4 total), deviating from the grand average signal generated by comparator average hi or hi-hi deviation setpoint</li> <li>Axial offset condition</li> <li>Failure of one <i>power range monitor</i> channel</li> </ul>
		<ul> <li>2. <u>IF</u> power range monitor has failed, PERFORM the following:</li> <li>2.1 OBTAIN necessary keys and PERFORM applicable actions to bypass the following power outputs for applicable RPS channel causing alarm:</li> <li>TM/LP Trip</li> <li>High Power Trip</li> <li>Local Power Density Trip</li> <li>Turbine Trip (RPS)</li> <li>2.2 LOG entry into applicable ACTION Statement(s) of T/S, 3.3.1.1.</li> <li>2.3 As necessary, Refer To OP 2380, "RPS and NI Safety Channel Operation," and PERFORM applicable actions to remove affected channels input to comparator averager.</li> </ul>

Op-Test No.	: ES16Ll2	Scenario No.: 2 Event No.: 2
·		PS Lower NI Failure (low)
Time	Position	Applicant's Actions or Behavior
		CORRECTIVE ACTIONS (cont.)
	ATC	<ol> <li><u>IF</u> power range <i>control</i> channel has failed, PLACE applicable power ration calculator input switch to "OUT" (rc-05e):         <ul> <li><u>IF</u> channel "X," "CH 9"</li> <li><u>IF</u> channel "Y," "CH 10"</li> </ul> </li> <li><u>IF</u> any axial offset condition exists for an unknown reason, NOTIFY Reactor Engineering</li> </ol>
		<ul> <li>Engineering.</li> <li>5. Refer To T/S LCO 3.2.4 and DETERMINE applicable and additional actions.</li> <li>6. <u>WHEN</u> alarm conditions clears, to reset "HI DEV" and "HI-HI DEV" alarms, <i>Power range monitor</i>, TOGGLE "LED RESET" momentarily</li> </ul>
		<ol> <li>IF ALL of the following alarms are coincident with this annunciator, REQUEST I&amp;C Department to refer to 25203-39069 sh. 23C and check ruses for interposing relay circuit in RC22:</li> </ol>
Examiner	Note: The fo	ollowing steps are from OP 2380, "RPS and NI Safety Channel Operation".
		4.3.2 IF, at any time, it is necessary to remove any power range safety channel
		input to comparator averager (used in computing average flux power level). PLACE applicable switch to "OFF" (rear of channel "A" RPS):
		IF desired to remove channel "A," "CHANNEL 5" switch
		• IF desired to remove channel "B," "CHANNEL 6" switch
		<ul> <li><u>IF</u> desired to remove "C," "CHANNEL 7" switch</li> <li>IF desired to remove "D," "CHANNEL 8" switch</li> </ul>
		Examiner Note: Crew should place the "CHANNEL 7" switch in "OFF".
Examiner	Note: Step 3	- 7 are N/A. SRO Should Review T/S for applicability and required action.
		Refer to Tech. Spec. 3.3.1.1
	000	<b>LCO 3.3.1.1 (RPS):</b> As a minimum, the reactor protective instrumentation channels and bypasses of Table 3.3-1 shall be OPERABLE.
	SRO	<b><u>APPLICABILITY</u></b> : As shown in Table 3.3-1.
		ACTION: As shown in Table 3.3-1.

Op-Test No.:ES16LI2	Scenario No.: _	2	Event No.: 2	
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## Event Description: "C" RPS Lower NI Failure (low)

Time	Position	Applicant's Actions or Behavior

		Per TS Table 3.3-1: FUNCTIONAL UNIT #2 Power Level - High; Total Number of Channels = 4, Minimum Channels Operable = 3, App. Modes = 1, 2, 3(d) Action = 2
		<u>Action 2</u> - With the number of OPERABLE channels one less than the Total Number of Channels, operation may continue provided the following conditions are satisfied:
	SRO	a. The inoperable channel is placed in either the bypassed or tripped condition within 1 hour. The inoperable channel shall either be restored to OPERABLE status, or placed in the tripped condition, within 48 hours.
		b. Within 1 hour, all functional units receiving an input from the inoperable channel are also declared inoperable, and the appropriate actions are taken for the affected functional units.
		c. The Minimum Channels OPERABLE requirement is met; however, one additional channel may be removed from service for up to 48 hours, provided one of the inoperable channels is placed in the tripped condition.
		hould note TSAS 3.3.1, Action 2 applies and is presently being met by the actions bass the affected channels.
		Review LCO 3.2.4 for applicability:
		<b>LCO 3.2.4 (T<sub>Q</sub>):</b> The AZIMUTHAL POWER TILT (T <sub>q</sub> ) shall be $\leq 0.02$ .
	SRO	<u><b>APPLICABILITY</b></u> : MODE 1 with THERMAL POWER > 50% of RATED THERMAL POWER <sup>(1)*</sup> .
		$\label{eq:action} \begin{array}{l} \underline{\textbf{ACTION}}: \\ \textbf{a.} & \text{With the indicated } T_q > 0.02 \text{ but } \leq 0.10 \text{, either restore } T_q \text{ to } \leq 0.02 \text{ within } 2 \text{ hours or verify} \\ & \text{the TOTAL UNRODDED INTRGRATED RADIAL PEAKING FACTOR } (F^Tr) \text{ is within} \\ & \text{the limit of Specification } 3.2.3 \text{ within } 2 \text{ hours; and }. \end{array}$
		hould note TSAS 3.2.4 does not apply because power tilt indication is due to an ot a
<b>Technical</b>	Specification	n the actions of ARP 2590C-089 have been addressed and the applicable is have been evaluated, or at lead examiner's direction, proceed to Event 3 eling Screen $\Delta P$ High.

Op-Test No.:	ES16LI2	Scenario No.: Event No.: _3
Event Descri	ption: " <b>D" Tra</b>	aveling Screen ΔP Fails High
Time	Position	Applicant's Actions or Behavior
Simulator	Operator: Wh	en directed, initiate Event 3, Failure of the "D" Traveling Screen $\Delta P$ High.
• TRA		REEN ΔΡ HI (C-06/7, D-10) rential Pressure indication on C-06/7 rising
Examiner M (C-06/7, D-		lowing steps are from ARP 2590E-056, "TRAVELING SCREEN $\Delta P$ HI"
	BOP	AUTOMATIC FUNCTIONS 1. None CORRECTIVE ACTIONS
		1. Go To AOP 2517, "Circulating Water Malfunctions."
	SRO	Enter AOP 2517, "Circulating Water Malfunctions."
Examiner N	Note: The foll	lowing steps are from AOP 2517, "Circulating Water Malfunctions."
		NOTE
	BOP	When power is less than 15% <u>AND</u> linear power bistable light clears ( <i>not</i> lit), on at least 3 RPS channels, the turbine trip is inhibited and turbine trip will <i>not</i> result in an automatic reactor trip.
		<ul> <li>3.1 <b>IF ANY</b> of the following conditions exist:</li> <li>"A" <u>AND</u> "B" circulating water pumps <i>not</i> operating</li> <li>"C" <u>AND</u> "D" circulating water pumps <i>not</i> operating</li> </ul>
		CHECK status of turbine trip bypass and PERFORM the following:
Examiner No	ote: Step 3.1 is I	N/A
	BOP	<ul> <li>3.2 IF ONE circulating water pump has tripped, PERFORM the following:</li> <li>a. ENSURE BOTH of the following exist:</li> <li>"A" OR "B" circulating water pump operating</li> <li>"C" OR "D" circulating water pump operating</li> <li>b. Go To Section 5.0, "Trip of One Circulating Water Pump."</li> </ul>

Time	Position	Applicant's Actions or Behavior		
		1		
		<ul> <li>3.3 IF "CIRC WATER PP LUBE WATER PRES LO" (C-06/7, A-10) annunciator in alarm, Go TO Section 6.0, "Circulating Water Pump Lube Water Pressure Low."</li> <li>3.4 IF "HI COND D/T" (C-06/7, DA-37) annunciator in alarm, Go TO Section 7.0, "High Condenser Differential Temperature."</li> <li>3.5 IF "HI COND DIS TEMP" (C-06/7, DB-37) annunciator in alarm, Go TO Section 8.0, "High Condenser Discharge Temperature."</li> <li>3.6 IF "TRAVELING SCREEN ΔP HI" (C-06/7, D-10) annunciator in alarm, Go TO Section 9.0, "Traveling Screen Differential Pressure High."</li> </ul>		
xaminer	Note: SRO sł	nould transition to Section 9.0 of AOP 2517 and the following is from Sec. 9.0.		
	BOP	<ul> <li>9.1 <u>IF</u> "TRAVELING SCREEN ΔP HI" (C-06/7, D-10) annunciator in alarm, PERFORM th following:</li> <li>a. PLACE BOTH screen wash pump switches "START:"</li> <li>"A" SCREENWASH PP, P8A, HS 6493"</li> <li>"B SCREENWASH PP, P8B, HS 6498"</li> <li>b. PLACE ALL available traveling screens in "RUN FAST."</li> </ul>		
II screen:	s in FAST. [E-	hen directed, trigger Event-10 to START the "B" Screen Wash Pump and plac 10; CWR06B (START), CWR07A, B, C & D (FAST)] e the "D" Traveling Screen, trigger Event-11 [E-11; CWR07D (STOP)]		
		CAUTION		
		Circulating water pumps may be unstable when operating near 50% speed; therefore circulating water pump operation below 60% speed is limited to pump starting and stopping only.		
	ВОР	<ul> <li>c. LOWER circulating water pump speed while monitoring condenser backpressure to lower traveling screen differential pressure.</li> <li>d. CHECK ALL screens rotating.</li> </ul>		
		d.1 <u>IF</u> ANY traveling screen motor is operating <u>AND</u> associated traveling screen is <i>not</i> rotating, PERFORM the following:		
		1) PLACE affected traveling screen control switch in "STOP" (C-47).		

4) Submit TR to Maintenance Department to replace shear pin on affected

water boxes.

traveling screen.

Op-Test No.	: <u>ES16LI2</u>	_ Scenario No.: <u>2</u> Event No.: <u>3</u>
Event Descr	iption: " <b>D" Tra</b>	aveling Screen ΔP Fails High
Time	Position	Applicant's Actions or Behavior
		hay secure "D" Circ. Pump at any time, based on rising screen dP. When 2517, Section 5.0 should be referenced (Section 5.0 follows).
		<ul> <li>5.1 <u>IF</u> any Circulating Water Pumps are in the VFD MODE, PERFORM the following: <ul> <li>a. RAISE speed of all VFD mode operating circulating water pumps to 100%.</li> <li>b. <u>IF</u> TRAVELING SCREEN HI, (C06/7, D10) annunciator in alarm, Refer To section 9.0 of this procedure.</li> </ul> </li> <li>5.2 STOP any in progress liquid waste discharges.</li> <li>5.3 ENSURE sodium hypochlorite shocking of bays <i>not</i> in progress.</li> </ul>
		NOTE A 15 to 20 second pause is required after receiving the full closed position indication to allow for full closure prior to opening the crosstie valve.
	BOP	<ul> <li>5.4 CLOSE applicable water box inlet valve for tripped pump:</li> <li>CW-11H, "A" water box inlet</li> <li>CW-11G, "B" water box inlet</li> <li>CW-11F, "C" water box inlet</li> <li>CW-11E, "D" water box inlet</li> </ul>
		CAUTION
		Supplying two condenser waterboxes from one circulating water pump increases traveling screen differential pressure. During periods of actual or predicted severe weather, where fouling is a concern, waterboxes should not be cross- connected. Water boxes may be cross- connected with SM permission.
		<ul> <li>Steps 5.5 – 5.10 apply to CW pumps "A", "B" &amp; "C" and are N/A.</li> <li>1. IF "D" circulating water pump tripped, <u>AND</u> cross tying water boxes is required to maintain turbine load, PERFORM the following: <ul> <li>a. PLACE "P- 6D Breaker" in Pull-To-Lock.</li> <li>b. ENSURE CW-11D, "D" water box outlet, is open.</li> <li>c. OPEN CW-12C, condenser 1B inlet cross-tie</li> </ul> </li> </ul>
	BOP/PEO	<ul> <li>IF "D" circulating water pump tripped, <u>AND</u> isolation of water box is required <u>THEN</u> PERFORM the following:</li> <li>{The remaining steps involve isolating the vacuum priming system and venting of the water box, closing CW-11D (water box outlet), notifing Security and monitoring condenser vacuum. None of these actions have any bearing on the scenario.}</li> </ul>

Op-Test No.:	ES16LI2	Scenario No.: Event No.: _3		
Event Description: " <b>D</b> " Traveling Screen ΔP Fails High				
Time	Position	Applicant's Actions or Behavior		

## Examiner Note: When the actions of ARP 2590C-089 have been addressed and the applicable Technical Specifications have been evaluated, or at lead examiner's direction, proceed to Event 4, CRAC Rad Monitor Failure high.

Op-Test No.: <u>ES16LI2</u>	Scenario No.: Event No.: _4		
Event Description: CRAC Rad Monitor and Filter Fan Failure			
Time Position	Applicant's Actions or Behavior		

• Fa	C.R.A.C.S. IN A ac. 2 CRAC h	AUTO RECIRC MODE" (C-01, C-40) has shifted to recirc. mode. following steps are form ARP 2590A-159 "C.R.A.C.S. IN AUTO RECIRC MODE".	
		AUTOMATIC FUNCTOINS	
		1. CRACS transfers to recircualtion mode.	
	ATC	CAUTION	
		To ensure proper cleanup of Control Room atmosphere, one complete <i>facility related train</i> of Control Room ventilation (i.e. supply, exhaust and filter fans) must be in operation	
		<ul> <li><u>CORRECTIVE ACTIONS</u></li> <li>a. PLACE "NORM/RECIRC MODE, HS-8346" AND "NORM/RECIRC MODE, HS-8359" switches in "RECIR" (C-25A and C-25B).</li> <li>b. Check damper positions.</li> <li>c. VERIFY "CRACS FLTR FAN, F32A, HS-8006" <u>OR</u> "CRACS FLTR FAN, F32E HS-8007," or both, operating (C-25A and C-25B).</li> </ul>	
		9. MONITOR system operation and VERIFY <i>one</i> complete CRACS train remains in operation.	
	ATC	9.1. IF sudden change in Control Room pressure occurs, VERIFY proper supply and exhaust fan operation (local).	
		9.2. IF fan belt failure occurs, STOP affected train AND START other train.	
	ATC	Crew may elect to use OP 2315A for specific guidance in starting Fac. 1 CRAC o use step 9.2 above to start it.	

Op-Test No.: <u>ES16LI2</u> Scenario No.: <u>2</u> Event No.:	4	
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Event Description: CRAC Rad Monitor and Filter Fan Failure

Time Position Applicant's Actions or Behavior
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	4.1.3 ENSURE the following:
	<ul> <li>a. "EMERG FRESH AIR INTAKE RECIRC OVERRIDE, HS-8004C" in "NORM."</li> </ul>
	<ul> <li>b. "EMERG FRESH AIR INTAKE RECIRC OVERRIDE, HS-8004D" in "NORM."</li> </ul>
	c. "FRESH AIR MU DMPR, HV-211," green "CLOSE" position indication light,
	lit. 4.1.4 ENSURE the following:
	a. "NORM/RECIRC MODE, HS-8346" in "O.A."
	b. "MIN FRESH AIR DMPR, HV-202," red "OPEN" position indication light lit. c. "CABLE VAULT EXH DMPR, HV-207," red "OPEN" position indication light
	lit. 4.1.5 ENSURE the following:
	a. "NORM/RECIRC MODE, HS-8359" in "O.A."
	<ul> <li>b. "FRESH AIR DMPR, HV-495," red "OPEN" position indication light lit.</li> <li>c. "CABLE VAULT EXH DMPR, HV-497," red "OPEN" position indication light lit.</li> </ul>
	4.1.8 PERFORM the following simultaneously (C-25A):
	<ul> <li>START "CRACS EXH FAN, F-31A, HS-8001"</li> </ul>
	<ul> <li>START "CRACS SPLY FAN, F-21A, HS-8009"</li> </ul>
	4.1.9 ENSURE the following:
	<ul> <li>"CRACS EXH FAN, F-31A, HS-8001" red run light lit</li> <li>"F-31A, EXH DMPR, HV-206A," red "OPEN" light lit</li> </ul>
	<ul> <li>"F-31A, EXH DMPR, HV-206A," red "OPEN" light lit</li> <li>"CRACS SPLY FAN, F-21A, HS-8009" red run light lit</li> </ul>
	<ul> <li>"F-21A, EXH DMPR, HV-203A," red "OPEN" light lit</li> </ul>
SRO	12. IF radiation monitor alarm is a result of a failure (Green "OPERATE" light is out), SUBMIT a CR to I&C Dept. and Refer To Tech Spec LCO 3.3.3.1.
Examiner Note: The follo	owing is from TS 3.3.3.1
	<ul> <li>3.3.3.1 The radiation monitoring instrumentation channels shown in Table 3.3-6 shall be OPERABLE with their alarm/trip setpoints within the specified limits.</li> <li><u>APPLICABILITY</u>: As shown in Table 3.3-6.</li> <li><u>ACTION</u>:</li> </ul>
500	<ul> <li>a. With a radiation monitoring channel alarm/trip setpoint exceeding the value shown in Table 3.3-6, adjust the setpoint to within the limit within 2 hours or declare the channel inoperable.</li> </ul>
SRO	<ul> <li>b. With the number of OPERABLE channels less than the number of MINIMUM CHANNELS OPERABLE in Table 3.3-6, take the ACTION shown in Table 3.3-6. The provisions of Specification 3.0.3 are not applicable.</li> </ul>
	TABLE 3.3-6, #1.b. Control Room Isolation: Minimum Channels Operable = 2, Action 16
	Action 16-1: with the number of OPERABLE channels one less than required, restore
	within 7 days
Examiner Note: SRO log	s into TSAS 3.3.1b, Action 16-1

Op-Test No.:ES16	Scenario No.:	2	Event No.: 4
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Event Description: CRAC Rad Monitor and Filter Fan Failure

Time Position Applicant's Actions or Behavior	
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	SRO	Refers to TS 3.7.6.1a for CRACS fan inoperable. Examiner Note: The following is from TS 3.7.6.1
		Review Technical Specifications:
		LCO 3.7.6.1: Two independent Control Room Emergency Ventilation Trains shall be OPERABLE.*
		APPLICABILITY: MODES 1, 2, 3, 4, 5 and 6.
	SRO	ACTION:
		<ul> <li>MODES 1, 2, 3, and 4:</li> <li>a. One Control Room Emergency Ventilation Train</li> <li>Required ACTION: Restore the inoperable ventillation train to OPERABLE status within 7 days or shutdown.</li> </ul>
Examiner Note: SRO enters TSAS "3.7.6.1a."		

Examiner Note: When the CRACS malfunction has been mitigated and the SRO has finished evaluating Technical Specifications, or at lead examiner's direction, proceed to Event 5, SGTL #1 SG.

Op-Test No.: <u>ES16LI2</u> Scenario No.: <u>2</u> Event No.: <u>5</u>				
Event Description: Steam Generator Tube Leak #1 SG				
Time	Position	Applicant's Actions or Behavior		

Simulator Operator: When directed, initiate Event 5, Steam Generator Tube Leak #1 SG.		
Indications: • S/G N16 monitor on PPC rising leakage for No. 1 Steam Generator • N16 Alert Alarm (C-06/7 CB-19) • N16 High Alarm (C-06/7 CA-19) • SJAE Discharge RIT-5099 level rising Examiner Note: The following steps are from ARP 2590E-094, CB-19 N16 Alert.		
	SRO/BOP	<ol> <li>AUTOMATIC FUNTIONS         <ol> <li>None</li> <li>CORRECTIVE ACTIONS</li> </ol> </li> <li>OBSERVE radiation monitor indication (RC-14A, PPC N16 screen).</li> <li>COMPARE with trends from RIT-4262, S/G blowdown gross activity and RI-5099, steam jet air ejector.</li> <li>REQUEST Chemistry to perform SP 2833, •Secondary Coolant Analysis for Primary to Secondary Leak Rate and Dose Equivalent Iodine Concentration" to aid in accomplishing the following:         <ol> <li>DETERMINE the presence of primary to secondary leakage.</li> <li>DETERMINE primary to secondary leak rate.</li> <li>IDENTIFY the leaking Steam Generator.</li> </ol> </li> <li>IF primary to secondary leak is confirmed by chemistry analysis or independent radiation monitor indications, Refer To AOP 2569 Steam Generator Tube Leak and perform applicable actions.</li> </ol>
CUE: Chemistry acknowledges request to perform secondary samples for a primary to secondary leak. Wait appropriate time and report back that frisk results indicate activity in No. 1 Steam Generator.		
		Enters AOP 2569, Steam Generator Leak. owing steps are from AOP 2569 Steam Generator Tube Leak. Steps marked continuously or once specified conditions are met.

Op-Test No.: <u>ES16LI2</u> Scenario No.: 2 Event No.: 5		
Event Description: Steam Generator Tube Leak #1 SG		
Time	Position	Applicant's Actions or Behavior

SRO/BOP	<ul> <li>3.1 <u>IF</u> leakage exceeds capability of available charging pumps to maintain pressurizer level, PERFORM the following: <ul> <li>a. IF in MODE 1 or 2, PERFORM the following:</li> <li>TRIP the reactor.</li> <li>Go To EOP 2525, "Standard Post Trip Actions."</li> </ul> </li> <li>b. IF in MODE 3 or lower, PERFORM the following: <ul> <li>ENSURE SIAS is actuated.</li> <li>Go To EOP 2541, Appendix 1, "Diagnostic Flowchart"</li> </ul> </li> <li>3.2 <u>IF</u> "MAIN STEAM LINE HI RAD / INST. FAIL" (C-01 A-30) is received AND is verified to be valid based on other changing RCS indications, PERFORM the following:</li> </ul>
	<ul><li>a. TRIP the reactor.</li><li>b. Go To EOP 2525, "Standard Post Trip Actions."</li></ul>
Examiner Note: Due to	the initial size of the SGTL, steps 3.1 and 3.2 are not applicable at this time.
SRO/BOP	<ul> <li>3.3 IF a SJAE OR SGBD Radiation Monitor alarm is received, ENSURE ALL of the following automatic actions occur:</li> <li>MS-220A and MS-220B, blowdown isolation, close.</li> <li>MS-15, blowdown tank discharge isolation, closes.</li> <li>MS-135, blowdown quench tank discharge isolation, closes.</li> <li>HV-4287 and HV-4288, SG blowdown sample discharge to secondary sample sink, close (secondary sample panel).</li> <li>Blowdown values in PPC reset to "0."</li> </ul>
SRO/BOP	<ul> <li>3.4 IF "N-16 HIGH" (C-06/7 CA19) is received AND is verified to be valid based on other indications, Refer To PPC "N16" screen to determine primary to secondary leak rate and PERFORM the following:</li> <li>a. IF primary to secondary leak rate is greater than or equal to 75 gpd AND is increasing by greater than or equal to 15 gpd / 30 minutes, Refer To AOP 2575, "Rapid Downpower," LOWER reactor power to less than 50% within one hour, and be in Hot Standby within the following two hours.</li> <li>b. Refer To MP-26-EPI-FAP06, "Classification and PARs," and DETERMINE reportability requirements.</li> <li>c. Refer To Technical Specification 3.4.6.2, "Reactor Coolant System Operational Leakage" and PERFORM applicable actions.</li> </ul>
>75 gpd AND increasing	I power reduction is required by this step if primary to secondary leak rate is g by > 15 gpd/30minutes. OP 2575, Rapid Downpower at this time.

Time	Position	Applicant's Actions or Behavior	
Event Description: Rapid Downpower due to Excessive Steam Generator Tube Leak			
Op-Test No.: <u>ES16LI2</u> Scenario No.: <u>2</u> Event No.: <u>6</u>			

	SRO	Enter AOP 2575, Rapid Downpower.
Examiner I Downpowe		blowing steps are from AOP 2575, Rapid Downpower, Section 3.0 Rapid
	SRO	<ul> <li>3.1 PERFORM focus brief on the following:</li> <li>REACTOR TRIP CRITERIA <ul> <li>Parameters associated with automatic reactor or turbine trips are challenged</li> <li>RCS T cold <i>not</i> within 10°F of temperature program and efforts to regain control are unsuccessful</li> </ul> </li> <li>RCS TEMPERATURE CONTROL <ul> <li>RCS T cold to be maintained within 10°F of Attachment 5, "Temperature vs. Power program" using Attachment 10, "Main Turbine Load Set Control."</li> <li>To avoid uncontrolled cooldowns or power transients, sudden changes in RCS temperature or boron concentration should be avoided.</li> </ul> </li> <li>3.2 REQUEST SM/STA to Refer To Attachment 8, "Required Notifications," and PERFORM notifications.</li> </ul>
	ATC	3.3 INITIATE forcing pressurizer sprays.
	SRO	CAUTION: In the case of a dropped CEA, rod motion is <i>not</i> used to initiate downpower. Examiner Note: Caution is N/A
	ATC	3.4 IF <i>not</i> downpowering due to a dropped rod, <u>AND</u> Reactor power is greater than 99% (2673 MWTh) INSERT Group 7 CEAs 10 + 2 steps to initiate downpower.
	BOP	3.5 Using the "Load Speed Control" switch, REDUCE turbine load to maintain Tc on program (+/-2 deg).
	SRO	3.6 Refer To PPC or Reactor Engineering Curve and Data Book and OBTAIN reactivity plan for the initial reactor power condition and desired load reduction.

Time Position	Applicant's Actions or Behavior		
Event Description: Rapid Downpower due to Excessive Steam Generator Tube Leak			
Op-Test No.: <u>ES16LI2</u> Scenario No.: <u>2</u> Event No.: <u>6</u>			

Examiner N	ote: The cre	w should refer to Reactivity Plan RE-G-14.
	BOP	NOTE Attachment 10 "Approximate Load Demand vs. Reactor Power," can be used to correlate the desired power level to a turbine load demand
		setpoint.
	BOP	3.7 Refer To Attachment 9, "Main Turbine Load Set Control," REDUCE turbine load and MAINTAIN Tc on program (+/-2 deg).
Examiner N Load Set Co		owing steps are from AOP 2575 Rapid Downpower Attachment 9 Main Turbin
		CAUTION
	BOP	Operation of the "Load/Speed CONTROL" switch will change turbine load at 600%/hour. and cancel any previous load setpoint.
		NOTE
		Steps provided in this attachment are dependent on plant conditions and may be performed in any sequence, and repeated as necessary.
		<ol> <li><u>IF</u> desired to commence or modify a turbine load ramp, PERFORM the following (HMI "Load" screen):</li> </ol>
		a. IF previous ramp has stopped, SELECT "Load Hold."
	BOP	b. SELECT "Load Setpt" and ENTER desired value.
		c. SELECT "Rate setpt" and ENTER desired value.
		<ul> <li>MHEN ready to commence load reduction, SELECT "Load Resume."</li> </ul>

Op-Test No.: <u>E</u>	<u>S16LI2</u>	Scenario No.: _	2	Event No.: 6
Event Description:	Rapid Do	wnpower due t	o Ex	cessive Steam Generator Tube Leak

Time	Position	Applicant's Actions or Behavior	
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	BOP	<ul> <li>2. <u>IF</u> desired to adjust the "Load Ramp Rate," PERFORM <i>any</i> of the following:</li> <li>SELECT "Rate setpt" and ENTER new value.</li> <li>SELECT "5% / hour," <u>OR</u> "10% / hour," <u>OR</u> "20% / hour."</li> <li>SELECT "Raise" or "Lower" (0.25% / hour change).</li> </ul>
		3. IF Tavg and Tc are high off program, PERFORM the following:
		a. SELECT "Load Hold" to stop ramp.
		<ul> <li><u>WHEN</u> Tavg and Tc are trending back to program, SELECT "Load Resume."</li> </ul>
	BOP	4. IF Tavg and Tc are low off program, PERFORM the following:
		a. JOG the "Load/Speed CONTROL" switch to "Lower."
		<ul> <li><u>WHEN</u> Tavg and Tc are back on program, SELECT Load Setpt" and ENTER desired value.</li> </ul>
		c. IF desired, Go To Step 1 and RESUME turbine load ramp.
		5. IF desired load has been reached SELECT "Load Hold."
Examiner N Downpowe		owing steps are from AOP 2575 Rapid Downpower Section 3.0 Rapid
	ATC	3.8 Based on required rate of downpower, START additional charging pumps as necessary and balance charging and letdown.

Op-Test No.: <u>ES16LI2</u>	Scenario No.:	<u>2</u> Ev	vent No.: <u>6</u>	
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Event Description: Rapid Downpower due to Excessive Steam Generator Tube Leak

Time	Position	Applicant's Actions or Behavior

	ATC	3.9 IF desired to borate from the RWST (preferred method)
		PERFORM the following:
		a. ENSURE at least one charging pump operating.
		b. ENSURE CH-196, VCT makeup bypass, closed.
		c. ENSURE CH-504, RWST to charging suction, open.
		d. OPEN CH-192, RWST isolation.
		e. CLOSE CH-501, VCT outlet isolation.
		f. CHECK charging flow at desired rate.
		g. Go To step 3.11
		Examiner Note: Crew should borate from the RWST.
	SRO/ATC/ BOP	3.11 During the downpower, Refer To Attachment 1, "Rapid Downpower Parameters," and MAINTAIN parameters as specified throughout downpower:
		Examiner note: Attachment 1 Rapid Downpower Parameters:
		<ul> <li>Condensate and heater drain flows and pressures: sufficient to maintain adequate SGFP suction pressure</li> <li>FRV D/P: greater than 40 psid</li> </ul>
		<ul> <li>Turbine load: responding to changes in load demand, with control valves operating together</li> <li>Steam generator levels 55 to 70%.</li> </ul>
		<ul> <li>MSR parameters tracking together</li> <li>Turbine Generator MVARs: as specified by CONVEX</li> </ul>
		<ul> <li>Reactor power: being monitored using delta T power indication</li> <li>ASI: In accordance with reactivity plan or within 0.01 of ESI or per COLR.</li> <li>CEA position: greater than PDIL</li> </ul>
		<ul> <li>Tc: less than or equal to 549 deg</li> <li>Pressurizer level: between 35 and 70%</li> </ul>
		Pressurizer pressure: between 2,225 and 2,300 psia (DNB margin)
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Op-Test No.:	ES16LI2	Scenario No.: Event No.: _6
Event Descrip	otion: Rapid [	Downpower due to Excessive Steam Generator Tube Leak
Time	Position	Applicant's Actions or Behavior

SRO/ATC	
	NOTE
	<ol> <li>Xenon rate of change should be considered when terminating boration.</li> </ol>
	<ol> <li>During rapid downpower, the PPC calorimetric may be inaccurate due to SG level transients. The most accurate available indication of reactor power is RPS delta T power.</li> </ol>
	wer has dropped at least 5%, or at the lead examiner's direction, proceed to or Tube Rupture, Manual Plant Trip.

Op-Test No.:	ES16LI2	Scenario No.: <u>2</u> Event No.: <u>7, 8</u>
Event Descri	ption: Steam	Generator Tube Rupture, Manual Plant Trip, Loss of Safety Injection Flow
Time	Position	Applicant's Actions or Behavior
Simulator	Operator: Wh	nen directed, initiate Event 7, Steam Generator Tube Rupture in #1 SG.
Examiner N	Note: A react	or trip will be required by CAS of AOP 2569.
<ul><li>PRC</li><li>Leto</li></ul>	am Line Radia	ation Monitor Alarm (C-01, A30) MON HI/HI FAIL (C-06, DA-24) I Lowering
Examiner N	Note: The follo	owing steps are from EOP 2525, Standard Post Trip Actions.
	ATC	<ul> <li>Determine Status of Reactivity Control – Reactor Trip</li> <li>1. DETERMINE that Reactivity Control acceptance criteria are met for the reactor by performing ALL of the following steps: <ul> <li>CHECK that all CEAs are fully inserted.</li> <li>CHECK that reactor power is dropping.</li> <li>CHECK that SUR is negative.</li> </ul> </li> </ul>
	BOP	<ul> <li>Determine Status of Reactivity Control – Turbine Trip</li> <li>2. DETERMINE that Reactivity Control acceptance criteria are met for the turbine by performing ALL of the following steps : <ul> <li>a. CHECK that the main turbine is tripped by BOTH of the following:</li> <li>ALL main stop valves are closed.</li> <li>Generator megawatts indicate zero.</li> <li>Turbine speed is lowering.</li> </ul> </li> <li>b. <u>IF</u> 15G-2XI-4, motor operated disconnect, is closed, CHECK that the main Generator output breakers 8T and 9T are open.</li> </ul>
	BOP	<ul> <li>Determine Status of Maintenance of Vital Auxiliaries</li> <li>3. DETERMINE that Maintenance of Vital Auxiliaries acceptance criteria are met by performing ALL of the following steps:</li> <li>3.1. CHECK that ALL Facility 1 and 2 electrical buses are energized: <ul> <li>6.9kV Electrical Buses 25A, 25B</li> <li>4.16kV Non-Vital Electrical Buses 24A, 24B</li> <li>4.16vV Vital Electrical Buses 24C, 24D</li> <li>Vital DC Buses 201A, 201B, DV-10, DV-20</li> <li>Vital AC Instrument Buses VA-10, VA-20</li> </ul> </li> <li>3.2. CHECK that BOTH facilities of service water are operating.</li> <li>3.3. CHECK that BOTH facilities of RBCCW are operating with service water cooling.</li> </ul>

Op-Test No.: <u>ES16LI2</u>	Scenario No.: 2	Event No.: <u>7, 8</u>	
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Time Position Applicant's Actions or Behavior
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ATC	Determine Status of RCS Inventory Control
	<ol> <li>DETERMINE that RCS Inventory Control acceptance criteria are met by performing ALL of the following:</li> </ol>
	4.1. CHECK that <b>BOTH</b> of the following conditions exist:
	Pressurizer level is 20 to 80%
	Pressurizer level is trending to 35 to 70%
	a.1 IF the Pressurizer Level Control System is not operating properly in automatic, RESTORE and MAINTAIN pressurizer level 35 to 70% by performing ANY of the following:
	1) OPERATE the Pressurizer Level Control System.
	2) Manually OPERATE charging and letdown.
	4.2. CHECK that RCS subcooling is greater than or equal to 30 °F

Op-Test No.: <u>ES16LI2</u>	Scenario No.: _	2	Event No.: <u>7, 8</u>
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Time Position	Applicant's Actions or Behavior
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	Determine Status of RCS Pressure Control
A	<ul> <li>5. DETERMINE RCS Pressure Control acceptance criteria are met by BOTH of the following: <ul> <li>CHECK that pressurizer pressure is 1900 to 2350 psia.</li> <li>CHECK that pressurizer pressure is trending to 2225 to 2300 psia.</li> <li>5.1. <u>IF</u> the Pressurizer Pressure Control System is <i>not</i> operating properly in automatic, <u>THEN</u> RESTORE and MAINTAIN pressurizer pressure between 2225 to 2300 psia by performing <b>ANY</b> of the following: <ul> <li>OPERATE the Pressurizer Pressure Control System.</li> <li>Manually OPERATE pressurizer heaters and spray valves.</li> </ul> </li> <li>5.2. <u>IF</u> <b>ANY</b> pressurizer spray valve will <i>not</i> close, <u>THEN</u> STOP RCPs as necessary.</li> <li>5.3. <u>IF</u> any PORV is open <u>AND</u> pressurizer pressure is less than 2250 psia, <u>THEN</u> CLOSE the associated PORV block valve.</li> <li>5.4. <u>IF</u> pressurizer pressure is less than 1714 psia, <u>THEN</u> ENSURE <b>ALL</b> of the following: <ul> <li>SIAS actuated. (C01)</li> <li>CIAS actuated. (C01)</li> <li>EBFAS actuated. (C01)</li> </ul> </li> <li>5.1. F pressurizer pressure is less than 1714 psia <u>AND</u> SIAS actuated, <u>THEN</u> ENSURE ONE RCP in each loop is stopped.</li> <li>5.6. <u>TCOA: IF</u> pressurizer pressure lowers to less than the minimum of Fig. 2 "RCP NPSH Curve" THEN STOP ALL RCPs</li> </ul></li></ul>
A	<ul> <li>Determine Status of Core Heat Removal</li> <li>6. DETERMINE that Core Heat Removal acceptance criteria are met by performing ALL of the following: <ul> <li>a. CHECK that at least one RCP is operating and that loop delta T is less than 10°F</li> <li>a.1 IF RCPs are <i>not</i> operating, OR loop _T is greater than 10° F, <u>THEN</u> PERFORM the following: <ol> <li>PLACE TIC- 4165, steam dump TAVG controller, in manual and closed.</li> <li>PLACE BOTH pressurizer spray valve controllers in manual and CLOSE the valves. <ul> <li>HIC- 100E</li> <li>HIC- 100F</li> </ul> </li> </ol></li></ul> </li> </ul>

Op-Test No.:	ES16LI2	Scenario No.:	2	Event No.: 7,8	

Time	Position	Applicant's Actions or Behavior

	Determine Status of RCS Heat Removal
	<ol> <li>DETERMINE that RCS Heat Removal acceptance criteria are met by ALL of the following conditions:</li> </ol>
ВОР	a. CHECK that at least one steam generator has <b>BOTH</b> of the following conditions met:
	a. Level is 10 to 80%.
	b. Main feedwater or TWO auxiliary feedwater pumps are operating to restore level 40 to 70%.
	<ul> <li>b. CHECK that RCS Tc is being maintained between 530 °F to 535 °F.</li> <li>c. CHECK that <b>BOTH</b> steam generators pressure are 880 to 920 psia.</li> </ul>
ATC	Determine Status of Containment Isolation
	8. DETERMINE that Containment Isolation acceptance criteria are met by <b>ALL</b> of the following:
	a. CHECK that containment pressure is less than 1.0 psig.
	b. CHECK that <b>NONE</b> of the following primary plant radiation monitors have an unexplained alarm or indicate an unexplained rise in activity:
	Radiation Monitors Inside Containment
	<ul> <li>RM-7890, Personnel Access Area</li> <li>RM-7891, Ctmt Refuel Floor Area</li> <li>RM-8240, High Range</li> <li>RM-8241, High Range</li> <li>RM-8123 A and B, Ctmt Atmosphere</li> <li>RM-8262 A and B, Ctmt Atmosphere</li> </ul>
	c. CHECK that NONE of the following steam plant radiation monitors have an unexplained alarm or indicate an unexplained rise in activity:
	Steam Plant Radiation Monitors
	<ul> <li>RM-5099, Steam Jet Air Ejector</li> <li>RM-4262, SG Blowdown</li> <li>RM-4299A and B, Main Steam Line 1</li> <li>RM-4299C, Main Steam Line 2</li> </ul>
	c.1 <u>IF</u> feed is available to <b>BOTH</b> steam generators, <u>THEN</u> THROTTLE feed to the steam generator with the highest radiation readings to maintain level 40 to 45%.

Or	-Test No	ES16LI2	Scenario No ·	2	Event No.: 7.8
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Time	Position	Applicant's Actions or Behavior

	ATC	<ul> <li>Determine Status of Containment Temperature and Pressure Control</li> <li>9. DETERMINE that Containment Temperature and Pressure Control acceptance criteria are met by BOTH of the following steps: <ul> <li>a. CHECK that containment temperature is less than 120 °F. (PPC or avg. of Points 5 and 6)</li> <li>b. CHECK that containment pressure is less than 1.0psig.</li> </ul> </li> <li>10. PERFORM the following: <ul> <li>a. DIAGNOSE the event. Refer To Appendix 1, "Diagnostic Flowchart."</li> </ul> </li> </ul>
	SRO	<ul><li>b. INITIATE Appendix 4, Attachment 4A "Reactor Trip Subsequent Actions."</li><li>c. Go To the appropriate EOP</li></ul>
	ATC/BOP	<ul> <li>{Step 10.b above}</li> <li>Perform Appendix 4, Attachment 4A "Reactor Trip Subsequent Actions".</li> <li>Examiner Note: EOP Appendix 4, Attachment 4A "Reactor Trip Subsequent Actions." are attached to guide.</li> </ul>
Examiner N diagnose t		t Supervisor refers to EOP 2541 Appendix 1, Diagnostic Flowchart to
	SRO	Enters EOP 2534, Steam Generator Tube Rupture.
steps, with preserve a	in the ORP or Safety Funct	lowing steps are from EOP 2534 Steam Generator Tube Rupture. Asterisked r selected FRPs being implemented, may be brought forward to restore or ion. Asterisked steps are "Continuously Applicable," and may be performed ave been accomplished once.
	SRO	<ul> <li>*1. CONFIRM diagnosis of Steam Generator Tube Rupture by performing the following:         <ul> <li>a. CHECK Safety Function Status Check Acceptance Criteria are satisfied.</li> </ul> </li> <li>Examiner Note: SRO checks EOP 2534-001 SGTR Safety Function Status Checks and confirms that all Safety Criteria are satisfied.</li> </ul>

O	o-Test No.:	ES16LI2	Scenario No.:	2	Event No.: 7.8
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Time	Position	Applicant's Actions or Behavior

BOP	<ul> <li>b. CHECK for steam generator tube rupture by performing the following: <ol> <li>CHECK "B" train RBCCW in service.</li> <li>ENSURE 2-RB-210 "Degasifier Effluent Cooler Return Outlet" is open.</li> <li>OPEN the steam generator sample valves: <ol> <li>MS-191A</li> <li>MS-191B</li> </ol> </li> <li>DIRECT Chemistry to perform the following: <ol> <li>Sample both steam generators</li> <li>Frisk the samples</li> <li>Report frisk results</li> <li>Analyze samples for boron and activity</li> </ol> </li> <li>WHEN Chemistry reports that samples have been taken, PERFORM the following: <ol> <li>CLOSE the steam generator sample valves</li> <li>IF SIAS has actuated, <u>AND</u> no other sampling is in progress, CLOSE 2-RB-210, "Degasifier Effluent Cooler Return Outlet"</li> </ol> </li> </ol></li></ul>
CUE: When directed to a	sample Steam Generators, respond 20 minutes later that samples have been
taken. Report that frisk results show indication of activity in No. 1 Steam Generator.	
SRO	<ul> <li>*2. CLASSIFY the event. Refer To MP-26-EPI-FAP06, "Classification and PARs"</li> <li>IF classification requires RCS sampling, Refer To Appendix 46, "Sampling for EAL Determination" and DIRECT Chemistry as required.</li> </ul>
SRO	<ul> <li>*3. PERFORM ALL of the following:</li> <li>OPEN the placekeeper and ENTER the EOP entry time.</li> <li>ENSURE the master alarm silence switch is in "NORMAL".</li> </ul>
ATC	<ul> <li>*4. <u>IF</u> pressurizer pressure is less than 1714 psia, PERFORM ALL of the following: <ul> <li>a. ENSURE SIAS, CIAS and EBFAS have actuated. (C01)</li> <li>b. ENSURE ONE complete facility of CRACS is operating in the recirc mode: (C25)</li> </ul> </li> <li>Facility 1 <ul> <li>HV-203A, Fan F-21A exhaust damper is open.</li> <li>Fan F-21A, supply fan is running.</li> <li>HV-206A, Fan F-31A exhaust damper is open.</li> <li>Fan F-31A, exhaust fan is running.</li> <li>HV-212A, Fan F-32A exhaust damper is open.</li> <li>Fan F-32A, filter fan is running.</li> <li>HV-202, minimum fresh air damper is closed.</li> <li>HV-207, cable vault exhaust damper is closed.</li> </ul> </li> </ul>

Op-Test No.:	ES16LI2	Scenario No.:	2	Event No.: 7,8	

Time	Position	Applicant's Actions or Behavior

	<ul> <li>Facility 2</li> <li>HV-203B, Fan F-21B exhaust damper is open.</li> <li>Fan F-21B, supply fan is running.</li> </ul>
ATC	<ul> <li>HV-206B, Fan F-31B exhaust damper is open.</li> <li>Fan F-31B, exhaust fan is running.</li> <li>HV-212B, Fan F-32B exhaust damper is open.</li> <li>Fan F-32B, filter fan is running.</li> <li>HV-495, fresh air damper is closed.</li> <li>HV-496, exhaust air damper is closed.</li> <li>HV-497, cable vault exhaust damper is closed.</li> </ul>
ATC	<ul> <li>*5. <u>IF</u> SIAS has initiated, PERFORM the following: <ul> <li>a. CHECK at least one train of SIAS, CIAS and EBFAS has properly actuated. (C01X)</li> <li>a.1 IF ANY component is <i>not</i> in its required position, manually ALIGN the applicable component.</li> <li>b. CHECK that safety injection flow is adequate. Refer To Appendix 2, "Figures."</li> <li>b.1 PERFORM ANY of the following to restore safety injection flow within the SI Flow Curve: <ol> <li>ENSURE electrical power to safety injection pumps and valves.</li> <li>ENSURE correct safety injection valve lineup.</li> <li>ENSURE operation of necessary auxiliary systems: <ol> <li>RBCCW</li> <li>ESF Room Coolers</li> <li>START additional safety injection pumps as needed until safety injection flow is within the SI Flow Curve.</li> </ol> </li> </ol></li></ul> </li> <li>Examiners Note: The "C" HPSI pump must be manually started using the control switch on C-01.</li> <li>ENSURE ALL available charging pumps are operating.</li> </ul>

Time	Position	Applicant's Actions or Behavior			
Event Description: Steam Generator Tube Rupture, Manual Plant Trip, Loss of Safety Injection Flow					
Op-Test No	Op-Test No.: <u>ES16LI2</u> Scenario No.: <u>2</u> Event No.: <u>7, 8</u>				

CRITICAL TASK: Manually establish the minimum design Safety Injection System flow within 30 minutes of entering the ORP (CT-1/SGTR-6).

The amount of time required to identify and report a failed safety function is subjective. A criteria may be agreed upon as follows: Usually one Safety Function Status Check is to be completed over a predefined time. Failure to report a failed Safety Function with twice the defined time frame may be reason to fail this critical task (SGTR-6). Safety Function assessments are performed upon entering the ORP/FRP and is required every 15 minutes. (OP 2260).

Time that "C" HPSI pump was started: \_\_\_\_\_ Time SIAS of actuation (manual or auto):\_\_\_\_\_

ATC	<ul> <li>d. ENSURE vital switchgear cooling is operating for each operating ECCS train as follows:</li> <li>Facility 1 <ul> <li>Fan F-51 is running.</li> <li>Fan F-134 is running.</li> <li>SW-178A, service water supply is open.</li> <li>SW-178B, service water supply is open.</li> </ul> </li> <li>Facility 2 <ul> <li>Fan F-52 is running.</li> <li>Fan F-142 is running.</li> <li>Fan F-133 is running.</li> <li>SW-178C, service water supply is open.</li> </ul> </li> </ul>
ATC	<ul> <li>*6. IE pressurizer pressure is less than 1714 psia AND SIAS has initiated, PERFORM the following: <ul> <li>a. ENSURE ONE RCP in each loop is stopped.</li> <li>b. PLACE associated pressurizer spray valve controller RC-100E or RC-100F in manual and CLOSE the valve.</li> <li>c. IF pressurizer pressure lowers to less than the minimum RCP NPSH limit, PERFORM the following: <ol> <li>STOP ALL RCPs.</li> <li>PLACE TIC-4165, steam dump TAVG controller, in manual and closed.</li> </ol> </li> <li>3) PLACE pressurizer spray valve controllers RC-100E and RC-100F in manual and CLOSE the valves.</li> </ul></li></ul>

0	p-Test No.:	ES16LI2	Scenario No.:	2	Event No.: 7,8

Time	Position	Applicant's Actions or Behavior

ATC/BOP	<ul> <li>*7. <u>IF</u> EBFAS has initiated AND the condenser is available, ALIGN the condenser air removal system to Unit 2 stack:</li> <li>a. ENSURE condenser air removal fan, MF55A or MF-55B, is running.</li> <li>b. <u>IF</u> condenser air removal fan MF-55A is operating, ENSURE makeup damper, EB-171, is open.</li> <li>c. OPEN EB-57, condenser air removal to Unit 2 stack.</li> <li>d. ENSURE AC-11, Purge exhaust filter outlet damper is closed.</li> <li>e. OPEN AC-59, Outside air makeup damper.</li> <li>f. START <b>ONE</b> main exhaust fan.</li> <li>g. ENSURE HV-118, Radwaste exhaust damper is closed.</li> <li>h. START F-20, Fuel handling area supply fan.</li> <li>i. ENSURE HV-173, Exhaust mod discharge damper to "MID" position.</li> </ul>
SRO/BOP	*8. COMMENCE an RCS cooldown at the maximum controllable rate to a $T_H$ of less than 515 °F in both loops using the steam dumps.
ATC	<ul> <li>*9. DEPRESSURIZE the RCS by performing the following: <ul> <li>a. MAINTAIN pressurizer pressure within ALL of the following criteria:</li> <li><u>IF</u> RCPs are operating, MAINTAIN RCS pressure above the NPSH curve. Refer to Appendix 2, "Figures."</li> <li>Less than 920 psia</li> <li>Within 50 psi of the most affected steam generator pressure</li> <li>Within the RCS P/T curve limits. Refer to Appendix 2, "Figures."</li> </ul> </li> <li>b. OPERATE main or auxiliary spray.</li> </ul>
ATC	<ul> <li>c. <u>IF</u> HPSI throttle/stop criteria are met, PERFORM <b>ANY</b> of the following to lower RCS pressure:</li> <li>CONTROL charging and letdown.</li> <li>THROTTLE or STOP HPSI flow.</li> </ul>

$Op^{-1}est No \_ LOTOLIZ Scenario No Z Event No 7, 0$	Op-Test No.:	ES16LI2	Scenario No.:	2	Event No.: 7, 8
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Time	Position	Applicant's Actions or Behavior
Time	FUSILIOII	Applicant's Actions of Benavior

BOP/ATC	<ul> <li>*10. <u>IF</u> the main condenser is available, MAINTAIN steaming to the condenser by performing the following: <ul> <li>a. <u>IF</u> MSI is <i>not</i> present AND MSI Block is permitted, BLOCK the automatic initiation as the cooldown and depressurization proceeds.</li> <li>b. IF MSI has actuated AND the following conditions exists: <ol> <li>Steam dumps are available</li> <li>Steaming to the condenser is desired PERFORM the following to open the MSIV for the unisolated steam generator:</li> <li>UNLOCK and CLOSE the disconnect as applicable for MSIV bypass valves: <ol> <li>MS65B (B6207)</li> <li>MS65B (B6208)</li> </ol> </li> </ol></li></ul> </li> </ul>
	<ul><li>2) MS65B (B6208)</li><li>2) CLOSE the steam dump valves.</li></ul>
	<ul> <li>4) THROTTLE the ADV as necessary to achieve less than 100 psid.</li> <li>5) WHEN differential pressure across the MSIVs is less than 100 psid, OPEN the MSIVs.</li> </ul>
	6) CLOSE the MSIV bypass valves.

Op-Test No.:	ES16LI2	Scenario No.:	2	Event No.: 7,8	

Time Posit	ion	Applicant's Actions or Behavior

SRO/ATC/ BOP	<ul> <li>c. <u>IF</u> MSI will actuate following the isolation of the most affected steam generator <u>AND</u> the following conditions exists:</li> <li>Steam dumps are available</li> <li>Steaming to the condenser is desired PERFORM the following to ensure steaming to the main condenser is maintained: <ol> <li>CLOSE the steam dump valves.</li> <li>Manually initiate MSI.</li> <li>IF differential pressure across the MSIV for the least affected steam generator is greater than 100 psid, OPEN the MSIV.</li> </ol> </li> <li>c.1 <u>IF</u> differential pressure across the MSIV for the least affected steam generator is greater than 100 psid, OPEN the MSIV.</li> <li>c.1 <u>IF</u> differential pressure across the MSIV for the least affected steam generator is greater than 100 psid, OPEN the MSIV.</li> <li>c.1 <u>IF</u> differential pressure across the MSIV for the least affected steam generator is greater than 100 psid, OPEN the MSIV.</li> <li>c.1 <u>IF</u> differential pressure across the MSIV for the least affected steam generator is greater than 100 psid, OPEN the MSIV.</li> <li>c.1 <u>IF</u> differential pressure across the disconnect, as applicable, for least affected steam generator MSIV bypass valve:     <ul> <li>MS-65B (B5207)</li> <li>MS-65B (B6208)</li> </ul> </li> <li>2) ENSURE the steam dump valves are closed.</li> <li>3) OPEN the MSIV bypass valve for the least affected steam generator.</li> <li>4) THROTTLE the ADV as necessary to achieve less than 100 psid.</li> <li>5) <u>WHEN</u> differential pressure across the MSIV for the least affected steam generator is less than 100 psid, OPEN the MSIV.</li> <li>6) CLOSE the MSIV bypass valve.</li> </ul>
SRO/ATC	*11. <u>IF</u> SIAS is <i>not</i> present <u>AND</u> SIAS Block is permitted, BLOCK the automatic initiation as the cooldown and depressurization proceeds.
BOP	<ul> <li>*12. <u>IF</u> offsite power has been lost OR the condenser is <i>not</i> available, PERFORM the following:</li> <li>a. CLOSE <b>BOTH</b> MSIVs.</li> <li>b. ENSURE <b>BOTH</b> MSIV bypass valves are closed.</li> <li>c. OPEN AR-17, condenser vacuum breaker.</li> </ul>

Op-Test No.:	ES16LI2	Scenario No.:	2	Event No.: 7,8	

Time	Position	Applicant's Actions or Behavior

	<ul> <li>*13. DETERMINE the most affected steam generator by considering ALL of the following:</li> <li>Steam generator activities</li> <li>Main steam piping radiation levels</li> <li>Steam generator level change when <i>not</i> feeding</li> <li>Steam generator blowdown activities</li> <li>Steam generator mismatch in level with essentially the same feed and steaming rate for both steam generators</li> <li>Feed flow mismatch between steam generators</li> <li>Steam flow versus feed flow mismatch in a steam generator prior to the trip</li> </ul>
SRO/BOP	<ul> <li>*14. WHEN BOTH RCS hot leg temperatures are less than 515° F, ISOLATE the most affected steam generator by performing the following:</li> <li>Number 1 Steam Generator <ul> <li>a. RECORD in the placekeeper, time and TC of the operating loop.</li> <li>b. ENSURE ALL of the following for the associated ADV: <ul> <li>ADV is in AUTO, PIC-4223</li> <li>ADV setpoint at 920 psia</li> <li>ADV is closed</li> </ul> </li> <li>c. ENSURE the MSIV, MS-64A, is closed.</li> <li>d. ENSURE the MSIV bypass valve, MS-65A, is closed.</li> <li>e. CLOSE the main feedwater regulating bypass valve,FW-41A.</li> <li>f. ENSURE the main feedwater block valve, FW-42A is closed.</li> <li>g. PLACE main feed isolation air assisted check valve, FW-5A to "CLOSE."</li> <li>h. ENSURE the steam generator blowdown isolation valve, MS-220A is closed.</li> <li>i. PLACE BOTH auxiliary feed "OVERRIDE/MAN/START/RESET" handswitches in "PULL TO LOCK".</li> </ul> </li> </ul>
	j. CLOSE the aux feedwater regulating valve, FW-43A.

CRITICAL TASK: Isolate the affected SG (CT-2/SGTR-5) [Within 60 minutes of the tube rupturing, per OP 2260]

Time SG Tube Rupture occurred (time of manual trip): \_\_\_\_\_ Time Affected SG Isolated (Step #14 complete):\_\_\_\_\_

Once the affected SG is isolated, or at the lead examiner's direction, the scenario is completed.

Event Description: EOP 2541, Followup Actions, Appendix 4A, Reactor Trip Subsequent Actions	cion mp Subsequent Actions	
Time Position Applicant's Actions or Behavior		

Examiner Note: The fo Subsequent Actions. ATC	<ul> <li>Ilowing steps are from EOP 2541, Followup Actions, Appendix 4A, Reactor Trip</li> <li>1. IF charging pumps suction is aligned to the VCT, <u>THEN</u> CHECK VCT level is between 72% to 86%:         <ul> <li>a. IF VCT level is less than 72%, THEN ALIGN charging pump suction to RWST as follows:</li> <li>1) OPEN CH- 192, RWST isolation.</li> <li>2) ENSURE CH- 504, RWST to charging suction is open.</li> <li>3) CLOSE CH- 501, VCT outlet isolation.</li> <li>4) ENSURE CH- 196, VCT makeup bypass is closed.</li> <li>b. IF VCT level is greater than 88%, THEN PLACE CH- 500, letdown divert handswitch, to the "RWS" position, and divert as</li> </ul> </li> </ul>
ATC	<ul> <li>required to maintain VCT level 72% to 86%.</li> <li>2. TCOA: <u>IF</u> SIAS actuated, <u>THEN</u> ENSURE <b>ONE</b> complete facility of CRAC operating, in RECIRC mode, as follows: (C25A/B)</li> <li>Facility 1 <ul> <li>HV- 203A, Fan F- 21A exhaust damper open</li> <li>Fan F- 21A, supply fan running</li> <li>HV- 206A, Fan F- 31A exhaust damper open</li> <li>Fan F- 31A, exhaust fan running</li> <li>HV- 212A, Fan F- 32A exhaust damper, open</li> <li>Fan F- 32A, filter fan, running</li> <li>HV- 202, minimum fresh air damper, closed</li> <li>HV- 207, cable vault exhaust damper, closed</li> <li>HV- 208, exhaust air damper, closed</li> </ul> </li> </ul>
	<ul> <li>Facility 2</li> <li>HV- 203B, Fan F- 21B exhaust damper open</li> <li>Fan F- 21B, supply fan running</li> <li>HV- 206B, Fan F- 31B exhaust damper open</li> <li>Fan F- 31B, exhaust fan running</li> <li>HV- 212B, Fan F- 32B exhaust damper, open</li> <li>Fan F- 32B, filter fan, running</li> <li>HV- 495, fresh air damper, closed</li> <li>HV- 496, exhaust air damper, closed</li> <li>HV- 497, cable vault exhaust damper, closed</li> </ul>

0	o-Test No.:	ES16LI2	Scenario No.:	2	Event No.: 7	
-						

Event Description: EOP 2541, Followup Actions, Appendix 4A, Reactor Trip Subsequent Actions

Time	Position	Applicant's Actions or Behavior

	<ol> <li>TCOA: <u>IF</u> SIAS not actuated, <u>THEN</u> CHECK ONE facility of CRAC operating, in NORMAL mode, as follows: (C25A/B)</li> </ol>
ATC	<ul> <li>Facility 1</li> <li>HV- 203A, Fan F- 21A exhaust damper is open</li> <li>Fan F- 21A, supply fan running</li> <li>HV- 206A, Fan F- 31A exhaust damper open</li> <li>Fan F- 31A, exhaust fan running</li> </ul>
	<ul> <li>Facility 2</li> <li>HV- 203B, Fan F- 21B exhaust damper open</li> <li>Fan F- 21B, supply fan running</li> <li>HV- 206B, Fan F- 31B exhaust damper open</li> <li>Fan F- 31B, exhaust fan running</li> </ul>
ATC	<ul> <li>4. <u>IF</u> charging pumps suction aligned to the RWST <u>AND</u> boration not required, <u>THEN</u> RESTORE charging pump suction to VCT as follows: <ul> <li>a. CHECK BOTH of the following:</li> <li>1) VCT level between 72% and 86%</li> <li>2) VCT pressure greater than 15 psig</li> <li>b. CHECK letdown is in service.</li> <li>c. OPEN CH- 501, VCT outlet isolation.</li> <li>d. CLOSE CH- 192, RWST isolation.</li> </ul> </li> </ul>
ВОР	5. CHECK instrument air pressure greater than 90 psig and stable.
BOP	<ol> <li>IF AFAS has actuated, WHEN BOTH steam generators are restored to greater than 33%, THEN PERFORM the following:         <ul> <li>a. PLACE the following switches in "M" (Manual) and ADJUST to obtain desired flow (C- 05):                 <ul></ul></li></ul></li></ol>
	once the SG Tube Rupture is identified and AFW flow is under control.

Op-T	est No.:	ES16LI2	Scenario No.:	2	Event No.: 7	

Event Description: EOP 2541, Followup Actions, Appendix 4A, Reactor Trip Subsequent Actions

Time	P P

osition

Applicant's Actions or Behavior

BOP	<ul> <li>7. CHECK Main Condenser is available, as indicated by ALL of the following: <ul> <li>At least ONE MSIV open</li> <li>Condenser vacuum better than 15 inches HG - ABS (0 to 15 inches)</li> <li>At least ONE condensate pump operating</li> <li>At least ONE Circ Water pump operating</li> </ul> </li> <li>8. OPEN HD- 106, subcooling valve.</li> </ul>
BOP	9. ENSURE BOTH heater drain pumps stopped.
BOP	<ul> <li>10. <u>IF</u> MFW is supplying feed to the steam generators, <u>THEN</u> PERFORM the following: <ul> <li>a. ENSURE that only <b>ONE</b> main feedwater pump is operating.</li> <li>b. ENSURE that <b>BOTH</b> main feed block valves are closed: <ol> <li>FW- 42A</li> <li>FW- 42B</li> </ol> </li> <li>c. ADJUST the operating main feedwater pump pressure to 50 to 150 psi greater than SG pressure.</li> <li>d. ENSURE BOTH main feed reg bypass valves are throttled to control SG level: <ol> <li>LIC- 5215</li> <li>LIC- 5216</li> </ol> </li> <li>e. <u>IF</u> Main Feedwater Pump A is secured, <u>THEN</u> CLOSE the following: <ol> <li>FW- 38A, main feedwater pump discharge valve</li> <li>FIC- 5237, main feedwater pump mini flow recirc valve</li> <li>FW- 38B, main feedwater pump discharge valve</li> <li>FIC- 5240, main feedwater pump mini flow recirc valve</li> </ol> </li> </ul></li></ul>
BOP	<ul> <li>11. <u>IF</u> BOTH MFW pumps are secured, <u>THEN</u> PERFORM the following:</li> <li>a. CLOSE BOTH main feedwater pump mini flow recirc valves.</li> <li>FIC- 5237</li> <li>FIC- 5240</li> </ul>
BOP	<ul> <li>12. <u>IF</u> 25A OR 25B is energized, <u>THEN</u> ALIGN condensate pumps as follows:</li> <li>a. ENSURE <b>ONE</b> pump is running.</li> <li>b. ENSURE <b>ONE</b> pump is in "PULL TO LOCK."</li> <li>c. ENSURE <b>ONE</b> pump is in "AUTO."</li> </ul>
	Examiner Note: End of Attachment 4- A

## SIMULATOR SCENARIO #3

Appen	idix D		Scenario Outline	Form ES-D-1
Facility: Mills Examiners:			Scenario No.: 3 Operators:	Op-Test No.: ES16Ll3 SRO ATC BOP
Initial Conditi	ions: 100% Po	ower IC-141		
Turnover: 10	0% Power, st	eady state,	Nothing out of service. 24E	E is aligned to 24C.
2. LOCA 3. 2260 an MS	A-13 Trip two F A-12 (TCOA) T	Frip ALL RCF ESDE-6); Isc	IAS actuation and a LOCA Ps within 5 minutes of NPS olate Aux Feed Water to th	1 0
Event No.	Malf. No.	Event Event Description Type*		
Not Run 1 Deleted	RP13D	TS (S)	"D" Ch. SG #1 Level (LI1	13D) fails to 0%
2	C03-A18B	C (ATC/S)	"A" RCP Anti Rev Rot Flo	ow Low
3	RC20A	C,TS (ATC/S)	"A" RCP Seal Cooler Lea	ak of 5-8 gpm
4	N/A	R (ALL)	Plant shutdown due to R	CS leak
5	RC20A (1005)	M (ALL)	"A" RCP Seal Cooler Rup of 550 gpm	pture resulting in an Inter-System SB-LOCA
6	MS02B	C (ALL)		5E06 lbm/hr), upstream of #2 MSIV on the y following the Reactor Trip
7	FW30A FW20B	C (BOP/S)	"A" AFW Pump degraded performance and "B" AFW pump trips. Start the TDAFW pump.	

Tar	get Quantitative Attributes (Per Scenario; See Section D.5.d)	Actual
1.	Total malfunctions (5–8)	5
2.	Malfunctions after EOP entry (1–2)	1
3.	Abnormal events (2–4)	2
4.	Major transients (1–2)	1
5.	EOPs entered/requiring substantive actions (1-2)	1
6.	EOP contingencies requiring substantive actions (0–2)	1
7.	Critical tasks (2–3)	3

## NRC 2016, Scenario 3 Summary:

The crew will take the shift with the unit at 100% power, steady state, nothing out of service.

**Event 1**: The crew takes the shift, then at the discretion of the Examiner malfunction for Ch. "D" SG #1 Level (LI-113D) failure to 0% is triggered. Crew will refer to applicable ARP(s), bypass the affected inputs on Channel "D" of RPS and log into the applicable instrumentation T.S.A.S.

**Event 2**: At the discretion of the Examiner a malfunction for "A" RCP Anti Reverse Rotation flow alarm will annunciate providing the prelude to the "A" RCP Seal Cooler leak. The Crew will refer to ARP 2590B-074 and start the "A" RCP Lift pump, evaluate the need for a Reactor Trip and then submit a CR.

**Event 3**: At the discretion of the Examiner a malfunction for RCP "A" seal cooler leak into RBCCW of 5-8 gpm. The Crew will enter AOP 2568 Reactor Coolant System Leak and validate by stabilizing PZR level, may start an additional Charging Pump and manually adjusting the bias on letdown. The U.S. will enter a Shutdown T.S.A.S and monitor for EAL Classification threshold. U.S. will direct a shutdown and transition to AOP 2575.

**Event 4**: The crew will enter AOP 2575 Rapid Downpower per the RCS Leak T.S.A.S. and commence a downpower. ATC will insert Group 7 Rods 10 steps while BOP reduces Turbine load to maintain RCS  $T_{COLD}$ . The Crew will then align for boration from the RWST requiring the ATC operator to start an additional Charging pump if not already started and the BOP to setup the Turbine HMI to lower load to maintain RCS  $T_{COLD}$ .

**Event 5**: At the discretion of the Examiner the malfunction for "A" RCP Seal Cooler rupture will causing a small break LOCA of approximately 550 gpm requiring the crew will validate using RCS parameter imitate and initiate a Reactor Trip and transition to perform EOP 2525 SPTA.

**Event 6**: On the Reactor trip during the performance of EOP 2525 an Excess Steam Demand Event will be initiated outside of CTMT, upstream of #2 S/G MSIV (Non-Isolable) requiring the BOP stop steaming to the condenser by closing both MSIVs and to secure feed the #2 S/G. When the S/G blows dry the BOP will stabilize RCS temperature using the unaffected S/G ADV. The U.S. will diagnose 2 events and enter EOP 2540 and implement the Resource Assessment Trees.

**Event 7:** Five minutes after the plant trip, the "A" Motor driven Aux Feedwater Pump performance will degrade and the "B" Motor driven Aux Feedwater pump will trip, requiring the BOP to start the Turbine driven Aux Feedwater Pump, if not previously already started.

The US should transition to the Functional Recovery Procedure, EOP 2540, and the Crew will begin addressing the CTMT Isolation Safety Function.

The crew is required to isolate the RCS leak into RBCCW and Isolate the #2 S/G while stabilizing RCS temperature after the #2 S/G blowdown.

	INPL	JT SUMM	ARY				
Either INP	UT or VERIFY the following functions:						
ID Num	n Description		Ramp Time	Event Time	Sev or Value	Final Value	Rel Order
	MAL	FUNCTIC	NS				
RP13D	SG #1 Ch. "D" Level failure			E-1	0%	0%	1
C03-A18B	"A" RCP Anti Rev Rot Flow Low			E-2	ON	ON	2
RC20A	"A" RCP Seal Cooler Leak of 8 gpm		2 min	E-3	8 gpm	8 gpm	3
RC20A (1005)	"A" RCP Seal Cooler Rupture of 550 gpm			E-5	550 gpm	550 gpm	5
MS02B	ESD outside CTMT, upstream of #2 MSIV		30 Sec.	E-30	4.75E06	4.75E06	6
FW30A	"A" AFW pump degraded performance.	5 min.		E-30	100%	100%	7
FW20B	"B" AFW pump trip.	5 min.		E-30	N/A	N/A	7
FW20A	"A" AFW pump trip.			E-8	N/A	N/A	8
C03-A18B	"A" RCP Anti Rev Rot Flow Low			E-9	NORMAL	NORMAL	2
	REMO	TE FUNC	TIONS	1			
CCR35	RBCCW pump "A" Rad Mon isolation			E-10	CLOSE	CLOSE	10
CCR35	RBCCW pump "A" Rad Mon isolation			E-11	OPEN	OPEN	11
CCR36	RBCCW pump "B" Rad Mon isolation			E-12	CLOSE	CLOSE	12
CCR36	RBCCW pump "B" Rad Mon isolation			E-13	OPEN	OPEN	13
CCR37	RBCCW pump "C" Rad Mon isolation			E-14	CLOSE	CLOSE	14
CCR37	RBCCW pump "C" Rad Mon isolation			E-15	OPEN	OPEN	15
	0		S				

Op-Test No.	: <u>ES16LI3</u>	Scenario No.: <u>3</u> Event No.: <u>1</u>
Event Descr	iption: "A" Ch	n. RWST Level Failure
Time	Position	Applicant's Actions or Behavior
Simulator	Operator: W	hen directed, initiate Event #1, SG #1 Ch. "D" Level (LI-113D) fails to 0%
• "SG	s Available: à LO LEVEL 1 PS PRE TRIP'	FRIP CH D" C04, DB-1 ' C04, AA-7
Examiner I	Note:	
The follow	ing steps are	e from ARP 2590C-008 (C04, DB-1)
	ATC	<ol> <li>AUTOMATIC FUNCTIONS         <ol> <li>If 2 RPS channels actuate, reactor trips.</li> </ol> </li> <li>CORRECTIVE ACTIONS         <ol> <li>IF reactor trips, Go To EOP 2525, Standard Post Trip Actions" and PERFORM necessary corrective actions.</li> <li>OBSERVE channel D" SG level indication and COMPARE to other safety channel indications (C-05, PPC).</li> <li>IF SG level is less than 49.5% AND no automatic reactor trip has occurred, manually TRIP reactor and Go To EOP 2525, Standard Post Trip Actions."</li> <li>IF SG level is greater than 49.5% AND alarm is due to instrument malfunction, PERFORM the following:             <ol> <li>OBTAIN necessary keys and PERFORM applicable actions to bypass channel "D" SG level bistables on RPS, C-517, and C-518.</li> <li>Refer To the following Technical Specifications LCOs and DETERMINE applicability:                 <ul> <li>3.3.1.1, Table 3.3-1</li> <li>3.3.2.1, Table 3.3-3</li> </ul> </li> </ol></li> </ol></li></ol>
	SRO	<ul> <li>4.3 SUBMIT Trouble Report to I&amp;C Department.</li> <li>3.3.1.1 As a minimum, the reactor protective instrumentation channels and bypasses of Table 3.3-1 shall be OPERABLE.</li> <li>TOTAL TRIP MIN MODE ACTION</li> <li>7. Steam Generator Water level – Low 4 2 3 1, 2 2</li> <li>4b) ≥ 1850 psia, operation may continue with the inoperable channel in the bypassed condition, provided the following condition is satisfied:</li> <li>Action 2 - With the number of OPERABLE channels one less than the Total Number of Channels, operation may continue provided the following conditions are satisfied:</li> <li>a. The inoperable channel is placed in either the bypassed or tripped condition within 1 hour. The inoperable channel shall either be restored to OPERABLE status, or placed in the tripped condition, within 48 hours.</li> </ul>

	3.3.2.1 The engineered safety feature actuation system instrumentation channels and bypasses shown in Table 3.3-3 shall be OPERABLE with their trip setpoints set consistent with the values shown in the Trip Setpoint column of Table 3.3-4.	
	TOTAL TRIP MIN MODE ACTION	
0.00	9. Auxiliary Feed Water SG Level - Low 4 2 3 1, 2, 3 2	
SRO	<ul> <li>Action 2 - With the number of OPERABLE channels one less than the Total Number of Channels, operation may continue provided the following conditions are satisfied:</li> <li>a. The inoperable channel is placed in either the bypassed or tripped condition within 1 hour.</li> </ul>	
Examiner Note: SRO should note that performing the actions of the ARP and bypassing channel "D"		
on RPS, C517 and C-518 meets the action for the LCO 3.3.1.1 and 3.3.2.1.		
Examiner Note: Once the actions of TS 3.3.1.1 and 3.3.2.1 have been completed, or at lead examiner's		

direction, proceed to Event 2, "A" RCP Anti Rev Rot Flow Low.

Op-Test No.:	ES16LI3	Scenario No.: <u>3</u> Event No.: <u>2</u>	
Event Descri	ption: " <b>A" R</b>	CP Anti Rev Rot Flow Low	
Time	Position	Position Applicant's Actions or Behavior	
Simulator	Operator: W	hen directed, initiate Event #2, "A" RCP Anti Rev Rot Flow Low.	
	s Available: CP A ANTIRE	V ROT FLOW LO" C02/3 AB-18	
Examiner I	Examiner Note: The following steps are from ARP 2590B-074 (C02/3 AB-18).		
		AUTOMATIC FUNCTIONS	
		1. None	
	АТС	CORRECTIVE ACTIONS	
	AIC	NOTE:	
		Oil lift pump may be operated indefinitely.	
		1. START "RCP-A LIFT PPS, P-51A/53A" (C-03).	
Simulator	Operator: ~1	I minute after the lift pump is started, trigger Event 9 to clear alarm AB-18.	
	SRO	2. NOTIFY OMOC (Duty Officer).	
Cue: OMO	C is notified.		
	ATC	<ol> <li>MONITOR <i>all</i> "A" RCP bearing temperatures (C-04R or PPC).</li> <li>CHECK the following alarm windows <i>not</i> lit (C-02/3):         <ul> <li>"RCP A ANTIREV BRG TEMP HI" (AA-19)</li> <li>"RCP A UPPER GUIDE TEMP HI" (BB-20)</li> <li>"RCP A UPPER THRUS TEMP HI" (CB-20)</li> </ul> </li> <li>IF alarm does <i>not</i> clear <u>AND</u> <i>any</i> alarm listed in step 4. is valid, PERFORM the following:         <ul> <li>TRIP reactor and turbine.</li> <li>STOP "A" RCP.</li> <li>Refer To EOP 2525, "Standard Post Trip Actions" and PERFORM required actions.</li> </ul> </li> </ol>	
Examiner l	Note: Once the second	use none of the annunciators listed in Step 4 are in alarm, Step 5 is N/A. he actions of ARP 2590B-074 have been completed, or at lead examiner's vent 3, "A" RCP Seal Cooler Leak.	

Op-Test No.:	ES16LI3	_ Scenario No.: <u>3</u> Event No.: <u>3</u>				
Event Descri	Event Description: "A" RCP Seal Cooler Leak					
Time	Position	Position Applicant's Actions or Behavior				
Simulator C	Operator: Who	en directed, initiate Event 3, "A" RCP Seal Cooler Leak of 5-8 gpm				
Indications	Available:					
• PPC						
	-	e TK level rise C06				
		R TEMP HI" C-02/3 DB-17 ( <u>no</u> alarm, but may be referenced for trip criteria) RAD HI HI/FAIL" C-06/7 DA-24				
Examiner N	Examiner Note: The following steps are from ARP 2590B-072 (C-02/3 DB-17).					
		AUTOMATIC FUNCTIONS				
		1. None				
		CORRECTIVE ACTIONS				
	ATC	1. IF "A" RCP RBCCW outlet temperature is above 125°F, PERFORM the				
		following: 1.1. TRIP Reactor.				
		1.2. TRIP Turbine.				
		1.3. STOP "A" RCP. 1.4. Go To EOP 2525, "Standard Post Trip Actions."				
		90B-072 (C-02/3 DB-17) may be referenced for trip criteria. However, the IId stabilize far enough below 125°F to not require a plant trip/RCP shutdown.				
Examiner N	lote: The foll	owing steps are from ARP 2590E-135 (C-06/7 DA-24).				
		AUTOMATIC FUNCTIONS				
		1. None				
		CORRECTIVE ACTIONS				
		NOTE:				
	SRO	<ol> <li>When the "PROCESS MON RAD HI/HI FAIL" alarm is received, all TS and REMODCM radiation monitors associated with this alarm are considered to be INOPERABLE until the alarm is cleared.</li> </ol>				
		<ol> <li>If the 12 hour maintenance window is applied and the "PROCESS MON RAD HI/HI FAIL" alarm will remain longer than 12 hours, then log into appropriate action statements. Track 12 hour maintenance window using LCO Module.</li> </ol>				

Event Description: "A" RCP Seal Cooler Leak	Op-Test No.: <u>ES16LI3</u>	cenario No.: <u>3</u>	Event No.: 3		
	Event Description: "A" RCP	al Cooler Leak			

Time	Position	Applicant's Actions or Behavior
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	SRO	<ol> <li>Refer To the following LCOs and DETERMINE applicability:         <ul> <li>TS 3.4.6.1 (RM-8123A, RM-8262A - applies only if alarm windows C-01, A-28, CH "A" CTMT AIR PARTICULATE RADIATION HI and B-28, CH "B" CTMT AIR PARTICULATE RADIATION HI are <i>not</i> OPERABLE).</li> <li>TS 3.3.3.1 (RM-8123A, RM-8262A - applies only if alarm windows C-01, A-28, CH "A" CTMT AIR PARTICULATE RADIATION HI are <i>not</i> OPERABLE).</li> <li>TS 3.3.3.1 (RM-8123A, RM-8262A - applies only if alarm windows C-01, A-28, CH "A" CTMT AIR PARTICULATE RADIATION HI and B-28, CH "B" CTMT AIR PARTICULATE RADIATION HI are <i>not</i> OPERABLE).</li> <li>REMODCM IV.C.1 TABLE IV.C-1 (RM-6038, RM-4262- REMODCM allows use of 12 hr Maintenance Window)</li> <li>REMODCM IV.C.2 TABLE IV.C-3 (RM-8132A/B- REMODCM allows use of 12 hr Maintenance Window), (RM-9095 applies only if performing a Waste Gas discharge)</li> </ul> </li> </ol>		
	BOP	2. OBSERVE which process radiation monitor is alarming (RC-14).		
		/ rad monitor RM-6038 is in alarm, therefore the SRO should note that <i>E IV.C-1</i> applies in Step #1 above.		
	BOP	3. IF no "ALARM" OR "INSTRUMENT FAIL" lights lit, PERFORM the following (RC-14):		
Examiner N	Examiner Note: Step #3 is N/A due to RBCCW rad monitor RM-6038 being in alarm.			
	SRO/BOP	<ol> <li>Refer To ARP 2590H, "Alarm Response for Control Room Radiation Monitor Panels," and PERFORM applicable corrective actions for alarming radiation monitor.</li> </ol>		
		O should refer to ARP 2590H for additional guidance and direct the BOP to guidance of ARP 2590-135 C-06/7 DA-24, or shift to performing ARP 2590H.		
		590H, the SRO should select "RBCCW GROSS ACTIVITY RIC-2300B", RC-14D, ollowing are the applicable steps from ARP 2590H-041A.		
		AUTOMATIC FUNCTIONS		
		1. None		
	SRO/BOP	CORRECTIVE ACTIONS		
		<ol> <li>OBSERVE radiation monitor indication (RC-14D, PPC).</li> <li>COMPARE indication to setpoint indicated on "SETPOINT" sticker on module.</li> <li>CHECK "PROCESS RADIATION, RJR-9373" (Channel 5) for trend data RC-14D).</li> </ol>		

Op-Test No.:	ES16LI3	_ Scenario No.: <u>3</u> Event No.: <u>3</u>		
Event Description: "A" RCP Seal Cooler Leak				
Time	Position	Applicant's Actions or Behavior		

	1	
• "A" i • "B" i	RBCCW pump RBCCW pump	<ul> <li>4. <u>IF</u> alarm is high, PERFORM the following:</li> <li>4.1. One at a time, UNLOCK and CLOSE the following valves and MONITOR instrument response for determination of which header has in-leakage:</li> <li>"A" RBCCW pump radiation element flow stop, 2-RB-43</li> <li>"B" RBCCW pump radiation element flow stop, 2-RB-41</li> <li>"C" RBCCW pump radiation element flow stop, 2-RB-39</li> </ul> en directed, trigger the applicable Event to close the requested RBCCW valve <ul> <li>2-RB-43 – Event #10 to CLOSE, Event #11 to RE-OPEN.</li> <li>2-RB-41 – Event #12 to CLOSE, Event #13 to RE-OPEN.</li> <li>2-RB-43 – Event #14 to CLOSE, Event #15 to RE-OPEN.</li> </ul>
	SRO/BOP	<ul> <li>4.2. REQUEST Chemistry Department sample the following for gamma activity: <ul> <li>Both RBCCW headers</li> <li>Service water effluent per SP 2854, Reactor Building Closed Cooling Water (RBCCW) Radiation Monitor RM 6038 Inoperative"</li> </ul> </li> <li>4.3. Refer To OP 2383C, Radiation Monitor Alarm Setpoint Control" and EVALUATE need to adjust alarm setpoint.</li> <li>4.4. IF Chemistry results indicate short-lived activity, Go To AOP 2568, "RCS Leak."</li> </ul>
Examiner N enter AOP 2	lote: The ren 2568, "RCS L	sample request, report short-lived activity detected in "A" RBCCW header. naining steps of ARP 2590H-041A are not applicable and the SRO should eak" at this time. m AOP 2568 follow the remaining applicable steps of ARP 2590E-135.
The follow	ing steps cor	ntinue from ARP 2590E-135 (C-06/7 DA-24), if they are addressed, but have no or crew actions.
	SRO/BOP	<ol> <li><u>IF</u> RM-8123A/B or RM-8262A/B, alarms, Refer To the following for additional guidance (C-01):</li> <li>CHECK "DIGITAL COMPARATOR STACK PARTICULATE, RI8132A" (RC14C), "ALARM" relay light <i>lit</i>.</li> </ol>
Examiner N	lote: Step #5	and #6 are N/A due to RBCCW rad monitor RM-6038 being in alarm.

Op-Test No.: <u>ES16LI3</u> Scenario No.: <u>3</u> Event No.: <u>3</u>			
Event Descri	Event Description: "A" RCP Seal Cooler Leak		
Time	Time Position Applicant's Actions or Behavior		

		7. DETERMINE cause of alarm and TRY to reset.
	SRO/BOP	<u>NOTE:</u>
		The "Latch/Reset" pushbutton on RIC-2300A, RIC-2300B, RIC-8123, and RIC-8262 is used to reset a "latched", or locked-in alarm.
		<ol> <li>ATTEMPT to reset module by pushing "RESET" button.</li> <li><u>IF</u> alarm does <i>not</i> reset, to allow other alarms to annunciate, PERFORM applicable action:</li> <li>9.1. IF RM-6038 alarms, OBTAIN key and place "NORMAL BYPASS CH.1 BYPASS, HS-6038B," switch to "BYPASS" (RC-14D).</li> </ol>
Examiner I	Note: The ren	naining steps from ARP 2590E-135 (C-06/7 DA-24) are not key to the scenario.
Examiner I	Note: The foll	lowing steps are from AOP-2568.
		1. Check Pressurizer Level – DECREASING
	ATC	<ul> <li>a. ADJUST the bias on HIC-110, LTDN FLOW CNTL</li> <li>b. CHECK Pressurizer Level – DECREASING</li> </ul>
		2. Increase Charging Flow
	ATC	<ul> <li>a. START second Charging Pump</li> <li>b. STABILIZE Pressurizer Level by performing the following:</li> </ul>
		<ol> <li>On HIC-110, LTDN FLOW CNTL, ADJUST the bias to obtain Pressurizer level to Program level</li> </ol>
		a. CHECK Pressurizer Level - STABLE or INCREASING
	ATC	3. INITIATE Forcing Pressurizer Sprays
		4. Check Reactor Power and RCS Temperature
	ATC	a. CHECK Reactor Power – STABLE
		b. CHECK RCS temperature - STABLE or INCREASING
		NOTE:
I		

Op-Test No.:	ES16LI3	_ Scenario No.: <u>3</u> Event No.: <u>3</u>		
Event Descrip	otion: "A" RCI	P Seal Cooler Leak		
Time	Time Position Applicant's Actions or Behavior			

SRO/A BOP	Charging and Letdown flow
SRC	<ul> <li>6. CHECK RCS Leakage Within Limits Of T/S LCO 3.4.6.2, Reactor Coolant System Operational Leakage: <ul> <li><u>NO</u> Pressure Boundary Leakage</li> <li>1_gpm Unidentified Leakage</li> <li>10 gpm Identified Leakage</li> <li>75 gpd Primary to Secondary Leakage through any one steam generator</li> </ul> </li> </ul>
ATC	7. CHECK VCT Level - STABLE
SRO/A BOP	
SRO/A BOP	
SRC	NOTE: Steps 11 through 20 may be performed in any order.

Op-Test No.: <u>ES16LI3</u> Scenario No.: <u>3</u> Event No.: <u>3</u>				
Event Description: "A" RCP Seal Cooler Leak				
Time	Position	Applicant's	Actions or Behavio	or
Examiner Note: SRO should note TSAS 3.4.6.2 a.No PRESSURE BOUNDARY LEAKAGE ACTION:With ACTION and associated completion time of ACTION a. not met, or PRESSURE BOUNDARY LEAKAGE exists, or primary to secondary LEAKAGE not within limits, be in HOT STANDBY within 6 hours and be in COLD SHUTDOWN within 36 hours.         EAL Classification:         BARRIER FAILURE         BU2 RCS LEAKAGE (Barrier Unusual Event)         1. Pressure Boundary Leakage > 10 GPM         2. Unidentified Leakage > 10 GPM         10. USE Table 1 to determine the order for dealing with leak identification				
		and isolation. Table 1		
		Event	Step	Completed
	000	Steam Generator Tubes Intact	11	
	SRO	RCS Leakage In Auxiliary Building	12	
		Actions to Locate Leak- Containment	13	
		RBCCW System	14	
	SRO	NOTE: Any of the following are possible R • RCP Thermal Barr • Letdown HX • Primary Sample S	ier	s to RBCCW System:

Time Position Applicant's Actions or Behavior			
Event Description: "A" RCP Seal Cooler Leak			
Op-Test No.: <u>ES16LI3</u> Scenario No.: <u>3</u> Event No.: <u>3</u>			

-			
	BOP 14. Check NO RCS Leakage Into RBCCW System a. CHECK the following parameters: • RBCCW Surge Tank level – NORMAL • RM-6038, RBCCW System Radiation Monitor - NORMAL b. RETURN TO step 10		
		RNO Step 14 a. PERFORM the following:	
	SRO	<b>1. REQUEST</b> Chemistry Department to sample the RBCCW System using CP 2802G, Sampling Closed Cooling Water Systems.	
		<b>2. ATTEMPT</b> to isolate the leak using ATTACHMENT D, Potential Leakage Paths, while continuing with this procedure starting with step 15	
		3. PROCEED TO step 21.	
Attachmen	t D Table 10		
RCS to RBC	CW Leakage P	Paths	
Item Descri	iption	Equipment Numbers Isolated Initials	
A and C RC	A and C RCP Seal Coolers RB-30.1A RBCCW CTMT ISOL HDR A SUPPLY		
		RB-37.2A RBCCW CTMT ISOL HDR A RTN	
		t plant conditions does not allow isolation, requires plant trip and Securing A tion following actions should be carried out in EOP 2540	
	•	vledges request to sample the RBCCW for a primary leak. Wait appropriate CCW Sample results.	
	SRO *21 Check RCS Leakage Has Been Reduced To Within Tech Spec 3.4.6.2 Limits		
		RNO Step 21	
		PERFORM ONE of the following to place the plant in MODE 5:	
	SRO	<ul> <li>GO TO AOP 2575, Rapid Downpower</li> </ul>	
		OR	
		GO TO OP 2207, Plant Cooldown	

Op-Test No.: <u>ES16LI3</u> Scenario No.: <u>3</u> Event No.: <u>3</u>				
Event Descript	Event Description: "A" RCP Seal Cooler Leak			
Time	Time Position Applicant's Actions or Behavior			

**Examiner Note:** SRO should note TSAS 3.4.6.2

a.No PRESSURE BOUNDARY LEAKAGE ACTION:

With ACTION and associated completion time of ACTION a. not met, or PRESSURE BOUNDARY LEAKAGE exists, or primary to secondary LEAKAGE not within limits, be in HOT STANDBY within 6 hours and be in COLD SHUTDOWN within 36 hours.

Examiner Note: Based on the RCS Leak administrative guidance, the SRO should proceed to Event 4, Plant Shutdown.

Time Position Applicant's Actions or Behavior			
Event Description: Plant S	hutdown Due to RCS Leak		
Op-Test No.: <u>ES16LI3</u> Scenario No.: <u>3</u> Event No.: <u>4</u>			

Examiner Note: The <u>SRO</u> should initiate Event 4, Plant Shutdown Due to RCS Leak.

Indications:

- RBCCW Surge Tank level Rising
- RM-6038, RBCCW System Radiation Monitor Rising
- Letdown Flow Several gpm Less than Charging Flow

Examiner Note: The following steps are from AOP-2575, Rapid Downpower, Section 3.0 Rapid Downpower.

SRO	Enters AOP 2575, Rapid Downpower.
SRO	<ul> <li>3.1 PERFORM focus brief on the following: REACTOR TRIP CRITERIA</li> <li>Parameters associated with automatic reactor or turbine trips are challenged</li> <li>RCS T cold <i>not</i> within 10 °F of temperature program and efforts to regain control are unsuccessful</li> <li>RCS TEMPERATURE CONTROL</li> <li>RCS T cold to be maintained within +/- 5 °F of Attachment 5, "Temperature vs. Power program" using Attachment 10, "Main Turbine Load Set Control."</li> <li>To avoid uncontrolled cooldowns or power transients, sudden changes in RCS temperature or boron concentration should be avoided.</li> <li>3.2 REQUEST SM/STA to Refer To Attachment 8, "Required Notifications," and PERFORM notifications.</li> </ul>
ATC	3.3 INITIATE forcing pressurizer sprays.

Event Description: Plant Shutdown Due to RCS Leak

Time	Position	Applicant's Actions or Behavior

	SRO	CAUTION In the case of a dropped CEA, rod motion is <i>not</i> used to initiate downpower.
	ATC	3.4 IF <i>not</i> down powering due to a dropped rod, INSERT Group 7 CEAs 10 $\pm$ 2 steps to initiate downpower.
	BOP	3.5 Using the "Load Speed Control" switch, REDUCE turbine load to maintain Tc on program (+/-2 deg).
	SRO	3.6 Refer To PPC or Reactor Engineering Curve and Data Book and OBTAIN reactivity plan for the initial reactor power condition and desired load reduction.
Examiner N	Note: The c	rew should refer to Reactivity Plan for downpower parameters.
	SRO	NOTE Attachment 10 "Approximate Load Demand vs. Reactor Power," can be used to correlate the desired power level to a turbine load demand setpoint.
	BOP	3.7 Refer To Attachment 9, "Main Turbine Load Set Control," REDUCE turbine load and MAINTAIN Tc on program (+/-2 deg).
Examiner N Load Set C		Iowing steps are from AOP 2575 Rapid Downpower Attachment 9 Main Turbine

Time Position Applicant's Actions or Behavior		
Event Description: Plant S	Event Description: Plant Shutdown Due to RCS Leak	
Op-Test No.: <u>ES16LI3</u> Scenario No.: <u>3</u> Event No.: <u>4</u>		

CAUTION Operation of the "Load/Speed CONTROL" switch will change turbine load at 600%/hour, and cancel any previous load setpoint.
NOTE
Steps provided in this attachment are dependent on plant conditions and may be performed in any sequence, and repeated as necessary.
<ol> <li><u>IF</u> desired to commence or modify a turbine load ramp, PERFORM the following (HMI "Load" screen):         <ul> <li><u>IF</u> previous ramp has stopped, SELECT "Load Hold."</li> <li>SELECT "Load Setpt" and ENTER desired value.</li> <li>SELECT "Rate setpt" and ENTER desired value.</li> <li><u>WHEN</u> ready to commence load reduction, SELECT "Load Resume."</li> </ul> </li> </ol>
<ol> <li>IF desired to adjust the "Load Ramp Rate," PERFORM <i>any</i> of the following:         <ul> <li>SELECT "Rate setpt" and ENTER new value.</li> <li>SELECT "5% / hour," <u>OR</u> "10% / hour," <u>OR</u> "20% / hour."</li> <li>SELECT "Raise" or "Lower" (0.25% / hour change).</li> </ul> </li> <li>IF Tavg and Tc are <u>high</u> off program, PERFORM the following:         <ul> <li>SELECT "Load Hold" to stop ramp.</li> <li>WHEN Tavg and Tc are trending back to program, SELECT "Load Resume."</li> </ul> </li> <li>IF Tavg and Tc are low off program, PERFORM the following:         <ul> <li>JOG the "Load/Speed CONTROL" switch to "Lower."</li> <li>WHEN Tavg and Tc are back on program, SELECT Load Setpt" and ENTER desired value.</li> <li>IF desired load has been reached SELECT "Load Hold."</li> </ul> </li> <li>Examiner Note: operator should select x load setpoint, x load rate. Program band for Tavg and Tc is x (+/- 2 deg for Tc).</li> </ol>

Op-Test No.: <u>ES16LI3</u> Scenario No.: <u>3</u> Event No.: <u>4</u>			
Event Description: Plant Shutdown Due to RCS Leak			
Time Position Applicant's Actions or Behavior			

## Examiner Note: The following steps are from AOP 2575 Rapid Downpower Section 3.0 Rapid Downpower.

		Examiner Note: Crew should borate from the RWST.
	ATC	g. Go To step 3.11
		f. CHECK charging flow at desired rate.
		e. CLOSE CH-501, VCT outlet isolation.
		d. OPEN CH-192, RWST isolation.
		c. ENSURE CH-504, RWST to charging suction, open.
		b. ENSURE CH-196, VCT makeup bypass, closed.
		a. ENSURE at least one charging pump operating.
		PERFORM the following:
		3.9 IF desired to borate from the RWST (preferred method)
	ATC	3.8 Based on required rate of downpower, START additional charging pumps as necessary and balance charging and letdown.

Op-Test No.: <u>ES16LI3</u> Scenario No.: <u>3</u> Event No.: <u>4</u>		
Event Description: Plant Shutdown Due to RCS Leak		
Time Position Applicant's Actions or Behavior		

	Parameters," and MAINTAIN parameters as specified throughout downpower Examiner note: Attachment 1 Rapid Downpower Parameters:
SRO/ATC/ BOP	<ul> <li>Condensate and heater drain flows and pressures: sufficient to maintain adequate SGFP suction pressure</li> <li>FRV D/P: greater than 40 psid</li> <li>Turbine load: responding to changes in load demand, with control valves operating together</li> <li>Steam generator levels 55 to 70%.</li> <li>MSR parameters tracking together</li> <li>Turbine Generator MVARs: as specified by CONVEX</li> <li>Reactor power: being monitored using delta T power indication</li> <li>ASI: In accordance with reactivity plan or within 0.01 of ESI or per COLF</li> <li>CEA position: greater than PDIL</li> <li>Tc: less than or equal to 549 deg</li> <li>Pressurizer level: between 35 and 70%</li> </ul>
	Pressurizer pressure: between 2,225 and 2,300 psia (DNB margin)
SRO/ATC	NOTE 1. Xenon rate of change should be considered when terminating boration. 2. During rapid downpower, the PPC calorimetric may be inaccurate due to SG level transients. The most accurate available indication of reactor power is RPS delta T power.

Examiner Note: Once power has dropped at least 5%, or at the lead examiner's direction, proceed to Event 5, "A" RCP Seal Cooler Rupture, Manual Plant Trip.

Op-Test No.: <u>ES16LI3</u> Scenario No.: <u>3</u> Event No.: <u>5</u>				
Event Descrip	Event Description: "A" RCP Seal Cooler Rupture of 550 gpm (Inter-System SB-LOCA)			
Time	Time Position Applicant's Actions or Behavior			

Simulator Operator: When directed, initiate Event 5, "A" RCP Seal Cooler Rupture (550 gpm) Resulting in an Inter-System SB-LOCA of 550 gpm by modifying RC20A to 550 gpm (or trigger E-5).

Verify Event 6 triggers 30 seconds post-trip, ESD Outside CTMT, Upstream of #2 MSIV

Verify Event 7 triggers five minutes post-trip; "A" AFW pump degraded performance and "B" Motor Driven AFW pump trip. Standby to trigger Event 8 if necessary.

Indications:

- RCS Pressure Dropping
- Pressurizer Level Dropping

Examiner Note: The following steps are from EOP 2525, Standard Post Trip Actions, modified slightly to improve clarity.

	<ul> <li>Determine Status of Reactivity Control – Reactor Trip</li> <li>1. DETERMINE that Reactivity Control acceptance criteria are met for the reactor by performing ALL of the following steps:</li> </ul>
ATC	<ul> <li>CHECK that all CEAs are fully inserted.</li> <li>CHECK that reactor power is dropping.</li> <li>CHECK that SUR is negative.</li> </ul>
BOP	<ul> <li>Determine Status of Reactivity Control – Turbine Trip</li> <li>2. DETERMINE that Reactivity Control acceptance criteria are met for the turbine by performing ALL of the following steps: <ul> <li>a. CHECK that the main turbine is tripped by BOTH of the following:</li> <li>ALL main stop valves are closed.</li> <li>Generator megawatts indicate zero.</li> <li>Turbine speed is lowering.</li> </ul> </li> <li>b. <u>IF</u> 15G-2XI-4, motor operated disconnect, is closed, CHECK that the main Generator output breakers 8T and 9T are open.</li> </ul>

Op-Test No.: <u>ES16LI3</u> Scenario No.: <u>3</u> Event No.: <u>5</u>			
Event Descri	Event Description: "A" RCP Seal Cooler Rupture of 550 gpm (Inter-System SB-LOCA)		
Time	Position	Applicant's Actions or Behavior	

		Determine Status of Maintenance of Vital Auxiliaries
		3. DETERMINE that Maintenance of Vital Auxiliaries acceptance criteria are met by performing ALL of the following steps:
		a. CHECK that ALL Facility 1 and 2 electrical buses are energized:
		6.9kV Electrical Buses 25A, 25B
	BOP	<ul> <li>4.16kV Non-Vital Electrical Buses 24A, 24B</li> <li>4.16vV Vital Electrical Buses 24C, 24D</li> </ul>
		<ul> <li>Vital DC Buses 201A, 201B, DV-10, DV-20</li> </ul>
		<ul> <li>Vital AC Instrument Buses VA-10, VA-20</li> </ul>
		<ul> <li>b. CHECK that BOTH facilities of service water are operating.</li> <li>c. CHECK that BOTH facilities of RBCCW are operating with service water cooling.</li> </ul>
		Determine Status of RCS Inventory Control
	470	<ol> <li>DETERMINE that RCS Inventory Control acceptance criteria are met by performing ALL of the following:</li> </ol>
	ATC	a. CHECK that <b>BOTH</b> of the following conditions exist:
		<ul> <li>Pressurizer level is 20 to 80%</li> <li>Pressurizer level is trending to 35 to 70%</li> </ul>
		b. CHECK that RCS subcooling is greater than or equal to 30 °F
		RNO
		a.1 <u>IF</u> the Pressurizer Level Control System is not operating properly in automatic, RESTORE and MAINTAIN pressurizer level 35 to 70% by performing ANY of the following:
	ATC	1) OPERATE the Pressurizer Level Control System.
		2) Manually OPERATE charging and letdown.
		(Starts all avialable Charging pumps and isolates letdown when PZR level <20% ref:OP2260)
	ATC	Determine Status of RCS Pressure Control
		<ol> <li>DETERMINE RCS Pressure Control acceptance criteria are met by BOTH of the following:</li> </ol>
		CHECK that pressurizer pressure is 1900 to 2350 psia.
		CHECK that pressurizer pressure is trending to 2225 to 2300 psia.

Event Desch	Event Description: "A" RCP Seal Cooler Rupture of 550 gpm (Inter-System SB-LOCA)				
Time Position Applicant's Actions or Behavior					

-		
	ATC	<ul> <li><u>RNO</u></li> <li>5.1 <u>IF</u> the Pressurizer Pressure Control System is not operating properly in automatic, THEN RESTORE and MAINTAIN between 2225 to 2300 psia by performing ANY of the following:</li> <li>a. OPERATE the Pressurizer Pressure Control System.</li> <li>b. Manually OPERATE pressurizer heaters and spray valves.</li> <li>(NOTE PZR Heaters will trip PZR level &lt;20%)</li> <li>5.2 PZR Spray valves (Verifies Closed)</li> <li>5.3 PORVs (Verifies Closed)</li> <li>5.4 RCS Pressure &lt;1750 psia SIAS CIAS EBFAS on C01 annuciators.</li> <li>5.5 &lt;1714 psia w/SIAS Secure ONE RCP in each loop</li> <li>(at 1800 psia manually initiates SIAS trips 2 RCP preferably "A" and "C" due to "A" RCP seal Leak)</li> <li>5.6 TCOA: RCS pressure &lt; NPSH SECURE ALL RCPs</li> </ul>
		13 Trip two RCPs with SIAS actuation and a LOCA in progress. Ps were tripped
(NOTE: RC Time RCS	S conditions pressure drop	12/TCOA: RCP < NPSH Curve - 5 minutes to STOP ALL RCPs may not go below RCP NPSH due to the Crews actions) oped below NPSH required pressure I due to loss of NPSH pressure
	ATC	<ul> <li>Determine Status of Core Heat Removal</li> <li>6. DETERMINE that Core Heat Removal acceptance criteria are met by performing ALL of the following: <ul> <li>a. CHECK that at least one RCP is operating and that loop delta T is less than 10 °F</li> <li>b. CHECK that Th subcooling is greater than or equal to 30 °F.</li> </ul> </li> </ul>
	ATC	<ul> <li>RNO</li> <li>a.1 IF <u>RCPs are not operating</u>, OR loop ∆T is greater than 10° F, THEN PERFORM the following: <ol> <li>PLACE TIC- 4165, steam dump TAVG controller, in manual and closed.</li> <li>PLACE BOTH pressurizer spray valve controllers in manual and CLOSE the valves.</li> <li>HIC- 100E</li> <li>HIC- 100F</li> </ol> </li> </ul>

Op-Test No.: <u>ES16LI3</u> Scenario No.: <u>3</u> Event No.: <u>6&amp;7</u>						
Event Description: ESD Outside CTMT, Upstream of #2 MSIV and Loss of AFW Flow						
Time	Position	Applicant's Actions or Behavior				
Simulator Operator:						
Verify Event 6 triggers 30 seconds post-trip, ESD Outside CTMT, Upstream of #2 MSIV						
Verify Event 7 triggers five minutes post-trip; "A" AFW pump degraded performance and "B" Motor Driven AFW pump trip. Standby to trigger Event 8 if necessary.						
Examiner Note: Once Event 6 ESDE is initiated the BOP will re-perform Step 7 of EOP 2525 if the OPERATOR has already completed the Step.						
		Determine Status of RCS Heat Removal				
		7. DETERMINE that RCS Heat Removal acceptance criteria are met by ALL of the following conditions:				
	BOP	<ul> <li>a. CHECK that at least one steam generator has <b>BOTH</b> of the following conditions met:</li> <li>Level is 10 to 80%.</li> </ul>				
		Main feedwater or TWO auxiliary feedwater pumps are operating to restore level 40 to 70%.				
		RNO				
		<ul> <li>a.1 RESTORE level to between 40% to 70% in at least ONE steam generator using ANY of the following:</li> <li>Main feedwater</li> </ul>				
	BOP	<ul> <li>Motor- driven auxiliary feedwater pump</li> <li>TDAFW Pump. Refer To Appendix 6, "TDAFW Pump Normal Startup."</li> <li>TDAFW Pump. Refer To Appendix 7, "TDAFW Pump Abnormal Startup."</li> </ul>				
		( <b>EVENT 7</b> Operator notes that "A" AFW Pump has degraded performance and the "B" AFW Pump has tripped, refers to Appendix 7 to start the TDAFW pump)				
Simulator Operator: If "A" AFW pump malfunction is not enough to cause unacceptable loss of feed flow such that the TDAFW pump is not needed, trigger Event 8, "A" AFW Pump trip.						
<u>CRITICAL TASK</u> : LOCA-2; Start the TDAFP (Ensure RCS Heat Sink is maintained). OP 2260 (TCOA); SG level lowering and no feedwater flow from MDAFW pumps, start TDAFW pump within 10 minutes of loss of feedwater flow. Time AFW Flow is lost						
Time TDAFW pump started						

		Scenario No.: <u>3</u> Event No.: <u>6&amp;7</u>
Event Descri	ption: ESD O	utside CTMT, Upstream of #2 MSIV and Loss of AFW Flow
Time	Position	Applicant's Actions or Behavior
	BOP	b. CHECK that RCS Tc is being maintained between 530 °F to 535"F.
	BOP	<b>RNO</b> <b>b.2</b> IF RCS TC is less than 530 °F, THEN CONFIRM steam generator steam and feed rates are NOT excessive:
	BOP	c. CHECK that <b>BOTH</b> steam generators pressure are 880 to 920 psia.
CRITICAL	TASK: Main S	team Line Break (SFRM 2.8.2.8.2) INITIATING EVENT:
Excess ste	eam demand	event resulting in a Main Steam Isolation Signal (MSIS)
•	• •	e AFW flow to the affected S/G from control room or local Within 30 minutes of
	am Isolation Steem lealet	
		ion Signal (MSIS): 2 2525 RNO Step 7c2:
		2323 HNO Step 7c2
		RNO (CRITICAL TASK START TIME)
	BOP	<ul> <li>RNO (CRITICAL TASK START TIME)</li> <li>c.1 IF ANY SG pressure is less than 572 psia, THEN ENSURE MSI actuated.</li> <li>(C01)</li> </ul>

Op-Test No.: _	ES16LI3	Scenario No.: _	3	Event No.:	6&7	

Event Description: ESD Outside CTMT, Upstream of #2 MSIV and Loss of AFW Flow

Time Position Applicant's Actions or Behavior
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		RNO				
		<b>c.3</b> IF ANY steam generator pressure is less than 572 psia AND an excess steam demand event is in progress, THEN PERFORM the following:				
	BOP	1) CLOSE the ADV for the most affected steam generator.				
		2) IF the most affected steam generator has boiled dry, as indicated by CET temperature rising, THEN OPERATE the ADV for the least affected steam generator to stabilize CET temperature.				
		3) Proceed To Step 8				
		<b>c.4</b> IF ANY steam generator pressure is less than 800 psia AND lowering, THEN PERFORM the following:				
	BOP	1) CLOSE BOTH MSIVs.				
		2) ENSURE BOTH MSIV bypass valves are closed.				
		3) NOT APPLICABLE FOR THIS SCENARIO				
		c.5 IF ANY steam generator pressure is less than 880 psia, THEN PERFORM the following:				
	BOP	1) NOT APPLICABLE FOR THIS SCENARIO				
		2) NOT APPLICABLE FOR THIS SCENARIO				
		3) CHECK main steam safety valves are closed.				
		c.6 NOT APPLICABLE FOR THIS SCENARIO				

Op-Test No.: <u>ES</u>	S16LI3 Scenario No.: <u>3</u> Event No.: <u>6&amp;7</u>
Event Description:	ESD Outside CTMT, Upstream of #2 MSIV and Loss of AFW Flow

Time

Position

Applicant's Actions or Behavior

	Containment Isolation
	8. ENSURE Containment Isolation met by ALL of the following:
	a. CHECK Containment pressure is less than 1.0 psig.
	b. CHECK NONE of the following primary plant radiation monitors have an unexplained alarm or indicate an unexplained rise in activity:
ATC	Radiation Monitors Inside Containment RM- 7890, Personnel Access Area RM- 7891, Ctmt Refuel Floor Area RM- 8240, High Range RM- 8241, High Range RM- 8123 A and B, Ctmt Atmosphere RM- 8262 A and B, Ctmt Atmosphere C. CHECK NONE of the following primary plant radiation monitors have an unexplained alarm or indicate an unexplained rise in activity: Steam Plant Padiation Monitore
	Steam Plant Radiation Monitors • RM- 5099, Steam Jet Air Ejector • RM- 4262, SG Blowdown • RM- 4299A and B, Main Steam Line 1 • RM- 4299C, Main Steam Line 2
	Containment Temperature and Pressure Control
ATC	9. ENSURE Containment Temperature and Pressure Control met by BOTH of the following conditions:
	<ul> <li>a. CHECK Containment temperature is less than 120°F. (PPC or avg of Points 5 and 6)</li> <li>b. CHECK Containment pressure is less than 1.0 psig</li> </ul>
	Event Diagnosis
	10. PERFORM the following:
SRO	a. DIAGNOSE the event. Refer To Appendix 1, "Diagnostic Flowchart."
	b. INITIATE Appendix 4, Attachment 4A "Reactor Trip Subsequent Actions."
	c. Go To the appropriate EOP.
Examiner Note: The Uni	t Supervisor refers to EOP 2541 Appendix 1, Diagnostic Flowchart to

diagnose the event.

Op-Test No	.: <u>ES16LI3</u>	Scenario No.: <u>3</u> Event No.: <u>6&amp;7</u>				
Event Desci	ription: ESD O	utside CTMT, Upstream of #2 MSIV and Loss of AFW Flow				
Time	Position	Applicant's Actions or Behavior				
		{Step 10.b above} Perform Appendix 4, Attachment 4A "Reactor Trip Subsequent Actions".				
	ATC/BOP	Examiner Note: EOP Appendix 4, Attachment 4A "Reactor Trip Subsequent Actions." are attached to guide.				
Examiner	Note: The fol	lowing steps are from EOP 2540, Functional Recovery.				
Indication	• #2 S/G Pre	Leg Temperature				
	000	1. CLASSIFY the event. Refer To MP-26-EPI-FAP06, "Classification and PARs"				
	SRO	<ul> <li><u>IF</u> classification requires RCS sampling, Refer To Appendix 46, "Sampling for EAL Determination" and DIRECT Chemistry as required.</li> </ul>				
Examiner N <u>RCS BARI</u>		CTMT BARRIER .				
RCB2	LOSS cooling < 30°F	CNB3				
Reactor C	POTENTIAL LOSS oolant Leak > CVCS Cay , Standard Post Trip Actio	Deacity AND Entry Into EOP-2532, Loss of Primary Coolant, AND Leakage Exists Outside CTMT Requiring Isolation From				
		2. PERFORM ALL of the following:				
	SRO	<ul> <li>OPEN the Safety Function Tracking Page and ENTER the EOP entry time.</li> <li>ENSURE the master alarm silence switch is in "NORMAL".</li> </ul>				
	ATC	<ul> <li>3. <u>IF</u> pressurizer pressure is less than 1714 psia <u>AND</u> SIAS has initiated, PERFORM the following: <ul> <li>a. ENSURE <b>ONE</b> RCP in each loop is stopped.</li> <li>b. PLACE associated pressurizer spray valve controller RC-100E or RC-100F in manual and CLOSE the valve.</li> <li>c. <u>IF</u> pressurizer pressure lowers to less than the minimum RCP NPSH limit, PERFORM the following: <ul> <li>1) STOP <b>ALL</b> RCPs.</li> </ul> </li> </ul></li></ul>				

Op-Test No.	: <u>ES16LI3</u>	Scenario No.: <u>3</u> Event No.: <u>6&amp;7</u>
Event Descr	iption: ESD O	utside CTMT, Upstream of #2 MSIV and Loss of AFW Flow
Time	Position	Applicant's Actions or Behavior
	1	

BOP	<ul> <li>4. SAMPLE steam generators that are available for RCS heat removal as follows: <ul> <li>a. CHECK "B" train RBCCW in service.</li> <li>b. ENSURE 2-RB-210 "Degasifier Effluent Cooler Return Outlet" is open.</li> <li>c. OPEN appropriate steam generator sample valves: <ul> <li>MS-191A</li> <li>MS-191B</li> </ul> </li> <li>d. DIRECT Chemistry to perform ALL of the following: <ul> <li>Sample ANY steam generator that is available for RCS Heat Removal</li> <li>Frisk the samples</li> <li>Report frisk results</li> <li>Analyze samples for boron and activity</li> </ul> </li> <li>e. WHEN Chemistry reports that samples have been taken, PERFORM the following: <ul> <li>CLOSE the steam generator sample valves</li> <li>IF SIAS has actuated, AND no other sampling is in progress, CLOSE 2-RB-210, "Degasifier Effluent Cooler Return Outlet"</li> </ul> </li> </ul></li></ul>
	<ul> <li>Sample Steam Generators, respond 20 minutes later that samples have been results show all background levels.</li> <li>5. PLACE the hydrogen analyzers in service. Refer To Appendix 19, "Hydrogen Analyzer Operation."</li> </ul>
SRO	<b>NOTE</b> If the Safety Function Status Checklist is <i>not</i> satisfied for the selected success path, the US may commence the operator actions for safety functions which are <i>not</i> met based on Safety Function hierarchy. The remaining Safety Functions should be prioritized as time permits.
SRO	<ul> <li>6. IDENTIFY success paths to be used to satisfy each safety function using BOTH of the following:</li> <li>Resource Assessment Trees</li> <li>Safety Function Tracking Page</li> </ul>

Op-Test No.: <u>E</u>	16LI3 Scenario No.: <u>3</u> Event No.: <u>6&amp;7</u>
Event Description:	ESD Outside CTMT, Upstream of #2 MSIV and Loss of AFW Flow

Time

Position

Applicant's Actions or Behavior

	following: a. Safety for the se b. Safety <i>not</i> opera c. Safety d. Safety e. Safety e. Safety se Board Operators	functions wh lected succe functions for ting. functions for functions for functions for through the re to use.	which the equipr which success p which success p which success p	he Safety Fund nent to suppor ath three has to ath two has be ath one has be sment Trees ar	ction Statu t the succ been select een select een select	us Checklist ess path is cted. ed. ed.
	SAFETY FUNCT					
	Safety Function		cess Path	Procedure	SFSC Met	Priority
		RC-1	CEA Insertion		Y	5
	Reactivity Control	RC-2	Boration CVCS	EOP 2540A		
	Control	RC-3	Boration SI			
SRO	Maintenance of Vital DC Power	MVA-DC-1	Battery Chargers/ Station Batteries	EOP 2540B	Y	6
		MVA-AC-1	RSST	EOP 2540B	Y	7
	Maintenance of Vital AC Power	MVA-AC-2	EDG			
		MVA-AC-3	BUS 34A/34B			
	RCS Inventory	IC-1	CVCS	EOP 2540C1 EOP 2540C2		2 NOTE MAYBE Sub or Sat depending on crew actions 7,3
	Control	IC-2	Safety Injection		Y	
		PC-1	Subcooled		Y	
	RCS Pressure Control RCS Core Heat Removal	PC-2	Saturated		Y	
		PC-3	PORVs			
		HR-1	SI no operating	EOP 2540D		
		HR-2	SI operating		Y	<mark>3</mark> ,4
SRO		HR-3	O-T-C			
	Containment Isolation	CI-1	Automatic/Manual	EOP 2540E	N	1
	Containment Temperature and Pressure Control	CTPC-1	CARs (Normal)	EOP 2540F	Y	8
		CTPC-1	CARs (Emerg)			
		CTPC-1	CTMT Spray			

Op-Test No.: <u>ES16LI3</u> Scenario No.: <u>3</u> Event No.: <u>6&amp;7</u>
Event Description: ESD Outside CTMT, Upstream of #2 MSIV and Loss of AFW Flow

Time Positior	Applicant's Actions or Behavior
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	SRO	8. DIRECT the STA to check that Safety Function Status Checklist Criteria are satisfied for chosen success paths.
		9. PERFORM operator actions for chosen success paths based on priority assigned.
		TRANSITION to EOP 2540E Functional Recovery Containment Isolation
Examiner N	lote: The fol	lowing steps are from EOP 2540E, Functional Recovery Containment Isolation.
		<ul> <li>Check SIAS/CIAS Actuation</li> <li>* 1. IF ANY of the following conditions exist: <ul> <li>Containment pressure is greater than or equal to 4.42 psig</li> <li>Radiation monitors inside containment are greater than their alarm setpoint</li> <li>An unexplained rise in containment radiation level or activity</li> </ul> </li> <li>NOT APPLICABLE FOR THIS SCENARIO previously checked</li> </ul>
		Identify and Isolate SGTR * 2. IF a SGTR is indicated by ANY of the following: NOT APPLICABLE FOR THIS SCENARIO no SGTR
		<ul> <li>Isolate RCS to RBCCW Leakage</li> <li>* 3. CHECK no leakage in the RBCCW system by BOTH of the following:</li> <li>CHECK RM-6038, RBCCW Radiation Monitor, is not alarming or trending to alarm.</li> <li>CHECK that the RBCCW surge tank level is not rising.</li> </ul>
		<ul> <li>RNO</li> <li>3.1 IF ANY RCPs are operating, PERFORM the following: <ul> <li>a. STOP the operating RCPs.</li> <li>b. PLACE the associated pressurizer spray valve controller, RC-100E or RC-100F, in manual and CLOSE the valve.</li> <li>c. IF ALL RCPs are stopped, PLACE TIC-4165, steam dump TAVG controller, in manual and closed.</li> </ul> </li> </ul>

# Examiner Note: Completed in previously step for SIAS actuation

Op-Test No.:	ES16LI3	Scenario No.: <u>3</u> Event No.: <u>6&amp;7</u>		
Event Description: ESD Outside CTMT, Upstream of #2 MSIV and Loss of AFW Flow				
Time	Position	Applicant's Actions or Behavior		
		<b>RNO</b> <b>3.2</b> CLOSE ALL of the following RBCCW CTMT header isolation valves:		
		Facility 1 • RB-30.1A • RB-37.2A		
		Facility 2 <ul> <li>RB-30.1B</li> <li>RB-37.2B</li> </ul>		
Examiner N	lote: Closing	Facility 1 isolation satisfies Safety Function for CTMT Isolation		
		4. CHECK that CI-1 (Automatic/Manual Isolation) is satisfied by ALL of Condition 1 or ALL of Condition 2 is met:		
		Condition 2		
		<ul> <li>Each containment penetration required to be closed for current plant conditions has an isolation valve closed</li> <li>ONE of the following:         <ul> <li>No steam plant radiation monitors have an unexplained alarm or unexplained rises in activity</li> <li>ALL release paths from the most affected SG to the environment isolated unless a planned release is in progress</li> </ul> </li> </ul>		
<b>Evening</b> •		IF SGTR is present, steam generator pressure is less than 920 psia		
		on 2 is Satisfied SRO should transition to the next priority Safety Function for C-2Safety Injection).		
Examiner N	Note: The foll	owing steps are from EOP 2540C1 IC-2, Functional Recovery Safety Injection		
		2.0 SUCCESS PATH: IC-2: SAFETY INJECTION		

	Ensure SIAS Initiated
ATC	* 1. PERFORM ALL of the following:
	Completed in EOP 2525 or 2540

Op-Test No.:	ES16LI3	Scenario No.: <u>3</u> Event No.: <u>6&amp;7</u>
Event Descri	iption: ESD O	utside CTMT, Upstream of #2 MSIV and Loss of AFW Flow
Time	Position	Applicant's Actions or Behavior
		Optimize Safety Injection
		* 2. PERFORM the following to optimize safety injection flow:
	ATC	a. CHECK at least one train of SIAS, CIAS and EBFAS has properly actuated. (C01X)
		<ul> <li>b. CHECK that safety injection flow is adequate. Refer To Appendix 2, "Figures."</li> </ul>
		c. ENSURE ALL available charging pumps are operating.
		d. ENSURE vital switchgear cooling is operating for each operating ECCS train as follows:
		(Completed previously)
•		C and Transition to RCS Core Heat Removal to isolate the #2 S/G EOP 2540D owing steps are from EOP 2540D HR-2, Functional Recovery Heat Removal
	SRO	2.0 SUCCESS PATH: HR- 2: SG HEAT SINK WITH SI OPERATING
		Pulls Forward Step 14
	SRO	<ul> <li>Determine Presence of ESDE</li> <li>* 14. DETERMINE if an ESDE is in progress by considering ALL of the following:</li> <li>Steam generator pressures</li> <li>Steam generator levels</li> <li>RCS cold leg temperatures</li> <li>(Determined #2 S/G)</li> </ul>
		NOTE If there is a conflict between isolating a SG and maintaining adequate heat removal, at least one SG should be maintained for heat removal whenever possible.
	SRO	Perform ESDE Response * 15. IF indications of an ESDE exist, PERFORM ESDE actions. Refer To Appendix 11, "ESDE Response." (SRO Directs the BOP to perform Appendix 11 isolate the #2 S/G)
		EOP 2541, Appendix 11 ESDE Response
		<ol> <li>PERFORM the following to isolate the leak:         <ul> <li>a. ENSURE MSI has actuated. (C01)</li> <li>b. CHECK at least one train of MSI has properly actuated. (C01X)</li> </ul> </li> </ol>

c. OPEN AR-17, condenser vacuum breaker.

Op-Test No.: <u>ES16LI3</u> Scenario No.: <u>3</u> Event No.: <u>6&amp;7</u>
Event Description: ESD Outside CTMT, Upstream of #2 MSIV and Loss of AFW Flow

Time	Position	Applicant's Actions or Behavior
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<ul> <li>2. DETERMINE the most affected steam generator by considering ALL of the following: <ul> <li>High steam flow from steam generator</li> <li>Lowering steam generator pressures</li> <li>Lowering steam generator levels</li> <li>Lowering RCS cold leg temperatures</li> </ul> </li> </ul>
<ul> <li>*3. IF the leak has <i>not</i> been isolated, ISOLATE the most affected steam generator by performing the following:</li> <li>Number 2 Steam Generator         <ul> <li>a. ENSURE MS-64B, MSIV, is closed.</li> </ul> </li> </ul>
b. ENSURE MS-65B, MSIV bypass valve, is closed.
c. ENSURE ALL of the following for the associated ADV:
PIC-4224, ADV controller, is in manual.
ADV is closed.
d. PLACE ADV Quick Open Permissive switch to "OFF".
e. CLOSE LIC-5216, main feedwater regulating bypass valve.
f. ENSURE FW-42B, main feedwater block valve, is closed.
g. PLACE FW-5B, main feed isolation air assisted check valve, to
"CLOSE".
h. UNLOCK and CLOSE "DISC FOR 2-MS-202" (NS6202).
i. CLOSE MS-202, steam to turbine driven aux feed pump supply valve.
j. ENSURE MS-220B, steam generator blowdown isolation valve, is
closed. k.PLACE BOTH auxiliary feed "OVERRIDE/MAN/START/ RESET"
handswitches in "PULL-TO-LOCK".
I. CLOSE FW-43B, aux feedwater regulating valve.
m. PLACE FW-12B, aux feed isolation air assisted check valve, to
"CLOSE".
n. CLOSE MS-266B, main steam low point drain.
o. CHECK main steam safety valves are closed.

# JOB PERFORMANCE MEASURE APPROVAL SHEET

		Desision 0/1
PM Number:	JPM-292-R-RO	Revision:0/1
nitiated:		
	David J. Jacobs	05/18/2016
	Developer	Date
Reviewed:		
	Robert L. Cimmino, Jr.	07/05/2016
	Technical Reviewer	Date
Approved:		
	Supervisor, Nuclear Training	Date

# **SUMMARY OF CHANGES**

DATE	DESCRIPTION	REV/CHANGE	
05/18/2016	New JPM for I LT	0/0	
08/18/2016	Incorporated NRC comments	0/1	

#### JPM WORKSHEET

Facility: MP-2	Examinee:		
JPM Number:	JPM-292-R-RO	Revision: 0/1	
Task Title: Shut	down Safety Assessment Decay	Heat Removal	
System: Cond	luct of Operations		
Time Critical Task:	YES NO		
Validated Time (minu	tes): <u>60</u>		
Task Number(s):	119-01-044	_	
Applicable To:	SRO X STA	RO <u>X</u> P	PEO
K/A Number:	2.1.18 K/A Rating:	3.6/3.8	
Method of Testing:	Simulated Performance:	Actual Pe	erformance: <u>X</u>
Location:	Classroom: X	Simulator:	In-Plant:
Task Standards:	At the completion of this JPM code for the predicted Decay I Inventory.		
Required Materials: (procedures, equipment, etc	MP-PROC-000-OU-M2-201[	r018.00] Shutdown Safety A	Assessment Checklist
General References:	MP-PROC-000-OU-M2-201[	r018.00] Shutdown Safety A	Assessment Checklist

## \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

## JPM WORKSHEET

	JPM Number:	JPM-292-R-RO	Revision :	0/1
Initial Conditio	React	lant is in MODE 5 day 2 of a sche or disassembly is in progress and t yed within the next 12 hours. ollowing additional conditions pre	the Reactor Head is ex	
	•	"A" Train is protected. "B" EDG tagged for Maintenan Back Feeding from the NSST No Off-Site GRID Risk Penalty RCS Boron concentration is 220 RCS temperature is 105°F. PZR level is 20% PZR Vent Port Removed	re Factors	
Initiating Cues:	di	<ul> <li>he US has given you the current S brected you perform a <u>predicted</u> SS for the following Sections only:</li> <li>Section 3 Decay Heat Remo</li> <li>Section 7 Power Availabilit</li> </ul>	SA for the RCS in Red	

Simulator Requirements: N/A

# \* \* \* \* <u>NOTES TO TASK PERFORMANCE EVALUATOR</u> \* \* \* \*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

	JPM Number: JPM-292-R-RO		Revision:	0/1	
	Task Title: Shutdown Safety Asses	ssment Decay Heat Removal			
				START TIME: _	
STEP #1	Performance: Refers to MP-PROC-000-OU-M2-201 Shutdown Safety Assessment Checklist And Current Condition SSA Cue: Comments:	Standard: Examinee reviews Current Condition	SSA	Critical: Y 🗌 N 🔀	Grade S 🗌 U 🗌
STEP #2	Performance:         3.1 Key Safety Functions         3.1.1 ASSESS and MANAGE the following         KSFs for risk during shutdown conditions:         • Decay Heat Removal (DHR)         • Power Availability         Cue:         Comments:	Standard: Examinee Assess the following Key S Functions: • Decay Heat Removal • Power Availability	afety	Critical: Y 🗌 N 🔀	Grade S 🗌 U 🗌

JPM Number: JPM-292-R-RO

Revision: 0/1

## Task Title: Shutdown Safety Assessment Decay Heat Removal

STEP #3	<ul> <li>Performance:</li> <li><b>3.3</b> Shutdown Safety Assessment (SSA) Checklist Preparation</li> <li><b>3.3.1</b> Using the following detailed information for each KSF, COMPLETE Attachment 1:</li> <li><b>f.</b> Section 3 - Decay Heat Removal (DHR)</li> <li><b>1</b>. RCS Decay Heat Removal (DHR)</li> <li>• REFER to Attachment 4 for background information of each element associated with the Decay Heat Removal KSF.</li> <li>• CHECK appropriate boxes for conditions supporting "Key Safety Function" of RCS decay heat removal.</li> <li>• TOTAL score and ENTER value in RCS DHR Total box.</li> <li>• CIRCLE Condition color corresponding to point total.</li> </ul>	<ul> <li>Standard:</li> <li>Examinee refers to the following: <ul> <li>ATTACHMENT 4 Decay Heat Removal Requirements and performs Attachment 1Section 3 Decay Heat Removal</li> <li>Subtracts 1 point for Reduced Inventory Operations</li> <li>Change Total Condition to 1 points</li> <li>Circles ORANGE</li> </ul> </li> </ul>	Critical: Y ⊠ N □	Grade S 🗌 U 🔲
	Cue:			
	Comments:			

JPM Number: JPM-292-R-RO

Revision: 0/1

## Task Title: Shutdown Safety Assessment Decay Heat Removal

STEP	Performance:	Standard:	Critical:	Grade
#4	<ul> <li>3.3 Shutdown Safety Assessment (SSA) Checklist Preparation</li> <li>3.3.1 Using the following detailed information for each KSF, COMPLETE Attachment 1:</li> <li>j. Section 7 - Power Availability</li> <li>1. REFER to Attachment 8 for background information for each element associated with Power Availability KSF.</li> <li>2. CHECK appropriate boxes for conditions supporting "Key Safety Function" of Power Availability.</li> <li>3. IF required, THEN RECORD applicable Off- Site GRID Risk Penalty Factor and SUBTRACT from Power Availability subtotal to determine Power Availability Total.</li> <li>4. TOTAL score and ENTER the value in Power Availability Total box.</li> <li>5. CIRCLE Condition color corresponding to point total.</li> </ul>	<ul> <li>Examinee refers to the following:</li> <li>1. ATTACHMENT 8 background information Power Availability Requirements and performs</li> <li>2. Attachment 1Section 7 Power Availability <ul> <li>Reviews Required Equipment and does not meet 2 EDG available = RED</li> </ul> </li> <li>3. No GRID Penalty <ul> <li>Total Score remains 4</li> </ul> </li> <li>5. Circle Condition RED for not meeting Minimum Equipment</li> </ul>	Y 🛛 N 🗖	S 🗌 U 🗌
	Cue:			
	Comments:			

Revision: 0/1

# Task Title: Shutdown Safety Assessment Decay Heat Removal

S T E P	Performance:	Standard:	Critical:	Grade
# 5	<b>3.4.6</b> DOCUMENT notification to OMOC and Maintenance Rule Coordinator for any unplanned RED or ORANGE conditions. a. <b>IF</b> OCC is staffed, <b>THEN NOTIFY</b> the SOM and OOM to ensure OMOC notifications are made.	Examinee Reports going to Reduce Inventory will cause a RED Condition for Power Availability	Y 🗌 N 🖾	S 🗌 U 🗌
	Cue: Comments:			

TERMINATION CUE: The evaluation for this JPM is concluded.

STOP TIME: \_\_\_\_\_

## **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-292-R-RO

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🖾 No	)		
Validated Time (minutes):	30	Actual Time to Complete (minutes):			
Work Practice Performance:		SAT	UNSAT		
Operator Fundamentals:		SAT	UNSAT		
JPM Question Portion Overall [NLO only]:		SAT	UNSAT		N/A
Attache	d Question #1	SAT	UNSAT		
Attached Question #2		SAT	UNSAT		
Overall Result of JPM:		SAT	UNSAT		

**Evaluator:** 

Print / Sign

Areas for Improvement / Comments:

# JPM QUESTIONS

Question #1:				
Answer #1:				
<u>Examinee</u> <u>Response</u> :				
Grade:	SAT	UNSAT		

Question #2:		
Answer #2:		
<u>Examinee</u> <u>Response</u> :		
Grade:	SAT	UNSAT

# STUDENT HANDOUT

JPM Number:	JPM-292-R-RO	Revision:	0/1				
Initial Conditions:	The plant is in MODE 5 day 2 of a schedule Reactor disassembly is in progress and the I removed within the next 12 hours.	• •	ę				
	The following additional conditions present	ly exist:					
	• "A" Train is protected.						
	• "B" EDG tagged for Maintenance						
	No Off-Site GRID Risk Penalty Factors						
	• RCS Boron concentration is 2200 ppm.						
	• RCS temperature is 105°F.						
	• PZR level is 20%						
	• PZR Vent Port Removed						
Initiatin <u>g Cues</u> :	The US has given you the current Shutc directed you perform a <u>predicted</u> SSA for for the following Sections only:	•					
	• Section 3 Decay Heat Removal	(DHR)					
	• Section 7 Power Availability						

Total Score and Condition Color

OMINION	OU-M2-201 REVISION 18 PAGE 22 OF 57
Section 1	Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist OU-M2-201 – Attachment 1 Page 1 of 9
Protected Train A 🗹/ B 🗌 (C	Check one or both) 🗌 with exception
Date/Time Performed: Today / 0000	
Actual Conditions     Predicted Conditions for	Days Shutdown: Reason for Shutdown Safety Assessment: 
Section 2 Heatup Data	
Time To Core Boil         Image: Time To Core Boil         Bubble does not exist in pressurizer AND fuel is in the vessel, THEN complete the following:         • RCS Temp: <u>105</u> °F         • RCS Level: <u>11</u> feet above flange         • RCS Time to Boil: <u>44.6 mins</u> □ NA if DEFUELED	Spent Fuel Pool Heatup Time         • SFP Temp: <u>95</u> °F         • SFP Level: <u>36</u> feet <u>10</u> inches         • SFP Time to 150°F         ☑ NA if <u>NO</u> freshly discharged fuel assemblies transferred to SFP or fuel assemblies are reloaded into reactor vessel         or         hrs mins         • SFP Time to 200°F <u>23</u> hrs <u>10</u> mins
Time to 200 °F (EA2 criterion): <u>39 mins</u>	Shutdown Risk Color is:  GREEN VELLOW
Time to Heatup $10^{\circ}$ F (EU1 criterion, uncontrolled heatup): <u>8.7 mins</u>	_ ORANGE _ RED Limiting Safety Function
	RCS or SFP Decay Heat Removal
RBCCW HX Outlet Temp: <u>80</u> °F Refuel Boron C <sub>b</sub> per TS: <u>2100</u> ppm RCS Boron C <sub>b</sub> : <u>2200</u> ppm	RCS or SFP Inventory Control     Reactivity Control     Containment     Power Availability
SFP Boron C <sub>b</sub> : <u>2200</u> ppm	SDC Responder phone: <u>x4335</u> Comments:

#### OU-M2-201 REVISION 18 PAGE 23 OF 57

# Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

	OU-M2-2	Page 2 of 9			
Section 3 Decay Heat Removal					
RCS Decay Heat Removal					
Check boxes for available equipment	Point Value	Score	Total	Condition	
✓ 'A' SDC with associated RBCCW and SW pump	(1)	1		(Circle)	
✓ 'B' SDC with associated RBCCW and SW pump	(1)	1	0	RED	
$\square$ 'A' CS with associated RBCCW and SW pump <sup>(3)</sup>	(1)		1	ORANGE	<b>,</b>
$\square$ 'B' CS with associated RBCCW and SW pump <sup>(3)</sup>	(1)		3	GREEN	
Both SGs <sup>(1)</sup>	(1)				
$\Box$ Refuel Pool $\geq$ 35'6" <sup>(4)</sup> or Notes <sup>(2)(4)</sup>	(1)				
Reduced Inventory Operation (RIO) Penalty	(-1)				
		[	1		
RCS Decay Heat Remov	al Total	2		NA if DEFU	ELED
Required Equipment (minimum):		(Che	ck)		
☐ If only one train of SDC available ensure:	А		В 🗌		
Associated train EDG available	А		В		
One U2 controlled offsite power source associated with available SDC train	RSS	Т	NSST 🗌		
During Reduced Inventory Operation (RIO) ensure:					
Both trains of SDC available with one train in service that is energized from a bus powered from an offsite source AND	Yes [	No 🗌	]	Required Equipment NOT met	RED
One RBCCW pump powered from independent power supplies for each credited SDC train AND	A 🗌	В	С□		
One SW pump powered from independent power supplies for each credited SDC train	A 🗌	В	С		
<sup>(1)</sup> Maintain all of the following satisfied to ensure two steam genera	ators availab	le and prop	er RCS condi	tions are established	to support

natural circulation:

- Both available SG NR levels greater than 10%
- Capability to feed available SGs with a MD AFW pump
- Capability to release steam from available SGs
- RCS loops associated with the available SGs; filled and unisolated
- Pressurizer pressure  $\geq$  50 psia <u>AND</u> a steam bubble is established in the pressurizer

<sup>(2)</sup>When refuel pool level is reduced to 31'6" to lift and set the UGS.

- <sup>(3)</sup>CS can be credited to backup LPSI for DHR in MODES 6 and Defueled per calculation ENG-04223M2, Rev. 0, Addendum 9. If CS is placed in service, no Core Alterations are allowed per Tech Specs.
- <sup>(4)</sup>In Modes 5 and 6, <u>IF</u> RCS is vented <u>AND</u> Refuel Pool is less than full (< 35'6"), an Operator must be stationed in the vicinity of the SW/Fire Water Supply valves to the EDG to be ready to take action to shift cooling water to Fire Water if directed by the SM.</p>

Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

	OU-M2-201 – Attachment 1	Page 3 of 9
Section 3 Decay Heat Removal (Continued)		
BEYOND	DESIGN BASIS	
Mode 5:	Ν	IA for Mode 0
Steam Generator available for Decay Heat Removal:	✓ Pressurize	er Vent Port Removed
AC Independent Aux Feedwater Pump:	OR DBB AFW RCS Injec	Pump Available for tion
Mode 6: ✓ BDB AFW Pump pre-staged for injection into the F	RCS	

#### OU-M2-201 REVISION 18 PAGE 25 OF 57

# Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

	(SSA) Cheo	CKIISI			
	OU-M2-201 -	Attachment	1		Page 4 of 9
Section 3 Decay Heat Removal (Continued)					
SFP Decay Heat Removal					
	Point Value	Score	Total	Condi	tion
✓ SFP level ≥ 35'6"	(1)	1		(Circle)	
✓ 'A' SFPC pump & HX with SFP level $\ge$ 36'4" ***	(0,½, 1)*	1	0	RE	c
✓ 'B' SFPC pump & HX with SFP level ≥ 36'4" ***	(0, 1/2, 1)*	1	1	ORAN	IGE
$\Box$ 'A' LPSI pump and SFP level $\geq$ 36'10" ***	(1)		2	YELLO	
$\Box$ 'B' LPSI pump and SFP level $\geq$ 36'10" ***	(1)		<u> </u>	GRE	
$\Box$ 'A' CS pump and SFP level $\geq$ 36'10" ***	(1)**				
$\Box$ 'B' CS pump and SFP level $\geq$ 36'10" ***	(1)**				
		0			
SFP Decay Heat Remo	val Total	3			
<ul> <li>counted as 0 points unless a cycle specific analysis demonstrate (i.e., each SFPC pump should be counted as ½ point). For 2R2: that both SFPC pumps together can be credited as one viable m and RBCCW temperature is maintained at &lt; 80° F. Following th point if requirements of TRM 3.9.3.3 are met. For 2R23, PM-170 following shutdown, since 85 fuel assemblies will be discharged</li> <li>** A cycle specific analysis is required for counting each available 04223M2, Rev 0, Addendum 9, supports each CS pump being and RBCCW temperature is maintained at &lt; 80° F. The UHS n average is &lt;70°F.</li> <li>***&gt; 36'10" if two SFP cooling pumps are operating or EITHER LF Level restriction is not applicable if SFP is cooled via Shutdowr and 2-SI-651 and 2-SI-652.</li> </ul>	3, Calculation EN eans of SFPC pr e core reload, ea 01, Rev. 0, demo during 2R23. CS pump as 1 p counted as 1 poi nust be <70°F, w PSI or CS supply	IG-04223M2, rovided the re the available s nstrates that point for SFPC nt provided th ith excursions ing SFP cooli RW-280 oper	Rev. 0, Adda actor has be SFPC pump TRM 3.9.3.3 c. For 2R23, lat fuel move s allowed for ng independ	endum 9, demo en shutdown $\geq$ should be cour will be met at $\geq$ Calculation EN ment begins $\geq$ <3 hours, if the ent of Shutdow	onstrates 8.3 days nted as 1 ≥ 17 days NG- 150 hours e moving m Cooling.
Required Equipment (minimum):	A 🗸	(Check) B □	СV	Required	
✓ One SW pump	A√	ВП	C √	Equipment	RED
✓ One RBCCW heat exchanger	A☑	вП	C 🔽	NOT met	
<ul> <li>✓ One SFPC or ☐ SDC heat exchanger</li> </ul>	A SFPC H A SDC H	IX 🗸 BSI			
NOTE: To maintain defense in depth for SFP cooling after the 81 <sup>st</sup> fuel as 3.9.3.3b requirements will be met for the 85 fuel assemblies disc					
Fuel Offload (81 to 217 Fuel Assemblies) Required					
Equipment (minimum, until reload complete):		Check)	_		
Two RBCCW pumps	A	B C [		Required	
Two SW pumps	A	в 🗌 с [		Required Equipment NOT met	ORANGE
One RBCCW heat exchanger	Α 🗌	B 🗌 C [		Hot mot	
Either of the following:					
Two SFPC heat exchangers	A	В			
One SDC heat exchanger	A	в			
<u> </u>				Form No. 729	0380(Oct 2015)

#### OU-M2-201 REVISION 18 PAGE 26 OF 57

# Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

	OU-M2-201	– Attachme	nt 1	Page 5 of 9		
Section 4 Inventory Control						
RCS Inventory Control						
Check boxes for available equipment:	Point Value	Score	Total	Condition		
☑ 'A' HPSI pump	(1)	_1	(	(Circle)		
☑ 'B' HPSI pump	(1)	1				
✓ 'C' HPSI pump	(1)	_1	0	RED		
$\checkmark$ 'A' Charging pump via $\checkmark$ RWST or $\square$ BAST <sup>(1,2)</sup>	$(1/2, 1)^{(3)}$	1/2	1	ORANGE		
✓ 'B' Charging pump via ✓ RWST or ☐ BAST <sup>(1,2)</sup>	$(1/2, 1)^{(3)}$	1/2	2	YELLOW		
$\Box$ 'C' Charging pump via $\Box$ RWST or $\Box$ BAST <sup>(1,2)</sup>	$(^{1}/_{2}, 1)^{(3)}$		≥3	GREEN		
RCS Inventory Control not required if						
DEFUELED <u>AND</u> RCS isolated from SFP by one of the following:						
□ 2-RW-280 CLOSED						
OR						
West SFP Gate INSTALLED			NIA 1			
				f DEFUELED AND		
RCS Inventory Con	trol Total	4	RCS iso	plated from SFP		
	l					
Required during RIO (minimum):			Require	d		
One HPSI pump			Equipme			
			NOT m	et		
SFP Inventory Control	Point					
Check boxes for available equipment:	Value	Score	Total	Condition		
One AFW pump aligned to CST	(1)			(Circle)		
One Refuel Purification pump	(1)		0	RED		
One PMW pump	(1)		1	ORANGE		
Makeup available from Fire Protection System			2	YELLOW		
(e.g., hoses)	(1)		<u>≥</u> 3	GREEN		
			<b>NA if</b>	MODE 5, 6, or		
SFP Inventory Con	trol Total	N/A		I Pool <u>&gt;</u> 36'4"		
	ı		-			
Requirements for RCS drain down conditions:						
SFP Cooling System vent and drain paths, which could	k					
SFP Cooling System vent and drain paths, which could affect SFP inventory, are identified and safety tagged		Number:				
SFP Cooling System vent and drain paths, which could	Tagout	Number:				

<sup>(2)</sup>RWST ≥ 57,300 gallons (12%) or BAST > 3,750 gallons (65.8%) to be available per TRM 4.1.2.7a and SP 2601F, "Borated Water Sources Verification, MODE 5 or 6."

<sup>(3)</sup> If  $\leq$  384 hrs (16 days) since shutdown, at least two Charging pumps with suction from the RWST or BAST and aligned to RCS to be credited as ONE viable makeup source.

#### OU-M2-201 REVISION 18 PAGE 27 OF 57

# Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

OU-M2-2	01 – Attachmer	Page 6 o	f 9	
Section 5 Reactivity Control				
Reactivity Control while in MODEs 5 or 6				
Check boxes for available equipment and conditions:	Point Value	Score	Total Condition	
RCS <u>AND</u> SFP boron concentrations greater than required by applicable Tech Specs	(1)	1	<i>(Circle)</i> 0-2 RED	
Dilution flowpaths identified (procedurally controlled <u>or</u> tagged) Tagout Number: <u>2207X99-0007</u>	(1)	1	3 ORANGE 4 YELLOW	
Inventory Flow Paths	(0-2)	2	(5 GREEN)	
🗹 ʻA' HPSI pump			GILLEN	
☐ 'B' HPSI pump				-
C' HPSI pump				
$\checkmark$ 'A' Charging pump aligned to $\checkmark$ RWST or $\square$ BAST <sup>(1,2)</sup>				
'B' Charging pump aligned to  RWST or  BAST <sup>(1,2)</sup>				
$\Box$ 'C' Charging pump aligned to $\Box$ RWST or $\Box$ BAST <sup>(1,2)</sup>				
$\boxed{\checkmark} \ge 2$ Source Range Monitor	(1)	1		
RCS Reactivity Control while in MODE 5 or 6 Total	5	NA if DEFUELED		
Required Equipment (minimum):	(Check)			
$\swarrow \ge 2$ Source Range Monitors			equired uipment RED	
✓ Inventory Flow Paths			DT met	
RCS AND SFP Boron concentrations greater than				
required by applicable Tech Specs				
Reactivity Control while DEFUELED				
Check boxes for available equipment and conditions <b>Point</b> Value	Score	Total	Condition	]
RCS AND SFP boron concentrations greater (1)			(Circle)	
than required by applicable rech Specs		0	RED	
Dilution flowpaths identified (procedurally controlled or Safety Tagging) (1)		1	YELLOW	
controlled or Safety Lagging)       (1)         Tagout Number:       2207X99-0007		2	GREEN	
				1
RCS Reactivity Control while DEFUELED Total	N/A	NA if	in MODE 5 or 6	

<sup>(1)</sup>Only two charging pumps must be capable of injecting based on T.S. 3.1.1.3.b., "Boron Dilution."

<sup>(2)</sup>RWST ≥ 57,300 gallons (12%) or BAST > 3,750 gallons (65.8%) to be available per TRM 4.1.2.7a and SP 2601F, "Borated Water Sources Verification, MODE 5 or 6."

#### Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

OU-M2-201	<ul> <li>Attachme</li> </ul>	nt 1	Page 7 of 9
Point Value	Score	Total	Condition
(0,2,3)	2	(0	Circle)
		0 1	RED ORANGE
		2	YELLOW GREEN
(1)	1		
<sup>3)</sup> (1)			
(1)			
(1)			
y (1)	1		
nent Total	5	NA if	DEFUELED
	Penetration V Penetration Tr Point Value (0,2,3) (1) (1) (1) (1)	Penetration Work Activit         Penetration Tracking She         Point Value       Score         (0,2,3)       2         (1)       1         (3)       (1)       1         (1)       1         (1)       1         (1)       1         (1)       1         (1)       1         (1)       1         (1)       1         (1)       1	Value       Score       Total $(0,2,3)$ $\frac{2}{2}$ $(0,1)$ $(0,2,3)$ $\frac{2}{2}$ $(0,1)$ $(1)$ $\frac{1}{2}$ $(2,3)$ $(3)$ $(1)$ $\frac{1}{1}$ $(1)$ $\frac{1}{1}$ $(1)$ $(1)$ $\frac{1}$

<sup>1</sup> Closure capability is scored based on all penetrations closed by at least one isolation valve or exceptions tracked and managed in accordance with OP 2264, "Conduct of Outages."

(2) This item is scored a "1" if no significant fuel failures are indicated by radiochemistry sampling. For the purposes of the SDR assessment, identification from radiochemistry samples and confirmation from NAF of significant fuel rod/pin failures is necessary to score this item as "0."

to score this item as "0." <sup>(3)</sup> No Core Alterations in progress in Containment is an indicator of the susceptibility to a fuel handling event. This item is scored a "1" if no Core Alterations are in progress or a "0" if Core Alterations are in progress. <sup>(4)</sup> This item is scored a "1" if the RCS is intact or a "0" if any RCS opening exists.

(5) After 8 days (from the start of the outage), it is assumed that the short-lived, volatile isotopes that are principally responsible for

early health effects have decayed sufficiently such that the event would not contribute to Large Early Release Frequency (LERF). <sup>(6)</sup> No Activities are in progress to preclude mitigation to a fuel handling accident. This item is scored a "1" if Decay Heat Removal, Inventory Control, and Power Availability are <u>NOT</u> Orange/Red. This item is scored a "0" if Decay Heat Removal, Inventory Control, and Power Availability are Orange/Red.

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Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

	(SSA) Ch	CCNIISI		
	OU-M2-201	- Attachme	nt 1	Page 8 of 9
Section 7 Power Availability				
<ul> <li>Check boxes for available equipment and conditions:</li> <li>✓ Power Availability</li> <li>✓ Bus 24E aligned to: ✓ 24C 24D</li> <li>On-site Power Source:</li> <li>✓ 'A' EDG with 'A' SW pump <u>or</u> 'B' SW pump supplied by the 'A' EDG</li> <li>'B' EDG with 'C' SW pump <u>or</u> 'B' SW pump supplied by the 'B' EDG</li> <li>SBO Diesel via 24E (Time to Boil &gt; 60 min)</li> </ul>	(1)	<b>Score</b>	Total 0-1 2 3 ≥ 4	Condition Circle) RED ORANGE YELLOW GREEN
Off-site Power Source: Unit 2 RSST Unit 2 NSST Unit 2 NSST Unit 3 RSST or NSST via 34A/B Power Source S	(1) (1) (1)	<u>1</u> <u>1</u> <u>1</u> <u>4</u>	]	
Required Equipment:         ✓       One EDG + One Unit 2 Controlled Off-site Source         □       IF in RIO at least one additional on site power source:         •       SBO Diesel and Calculated Time to Boil > 60 minut         •       Additional Unit 2 EDG	es		Required Equipment NOT met	RED
Off-Site GRID Risk Penalty Factor         Environmental Conditions <sup>(1)</sup> Avg sustained wind speed ≥ 75 mph         Salt contamination buildup or arcing in the 345 kV switch         OR         Switchyard Activities <sup>(1)</sup> Trip Testing affecting more than one 345 kV line         OR         OR         Switchyard Activities <sup>(1)</sup> Trip Testing affecting more than one 345 kV line         OR         ISO-NE/CONVEX Alerts <sup>(1)</sup> Abnormal transmission network conditions with potential loss of grid				
OR Planned Maintenance or Projects <sup>(2)</sup> SUBTRACT from Power Sub-Total <sup>(1)</sup> Power Availabi	Γ	() 4	Penalty	

<sup>(1)</sup>Apply offsite power source sub-total
 <sup>(2)</sup>If 345 kV or main transformer switchyard work is in progress which jeopardizes off-site sources, then deduct points equivalent to the number of offsite sources that could be affected.

#### OU-M2-201 REVISION 18 PAGE 23 OF 57

# Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

	OU-M2-	201 – Attacl	hment 1		Page 2 of 9
Section 3 Decay Heat Removal					
RCS Decay Heat Removal					
Check boxes for available equipment	Point Value	Score	Total	Condition	
☐ 'A' SDC with associated RBCCW and SW pump	(1)			(Circle)	
☐ 'B' SDC with associated RBCCW and SW pump	(1)		0	RED	
$\Box$ 'A' CS with associated RBCCW and SW pump <sup>(3)</sup>	(1)		1	ORANGE YELLOW	
$\Box$ 'B' CS with associated RBCCW and SW pump <sup>(3)</sup>	(1)		3	GREEN	
☐ Both SGs <sup>(1)</sup>	(1)				
$\Box$ Refuel Pool $\geq$ 35'6" <sup>(4)</sup> or Notes <sup>(2)(4)</sup>	(1)				
Reduced Inventory Operation (RIO) Penalty	(-1)				
			7		
RCS Decay Heat Remov	al Total			NA if DEFU	ELED
Required Equipment (minimum):		(Che	ck)		
☐ If only one train of SDC available ensure:	А		В 🗌		
Associated train EDG available	А		ВП		
One U2 controlled offsite power source associated with available SDC train		GT 🗌	NSST 🗌		
During Reduced Inventory Operation (RIO)					
ensure: Both trains of SDC available with one train in service that is energized from a bus powered from an offsite source AND	Yes [	No 🗌	]	Required Equipment NOT met	RED
One RBCCW pump powered from independent power supplies for each credited SDC train	A	] в 🗆	С 🗆		
AND One SW pump powered from independent power supplies for each credited SDC train	A	] В	С□		
<sup>(1)</sup> Maintain all of the following satisfied to ensure two steam general natural circulation:	ators availab	ble and prop	er RCS condi	tions are established t	o support

• Both available SG NR levels greater than 10%

• Capability to feed available SGs with a MD AFW pump

• Capability to release steam from available SGs

• RCS loops associated with the available SGs; filled and unisolated

• Pressurizer pressure > 50 psia AND a steam bubble is established in the pressurizer

 $^{\scriptscriptstyle (2)}When$  refuel pool level is reduced to 31'6" to lift and set the UGS.

<sup>(3)</sup>CS can be credited to backup LPSI for DHR in MODES 6 and Defueled per calculation ENG-04223M2, Rev. 0, Addendum 9. If CS is placed in service, no Core Alterations are allowed per Tech Specs.

<sup>(4)</sup>In Modes 5 and 6, <u>IF</u> RCS is vented <u>AND</u> Refuel Pool is less than full (< 35'6"), an Operator must be stationed in the vicinity of the SW/Fire Water Supply valves to the EDG to be ready to take action to shift cooling water to Fire Water if directed by the SM.</p>

Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

	OU-M2-201 – Attach	ment 1 Page 3 of 9
Section 3 Decay Heat Removal (Continued)		
BEYOND	DESIGN BASIS	
Mode 5:		NA for Mode 0
Steam Generator available for Decay Heat Removal:		Pressurizer Vent Port Removed
AC Independent Aux Feedwater Pump:	<u>OR</u>	BDB AFW Pump Available for
TDAFW Pump BDB AFW Pump		RCS Injection
Mode 6:		
BDB AFW Pump pre-staged for injection into the	RCS	
	1100	

# OU-M2-201 REVISION 18 PAGE 29 OF 57

#### Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

	OU-M2-201	- Attachmer	nt 1	Page 8 of 9
Section 7 Power Availability				
Check boxes for available equipment and conditions:  Power Availability	Point Value	Score	Total	
<ul> <li>Bus 24E aligned to: 24C 24D</li> <li>On-site Power Source:</li> <li>'A' EDG with 'A' SW pump <u>or</u> 'B' SW pump supplied by the 'A' EDG</li> <li>'B' EDG with 'C' SW pump <u>or</u> 'B' SW pump supplied by</li> </ul>	(1)		0-1 2 3 ≥4	(Circle) RED ORANGE YELLOW GREEN
the 'B' EDG SBO Diesel via 24E (Time to Boil > 60 min)	(1) (1)			
Off-site Power Source: Unit 2 RSST Unit 2 NSST Unit 3 RSST or NSST via 34A/B	(1) (1) (1)		-	
Power Source S	ub-Total			
Required Equipment:         One EDG + One Unit 2 Controlled Off-site Source         IF in RIO at least one additional on site power source:         SBO Diesel and Calculated Time to Boil > 60 minut         Additional Unit 2 EDG	es		Required Equipmer NOT met	it RED
Off-Site GRID Risk Penalty Factor         Environmental Conditions <sup>(1)</sup> Avg sustained wind speed ≥ 75 mph         Salt contamination buildup or arcing in the 345 kV swite         OR         Switchyard Activities <sup>(1)</sup> This Testing effection means then one 045 kV line	hyard			
<ul> <li>Trip Testing affecting more than one 345 kV line</li> <li>Two 345 kV lines out of service</li> <li>OR</li> <li>ISO-NE/CONVEX Alerts<sup>(1)</sup></li> <li>Abnormal transmission network conditions with potential loss of grid</li> </ul>	ll for			
OR Planned Maintenance or Projects <sup>(2)</sup>				
SUBTRACT from Power Sub-Total <sup>(1</sup>	) _ Г	( )	Penalty	
Power Availabil	ity Total			

<sup>(1)</sup>Apply offsite power source sub-total
 <sup>(2)</sup>If 345 kV or main transformer switchyard work is in progress which jeopardizes off-site sources, then deduct points equivalent to the number of offsite sources that could be affected.

# JOB PERFORMANCE MEASURE APPROVAL SHEET

PM Number:	JPM-291-R-RO	Revision: 0/1
initiated:		
	David Jacobs	05/20/2016
	Developer	Date
Reviewed:		
	Robert L. Cimmino, Jr.	07/12/2016
	Technical Reviewer	Date
Approved:		
	Supervisor, Nuclear Training	Date

# **SUMMARY OF CHANGES**

DATE	DESCRIPTION	<b>REV/CHANGE</b>
05/20/2016	Modified for ILT NRC Exam	0/0
08/18/2016	Incorporated NRC comments	0/1

#### JPM WORKSHEET

Facility:	Examinee:					
JPM Number:	JPM-291-R-RO	Revision: 0/1				
Task Title: OP 230	Task Title:   OP 2304C Batch Makeup to VCT Calculation					
System: CVCS	004					
Time Critical Task:	🗌 YES 🖾 NO					
Validated Time (minutes)	):30					
Task Number(s):	004-01-191	_				
Applicable To:	SRO STA	RO <u>X</u> PEO	_			
K/A Number:	A4.12 K/A Rating:	3.8 / 3.3				
Method of Testing: S	imulated Performance:	Actual Performance:	X			
Location: C	lassroom: <u>X</u> S	imulator: In-Plant	:			
Task Standards:	At the completion of this JPM Make Up to the Volume Contro	the Examinee will perform a calculation of Tank	on for a Batch			
Required Materials: (procedures, equipment, etc.)	MP-PROC-OPS-OP 2208[r015 MP-PROC-OPS-OP 2304C[r02	-				
General References:	MP-PROC-OPS-OP 2208[r015 MP-PROC-OPS-OP 2304C[r02	-				

#### \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

# JPM WORKSHEET

JPM	Number:	JPM-291-R-RO	Revision :	0/1
Initial Conditions:	• • •	nt is currently at 100% reactor p RCS boron concentration is 609 "C" Charging pump in service "A" BAST is in service with a Makeup Reactivity Correction T The PPC is currently unavailab	5 ppm boron boron concentration of Factor = 1	-
Initiating Cues:	Usi	<ul> <li>Unit Supervisor has directed ye</li> <li>Calculate a Neutral blend to</li> <li>Disregard volume of the b suction</li> <li>Determine Total Gallons</li> <li>Determine Gallons of PMV</li> <li>Determine Gallons of "A" 1</li> </ul>	o raise VCT level from olending tee to the char V BAST Boric Acid on & Dilution) Portion	n 75% to 85% rging pump

Simulator Requirements: N/A

# \* \* \* \* <u>NOTES TO TASK PERFORMANCE EVALUATOR</u> \* \* \* \*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

	JPM Number:	JPM-291-R-RO		Revision:	0/1	
	Task Title:	OP 2304C Batch Make	eup to VCT Calculation			
					START TIME: _	
STEP #1	Portion of CVCS" Section 4.6 Batch Mak	xeup (Boration & Dilution) xe Up to VCT. ed VCT level change in %	Standard: Examinee refers to the Initial Condition determines 10%	ons and	Critical: Y □ N ⊠	Grade S 🗌 U 🗌
	Comments:					
STEP #2	Performance: <b>4.6.4</b> DETERMINE to make desired level change in desired level change = $\frac{\% \times 34 \text{ gallons}}{1\% \text{ level}} = Total$	nge as follows: n	Standard: Examinee calculates: <u>10% x 34 gallons</u> = 340 gallo 1% level	ons total	Critical: Y 🛛 N 🗌	Grade S 🗌 U 🗌
	Cue: Comments:					

JPM Number: JPM-291-R-RO

Revision: 0/1

## Task Title: OP 2304C Batch Makeup to VCT Calculation

STEP #3	Performance: 4.6.5 Refer To OP 2208, "Reactivity Calculations," or PPC and DETERMINE	Standard: Examinee refers to OP 2208 Reactivity Calculations	Critical: Y $\square$ N $\boxtimes$	Grade S 🗌 U 🗌
	required ratio of boric acid flow to PMW flow, corrected for Boron-10 depletion.			
	Blended Makeup Flowrate Determination Formula: flowrate = [K x (boric acid flowrate)] ÷ CF	PMW flowrate = $\frac{18 \times (1)}{1 \div 1} = 8$		
	Where, K= <u>(ppm boron in BAST) - (ppm boron in makeup)</u> ppm boron in makeup CF= Makeup Reactivity Correction Factor	Where, $8 = (5445) - (605)$ 605		
	(From Att 5. "Reactivity Thumb Rules," sheet OR Reactor Engineer)	CF = 1 from initial conditions		
	Cue:			
	Comments:			
STEP #4	Performance: <b>4.6.6</b> DETERMINE total gallons of boric acid required to make desired level change as follows: (BA = boric acid)	Standard: Examinee determines:	Critical: Y ⊠ N □	Grade S 🗌 U 🗌
	Total gallons $\mathbf{x}$ BA flowrate= Total boric acid volumefor makeupBA + PMW flow rate	340 gallons total x $\underline{1}$ (1 + 8)		
		= 37.7777777 (38 gallons)		
	Cue:		1	
	Comments:			

	JPM Number: JPM-291-R-RO	Revision:	0/1	
	Task Title: OP 2304C Batch Make	eup to VCT Calculation		
STEP #5	Performance: <b>4.6.7</b> DETERMINE total gallons of PMW         required to make desired level change as follows:         (BA = boric acid):         Total gallons x       PMW flowrate         for makeup       BA+ PMW flow rate	Standard:      Examinee determines:      340 gallons total x    8      (8+1)	Critical: Y ⊠ N □	Grade S 🗌 U 🗌
	Cue: Comments:	= 302.22222222 (302 gallons)		

TERMINATION CUE: The evaluation for this JPM is concluded.

**STOP TIME**: \_\_\_\_\_

Total Gallons	
	340 gallons
PMW Gallons	
	<b>302</b> gallons
"A" BAST Gallons	
	38 gallons

# **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-291-R-RO

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🖾 No	0	
Validated Time (minutes):	30	Actual Time to Complete (minutes):		
Work Practice Performance:		SAT	UNSAT	
Operator Fundamentals:		SAT	UNSAT	
JPM Question Portion Overall [ <i>NLO only</i> ]:		SAT	UNSAT	N/A
Attached Question #1		SAT	UNSAT	
Attached Question #2		SAT	UNSAT	
Overall Result of JPM:		SAT	UNSAT	

Evaluator:

Print / Sign

Areas for Improvement / Comments:

# JPM QUESTIONS

Question #1:				
Answer #1:				
<u>Examinee</u> <u>Response</u> :				
Grade:	SAT	UNSAT		

Question #2:				
Answer #2:				
<u>Examinee</u> <u>Response</u> :				
Grade:	SAT	UNSAT		

# STUDENT HANDOUT

JPM Number:	JPM-291-R-RO	Revision:	0/1
Initial Conditions:	The plant is currently at 100% reactor po	wer with the following	g conditions:
	• RCS boron concentration is 605	ppm boron	
	• "C" Charging pump in service		
	• "A" BAST is in service with a bo	oron concentration of 5	5445 ppm
	Makeup Reactivity Correction Fa	actor = 1	
	• The PPC is currently unavailable		
Initiating Cues:	<ul><li>The Unit Supervisor has directed you</li><li>Calculate a Neutral blend to a</li></ul>		U U
	• Disregard volume of the ble suction	ending tee to the charg	ing pump
	Determine Total Gallons		
	• Determine Gallons of PMW		
	• Determine Gallons of "A" B.	AST Boric Acid	
	Using OP 2304C "Makeup (Boration section 4.6 Batch Make Up to VCT.	& Dilution) Portion of	of CVCS"

ANSWER:

Total Gallons	
PMW Gallons	
"A" BAST Gallons	

# JOB PERFORMANCE MEASURE APPROVAL SHEET

PM Number:	JPM-290-R-4B	Revision: 0/1
nitiated:		
	David J. Jacobs	05/24/2016
	Developer	Date
Reviewed:		
	Robert L. Cimmino, Jr.	07/13/2016
	Technical Reviewer	Date
Approved:		
	Supervisor, Nuclear Training	Date

# **SUMMARY OF CHANGES**

DATE	DESCRIPTION	<b>REV/CHANGE</b>
05/24/2016	New JPM for NRC ILT Exam 2016	0
08/18/2016	Incorporated NRC comments	0/1

Facility: MP-2	Examinee:		
JPM Number:	JPM-290-R-4B	Revision: 0/1	_
Task Title: SP 260	2B Transient Temperature,	Pressure Verification	
System: 005 RH	IR		_
Time Critical Task:	YES NO		
Validated Time (minutes	):30		
Task Number(s):	121-01-167		
Applicable To:	SRO X STA	ROX	PEO
K/A Number: 005 A1	.01 / 2.2.42 K/A Rating	: 3.5 / 3.6 3.9 /4.6	
Method of Testing: S	imulated Performance:	Actua	l Performance: X
Location: C	Classroom: X	Simulator:	In-Plant:
Task Standards:	At the completion of the JP and referred to SP 2602B to Spec Limits		wed the Computer Printout es are NOT within the Tech.
Required Materials:	SP 2602B Transient Tempe	erature, Pressure Verification	on
(procedures, equipment, etc.)	SP 2602B-001 Transient Te PPC SP 2602B Printout	emperature, Pressure Verif	ication Data Sheet
General References:	SP 2602B Transient Tempe	erature, Pressure Verification	on

## \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

JPM Nur	JPM Number: <b>JPM-290-R-4</b>		Revision : 0/1
Initial Conditions:		t has completed a cooldown to vn" using "A" and "B" RCPs	o 125°F in accordance with OP 2207 concurrently with SDC.
		DC was placed in service at 0 CPs secured at 0300.	)105
Initiating Cues:			
		and perform SP 2602B "Tra	o use the computer data from the nsient Temperature, Pressure
Simulator Requirements:	N/A		

# \* \* \* \* <u>NOTES TO TASK PERFORMANCE EVALUATOR</u> \* \* \* \*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

S

S

	JPM Number:	JPM-290-R-4B	Re	evision:	0/1	
	Task Title:	SP 2602B Transient Te	mperature, Pressure Verification			
				S	START TIME: _	
E P 1	<ul> <li>Actions</li> <li>4.1.2 IF available, DES involved in controlling to perform the following</li> <li>DETERMINE and on SP 2602B-001.</li> <li>MONITOR param or cooldown rates</li> </ul>	own Initial and Conditional SIGNATE a person not the Heatup or cooldown ng: RECORD required data eters, limits, and Heatup	<ul> <li>Standard:</li> <li>Examinee refers to OP 2602B, the Computouts performs the following:</li> <li>Records the Cooldown starting at 000</li> <li>Uses T115 (refer to Stem A&amp;B RCPs</li> <li>Uses T351Y at 0330 when RCPs are set of the start of the s</li></ul>	0 Loop #1)	Critical: Y 🗌 N 🕅	Grade S 🗌 U 🗌
	Cue:					
	Comments:					
E P 2	<ul><li>for trending and da Heatup or cooldow</li><li>OBTAIN required</li></ul>	ing: d(s) of selected parameters ata gathering during /n. 30 minute data sheets	Standard: Examinee records and determines Hourly Cooldown Rates recording parameters even hour	ery half	Critical: Y 🗌 N 🔀	Grade S 🗌 U 🗌
	from PPC printer, Cue:	and Go To step 4.1.10.				
	Comments:					

JPM Number: JPM-290-R-4B

Revision: 0/1

# Task Title:SP 2602B Transient Temperature, Pressure Verification

STEP #3	<ul> <li>Performance:</li> <li>4.1.10 IF at any time, any administrative limit (except pressurizer spray line differential temperature between 200°F and 350°F) or any TS/TRM acceptance criteria is not met, PERFORM the following:</li> <li>Immediately NOTIFY Shift Manager.</li> </ul>	Standard: Examine determines during the transition from RCPs to SDC when securing the RCPs the Technical Specification was exceeded at 51°F per hour below 220°F at time 0230 to 0330 and informs the Unit Supervisor	Critical: Y ⊠ N □	Grade S 🗌 U 🗌
	Cue:			
	Comments:			

TERMINATION CUE: The evaluation for this JPM is concluded.

**STOP TIME**: \_\_\_\_\_

# **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-290-R-4B

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🖾 No	)	
Validated Time (minutes):	30	Actual Time to Complete (minutes):		
Work Practice Performance:		SAT	UNSAT	
Operator Fundamentals:		SAT	UNSAT	
JPM Question Portion Overall [NLO only]:		SAT	UNSAT	N/A
Attached Question #1		SAT	UNSAT	
Attached Question #2		SAT	UNSAT	
Overall Result of JPM:		SAT	UNSAT	

Evaluator:

Print / Sign

Areas for Improvement / Comments:

# JPM QUESTIONS

Question #1:				
Answer #1:				
<u>Examinee</u> <u>Response</u> :				
Grade:	SAT	UNSAT		

Question #2:		
Answer #2:		
<u>Examinee</u> <u>Response</u> :		
Grade:	SAT	UNSAT

JPM Number:	JPM-290-R-4B	8	Revision:	0/1
Initial Conditions:				
	The Plant has complet "Cooldown" using "A			ith OP 220'
		ed in service at 0105		
Initiating Cues:				
Initiating Cues:	The Unit Supervisor h cooldown and perform Verification."			
<u>Initiating Cues</u> :	cooldown and perform			
<u>Initiating Cues</u> :	cooldown and perform			
<u>Initiating Cues</u> : Technical Specificati	cooldown and perform Verification."	1 SP 2602B "Transier		

# JOB PERFORMANCE MEASURE APPROVAL SHEET

JPM Number:	JPM-293-R-RO	Revision: 2/3
initiated:		
	David Jacobs/Robert L. Cimmino	09/28/2016
	Developer	Date
Reviewed:		
	Paul Prichard	09/28/2016
	Technical Reviewer	Date
Approved:		
	John W. Riley	09/29/16
	Supervisor, Nuclear Training	Date

# **SUMMARY OF CHANGES**

DATE	DESCRIPTION	<b>REV/CHANGE</b>
10/23/08	Revised JPM for LOIT 2008 NRC Exam	1/0
12/29/08	Incorporated NRC Post-Validation comments	1/0
06/02/2016	Up dated for ILT NRC Exam 2016	2/0
08/18/2016	Incorporated NRC comments	2/1
09/15/2016	Incorporated changes due to abbreviated survey map	2/2
09/28/2016	Incorporated changes due to change in interpretation of Initiating Cue Item #7	2/3

Facility: MP2	Examinee:		
JPM Number:	JPM-293-R-RO	Revision: <u>2/3</u>	
Task Title: Rev	view RWP and Survey Map		
System: Rad	liation Control		
Time Critical Task:	🗌 YES 🖾 NO		
Validated Time (min	utes): <u>15</u>		
Task Number(s):	404-01-004		
Applicable To:	SRO X STA	RO <u>X</u> PEO	
K/A Number:	2.3.7 K/A Rating	g: <u>3.5 / 3.6</u>	
Method of Testing:	Simulated Performance:	Actual Performance:	X
Location:	Classroom: X	Simulator: In-Plant:	
Task Standards:	*	PM the examinee has reviewed the applicable ne radiological requirements to perform the as	
<u>Required Materials</u> : (procedures, equipment, e	Operations blanket RWP N tc.) Survey map for -5' 6" Wes		
General References:	MP-PROC-HP-RPM 5.2.2[	[r016] Basic Radiation Worker Responsibilit	ties

#### \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

	JPM Number:	JPM-293-R-RO	Revision :	2/3
Initial Condition	<u>ы.</u> На • ТҮ Тү	the crew is performing a plant heat teatup the plant is in MODE 3 with pressur two Charging Pumps are in operation the gate in the -5 penetration room i	rizer pressure at 1400 j on.	
Initiating Cues:	<ul> <li>2</li> <li>Ba mi</li> <li>Yo</li> <li>St an 1.</li> <li>2.</li> <li>3.</li> <li>4.</li> <li>5.</li> <li>6.</li> <li>7.</li> </ul>	<ul> <li>Highest radiation level in the wood Highest contamination level in the units of measure)</li> <li>Protective clothing required in the transition to and from the area)</li> <li>Expected dose for this assignment Dose rate alarm for this area (incomposed on the second on the</li></ul>	stimate that this task v or entering this area. I opropriate for this assig rk area (including unit ne immediate work are ne immediate work are nt area (including units cluding units of measu are encountered, the l er the RWP requireme	vill take 20 Include in your gnment ts of measure) ea (including a (including s of measure) re) longest nts

Simulator Requirements: N/A

# \* \* \* \* <u>NOTES TO TASK PERFORMANCE EVALUATOR</u> \* \* \* \*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

	JPM Number: JPM-293-R-RO	Revision:	2/3
	Task Title: <b>Review RWP and Su</b>	arvey Map	
		S	START TIME:
STEP #1	Performance: Review Operations Blanket RWP No. 5 and Radiation Survey Figure 21A	Standard: Examinee reviews Operations Blanket RWP No. 5 and Radiation Survey Figure 21A and answers the following questions:	Critical:GradeYNSU
	Cue:	anket RWP No. 205 and Radiation Survey map.	
	Comments:	anket KWF NO. 203 and Kadiation Survey map.	
STEP #2	Performance: 1. Determine which RWP task (job step) is appropriate for this assignment.	Standard: Examinee states that task (job step) No. 1 (No. 2, if HRA is noted on map) is appropriate for this task.	Critical:GradeYNSU
	Cue:		
		esignated as a High Radiation Area in the past due to cha 1. Critical aspect of JPM is that subsequent data perf	
STEP #3	Performance: 2. Determine the highest radiation level in the immediate work area.	Standard:	Critical:GradeYNSU
	Cue:		
	Comments: The examinee may point out the 400 mr/hr hot s area. Hence the possibility of using Task No. 1.	pot near 2-SI-709, but that the assigned task does NOT r	equire him/her to approach that
STEP #4	Performance: 3. Determine the highest contamination level in the work area.	Standard: Examinee states that the highest contamination level in this area is 2,000 DPM/100cm <sup>2</sup> .	Critical:GradeYNSU
	Cue:		
	Comments:		

# JPM Number: JPM-293-R-RO

Revision: 2/3

# Task Title:**Review RWP and Survey Map**

STEP #5	Performance: 4. Determine what protective clothing is required in the area.	Standard: Examinee states that contamination levels require <b>full PCs</b> (with modesty garments underneath).	Critical: Y $\boxtimes$ N $\square$	Grade S 🗌 U 🗌
	•	e up "full PCs" (Cotton liners, Booties, Coveralls, Sho	e covers, Rubber §	gloves, Modesty
S T E P # 6	garments) Performance: 5. Determine the expected dose for this assignment. Cue:	Standard: Examinee states that the expected dose is approximately <b>15mrem</b> (14-16 mrem ).	Critical: Y 🖾 N 🗌	Grade S 🗌 U 🗌
	Comments: 45 mrem/hr Dose Rate X 1/3 hrs. (i.e.; 20 min.) ≅	15 mr		
S T E P # 7	<ul><li>Performance:</li><li>6. Determine the expected dose rate alarm for this assignment.</li></ul>	Standard: Examinee states the following: <u>If using Task #1</u> : 50 mr/hr dose rate alarm	Critical: Y ⊠ N □	Grade S 🗌 U 🗌
	Cue:	<u>If using Task #2</u> : 300 mr/hr dose rate alarm		
	Comments:			

\_\_\_\_\_

JPM Number: JPM-293-R-RO

Revision: 2/3

## Task Title: **Review RWP and Survey Map**

STEP #8	<ul><li>Performance:</li><li>7. Determine the longest possible stay time for this assignment.</li></ul>	Standard: <i>Examinee states the maximum stay time:</i> <u>If using Task #1</u> : <b>26 minutes</b> <u>If using Task #2</u> : <b>53 minutes</b>	Critical: Y 🛛 N 🗌	Grade S 🗌 U 🗌
	Cue:			
		tte X 80% RWP Exit Requirement = 26.6 minutes (25 tte X 80% RWP Exit Requirement = 53.3 minutes (52		

TERMINATION CUE: The evaluation for this JPM is concluded.

STOP TIME: \_\_\_\_\_

# **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-293-R-RO

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		Yes No			
Validated Time (minutes):	15	Actual Time to	Complete (minutes):		
Work Practice Performance:		SAT	UNSAT		
Operator Fundamentals:		SAT	UNSAT		
JPM Question Portion Overal	l [NLO only]:	SAT	UNSAT		N/A
Attached Question #1		SAT	UNSAT		
Attache	d Question #2	SAT	UNSAT		
Overall Result of JPM:		SAT	UNSAT		

Evaluator:

Print / Sign

Areas for Improvement / Comments:

# JPM QUESTIONS

Question #1:				
Answer #1:				
<u>Examinee</u> <u>Response</u> :				
Grade:	SAT	UNSAT		

Question #2:				
Answer #2:				
<u>Examinee</u> <u>Response</u> :				
Grade:	SAT	UNSAT		

# STUDENT HANDOUT

JPM Number:	JPM-293-R-RO	Revision:	2/3
Initial Conditions:	<ul> <li>The crew is performing a plant hear Heatup</li> <li>The plant is in MODE 3 with press Two Charging Pumps are in operat</li> <li>The gate in the -5 penetration room</li> </ul>	urizer pressure at 1400 j	
Initiating Cues:	<ul> <li>You have been directed to isolate L 2-CH-110Q.</li> <li>Based on previous experience, you minutes.</li> <li>Your available dose is 1,000 mR.</li> <li>State the radiological requirements answer: <ol> <li>Which RWP task (job step) is a</li> <li>Highest radiation level in the w</li> <li>Highest contamination level in units of measure)</li> </ol> </li> <li>Protective clothing required in transition to and from the area)</li> <li>Expected dose for this assignm</li> <li>Dose rate alarm for this area (in 7. Assuming significant difficultio possible stay time for this area</li> </ul>	estimate that this task v for entering this area. I appropriate for this assig york area (including unit the immediate work are the immediate work are ent area (including units including units of measu es are encountered, the I per the RWP requireme	will take 20 Include in your gnment ts of measure) ea (including ea (including s of measure) re) longest ents
	The examiner will act as Health Physic	s (HP) for any related q	uestions.

1. RWP Task	
2. Highest Rad. Lvl.	
3. Highest contamination Lvl.	
4. Required PCs	
5. Expected Dose	
6. Dose Rate Alarm	
7. Max Stay Time	

# JOB PERFORMANCE MEASURE APPROVAL SHEET

PM Number:	JPM-295-R-SRO	Revision: 0/1
i wi ivuilloer.	J1 WF2/J-K-5KO	
nitiotodu		
initiated:		
	David Jacobs	06/14/2016
	Developer	Date
Reviewed:		
	Robert L. Cimmino, Jr.	07/05/2016
	Technical Reviewer	Date
Approved:		
	Supervisor, Nuclear Training	Date

# **SUMMARY OF CHANGES**

DATE	DESCRIPTION	<b>REV/CHANGE</b>		
06/14/2016	Newly Created for NRC ILT Exam 2016	0		
08/19/2016	Incorporated NRC comments	0/1		

Facility: MP2	Examinee:				
JPM Number: J	PM-295-R-SRO	Revision:	0/1		
Task Title: Shutdo	wn Safety Assessment Review	for RIO condition	ons		
System: Conduc	t of Operations				
Time Critical Task:	🗌 YES 🖾 NO				
Validated Time (minutes)	):40				
Task Number(s):	119-01-044				
Applicable To:	SRO X STA	RO	PEO		
K/A Number: 2	.1.23 K/A Rating:	4.3 / 4.4	_		
Method of Testing: Si	imulated Performance:		Actual Performance:	X	
Location: C	lassroom: <u>X</u> S	Simulator:	In-Plant:		
Task Standards:	At the completion of this JPM condition of the Key Safety Fu			he predicted	
Required Materials:	MP-PROC-000-OU-M2-201[r	018.00] Shutdow	n Safety Assessment Ch	necklist	
(procedures, equipment, etc.)	Handout SSA Predicted Conditions pdf format				
	Handout CTMT pen closure p	an pdf format			
General References:	MP-PROC-000-OU-M2-201[r	018.00] Shutdown	n Safety Assessment Ch	necklist	

# \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

	JPM Number:	JPM-295-R-SRO	Revision :	0/1
Initial Conditior	React	plant is in MODE 5 day 1 of a sche tor disassembly is in progress and t tory within the next 12 hours.		
	The f	ollowing additional conditions pres	sently exist:	
	•	PZR level is 20%		
	•	PZR Vent Port Removed		
	•	No Equipment out of service rea	quired for Mode 5	
	•	Containment Closure is set with closure time of 30 minutes. (See	*	
	•	Time to Core Boil for Reduced	Inventory was verified	d by the STA
		me no change in Equipment Status ant is in Reduced Inventory.	from the current cond	lition to when
Initiating Cues:				
	RCS	ew the <u>Predicted</u> changes to the Shu in Reduce Inventory that was comp me To Core Boil calculation.	•	

Simulator Requirements: N/A

# \* \* \* \* <u>NOTES TO TASK PERFORMANCE EVALUATOR</u> \* \* \* \*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

	JPM Number: JPM-295-R-SRO	Revision:	0/1	
	Task Title: Shutdown Safety Ass	essment Review for RIO conditions		
		S	START TIME: _	
STEP #1	Performance: Reviews Section 2 Heatup Data: • Time to Core Boil • Shutdown Risk Color Cue: Comments:	<ul> <li>Standard:</li> <li>Examinee should note the following:</li> <li>Shutdown risk Color should be ORANGE</li> </ul>	Critical: Y ⊠ N □	Grade S 🗌 U 🗌
	Section 3 Decay heat removal and Section 6 Con	tainment were incorrectly calculated and should be ORA	1	
S T E P # 2	Performance: Reviews Section 3 Decay Heat Removal Data: • Reduced Inventory Operation (RIO) Penalty • RCS Decay Heat Removal Total • DHR Color Condition Cue:	<ul> <li>Standard:</li> <li>Examinee should note the following:</li> <li>The RIO Penalty was not subtracted</li> <li>RCS Decay Heat Removal Total was not calculated correctly and should be 1</li> <li>SF Color ORANGE should be circled</li> </ul>	Critical: Y 🛛 N 🗌	Grade S 🗌 U 🗌
	Comments:			
STEP #3	Performance: Reviews Section 4 Inventory Control Data: • No Discrepancies	Standard: Examinee should note No Discrepancies	Critical: Y 🗌 N 🔀	Grade S 🗌 U 🗌
	Cue:		1	
	Comments: For Inventory control refer to Control Room dail	y Surveillance SP 2619A-003 page 20 for the availabilit	ty of all 3 HPSI pu	mps

Revision: 0/1

# Task Title: Shutdown Safety Assessment Review for RIO conditions

GERE			<b>G</b> 1.1	0 1
S T E P	Performance:	Standard:	Critical:	Grade
#4	Reviews Section 5 Reactivity Control Data:	Examinee should note No Discrepancies	Y 🗌 N 🔀	S U U
	<ul> <li>No Discrepancies</li> </ul>			
	Cue:			
	Comments:			
	For Reactivity the difference in the number of HPS	SI pumps available as compared to Inventory Control is	s described in a N	OTE in OU-
	M2-201 Attachment 6 page 48 of 57.			
STEP	Performance:	Standard:	Critical:	Grade
# 5	Reviews Section 6 Containment Data:	Examinee should note the following:	Y 🔀 N 🗌	S 🗌 U 🗌
	Containment Closure Capability	Containment Closure Capability		
	• Containment Closure Set with	• Set with Exception Tracked, no		
	exceptions tracked and capable	longer Qualifies due to the change		
	of being closed prior to the lesser	in Time to Core Boil is less than		
	of:	the closure time for the penetration		
	<ul> <li>Time to Core Boil</li> </ul>	Decay Heat, Inventory Control, Power		
	Decay Heat, Inventory Control, Power	Availability Functions <b>NOT</b>		
	Availability Functions <b>NOT</b>	Orange/Red <sup>(6)</sup> should <b>not</b> be checked due		
	Orange/Red <sup>(6)</sup>	to the discrepancy in the DHR SF changing		
	Containment Total Score	to <b>ORANGE</b> (Error carried forward)		
		<ul> <li>Containment Total Score should be 1</li> </ul>		
	CTMT Color Condition			
		CTMT Color <b>ORANGE</b> should be circled		
	Cue:			
	Comments:			
S T E P	Performance:	Standard:	Critical:	Grade
#6	Reviews Section 7 Power Availability Data:	Examinee should note No Discrepancies	Y 🗌 N 🔀	S 🗌 U 🗌
	Cue:			
	Comments:			

TERMINATION CUE: The evaluation for this JPM is concluded.

**STOP TIME**: \_\_\_\_\_

# **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-295-R-SRO

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		Tyes X No			
Validated Time (minutes):	40	Actual Time to C	Complete (minutes):		
Work Practice Performance:		SAT	UNSAT		
Operator Fundamentals:		SAT	UNSAT		
JPM Question Portion Overal	l [NLO only]:	SAT	UNSAT		N/A
Attache	d Question #1	SAT	UNSAT		
Attache	d Question #2	SAT	UNSAT		
Overall Result of JPM:		SAT	UNSAT		

**Evaluator:** 

Print / Sign

Areas for Improvement / Comments:

# JPM QUESTIONS

Question #1:				
Answer #1:				
<u>Examinee</u> <u>Response</u> :				
Grade:	SAT	UNSAT		

Question #2:		
Answer #2:		
<u>Examinee</u> <u>Response</u> :		
Grade:	SAT	UNSAT

# STUDENT HANDOUT

JPM Number:	JPM-295-R-SRO	Revision:	0/1				
Initial Conditions:	The plant is in MODE 5 day 1 of a scheduled 28 day refueling outage. Reactor disassembly is in progress and the RCS is expected to be in Reduced Inventory within the next 12 hours.						
	The following additional conditions presently exist:						
	• PZR level is 20%						
	PZR Vent Port Removed						
	• No Equipment out of service required for Mode 5						
	• Containment Closure is set with 1 exception Penetration #48 with a closure time of 30 minutes. (See attached Closure Plan)						
	• Time to Core Boil for Reduced Inventory was verified by the STA						
	Assume no change in Equipment Status from the current condition to when the plant is in Reduced Inventory.						
Initiating Cues:							
	Review the <u>Predicted</u> changes to the Shutdo RCS in Reduce Inventory that was complet	•					

Review the <u>Predicted</u> changes to the Shutdown Safety Assessment for the RCS in Reduce Inventory that was completed by the RO with the <u>exception</u> of Time To Core Boil calculation.

OMINION	OU-M2-20 REVISION 1 PAGE 22 OF 5			
Dominion	Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist			
	OU-M2-201 – Attachment 1 Page 1 of 9			
Section 1 Protected Train A / B (C	<i>theck one or both)</i> with exception			
Date/Time Performed: <u>Today / 0000</u>	Date/Time of Shutdown: <u>3 DAYS AGO 0000</u> Days Shutdown: <u>3</u>			
Predicted Conditions for <u>reduced inventory</u>	Reason for Shutdown Safety Assessment: Configuration Change (00:00 hour, mode change, configuration changes)			
Section 2 Heatup Data Time To Core Boil	Spent Fuel Pool Heatup Time			
<ul> <li>Bubble does not exist in pressurizer <u>AND</u> fuel is in the vessel, <u>THEN</u> complete the following:</li> <li>RCS Temp: <u>105</u> °F</li> <li>RCS Level: <u>-3</u> feet above flange</li> <li>RCS Time to Boil: <u>12.77 min</u>s</li> <li><b>NA if DEFUELED</b></li> </ul>	<ul> <li>SFP Temp: <u>95</u> °F</li> <li>SFP Level: <u>36</u> feet <u>10</u> inches</li> <li>SFP Time to 150°F</li> <li>NA if <u>NO</u> freshly discharged fuel assemblies transferred to SFP or fuel assemblies are reloaded into reactor vessel</li> <li>or hrs mins</li> <li>SFP Time to 200°F <u>23</u> hrs <u>10</u> mins</li> </ul>			
Time to 200 °F (EA2 criterion): <u>11.34 mins</u> □ <b>NA if DEFUELED</b> Time to Heatup 10 °F (EU1 criterion, uncontrolled heatup): <u>2.15 mins</u>	Shutdown Risk Color is: GREEN YELLOW ORANGE RED Limiting Safety Function			
$\square$ NA if DEFUELEDRBCCW HX Outlet Temp: <u>80</u> °FRefuel Boron Cb per TS: <u>2100 ppm</u> RCS Boron Cb: <u>2200 ppm</u> SFP Boron Cb: <u>2200 ppm</u>	<ul> <li>RCS or SFP Decay Heat Removal</li> <li>RCS or SFP Inventory Control</li> <li>Reactivity Control</li> <li>Containment</li> <li>Power Availability</li> <li>SDC Responder phone: x4335</li> <li>Comments:</li></ul>			

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#### DOMINION

#### OU-M2-201 REVISION 18 PAGE 23 OF 57

# Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

	OU-M2-2	201 – Attacł	nment 1	Pa	age 2 of 9
Section 3 Decay Heat Removal					
RCS Decay Heat Removal					
Check boxes for available equipment	Point Value	Score	Total	Condition	
✓ 'A' SDC with associated RBCCW and SW pump	(1)	1		(Circle)	
♂ 'B' SDC with associated RBCCW and SW pump	(1)	1	0	RED ORANGE	
$\Box$ 'A' CS with associated RBCCW and SW pump <sup>(3)</sup>	(1)		2	YELLOW	
$\Box$ 'B' CS with associated RBCCW and SW pump <sup>(3)</sup>	(1)		3	GREEN	
□ Both SGs <sup>(1)</sup>	(1)				
$\square$ Refuel Pool $\ge$ 35'6" <sup>(4)</sup> or Notes <sup>(2)(4)</sup>	(1)				
Reduced Inventory Operation (RIO) Penalty	(-1)				
RCS Decay Heat Remov	al Total	2		NA if DEFUE	LED
Required Equipment (minimum):		(Chea	ck)		
<ul> <li>If only one train of SDC available ensure:</li> <li>Associated train EDG available</li> <li>One U2 controlled offsite power source associated with available SDC train</li> </ul>	А		B 🗌 B 🗍 NSST 🗍		
<ul> <li>During Reduced Inventory Operation (RIO) ensure:</li> <li>Both trains of SDC available with one train in service that is energized from a bus powered from an offsite source</li> </ul>	Yes 🖸	No [	]	Required Equipment NOT met	RED
<u>AND</u> ✓ One RBCCW pump powered from independent power supplies for each credited SDC train <u>AND</u> ✓ One SW pump powered from independent	A 🔽	В			
power supplies for each credited SDC train	A	В	C 🗸		
<sup>(1)</sup> Maintain all of the following satisfied to ensure two steam genera natural circulation;	ators availab	le and prope	er RCS condi	tions are established to	support

• Both available SG NR levels greater than 10%

Capability to feed available SGs with a MD AFW pump

Capability to release steam from available SGs

• RCS loops associated with the available SGs; filled and unisolated

• Pressurizer pressure > 50 psia AND a steam bubble is established in the pressurizer

 $^{(2)}$  When refuel pool level is reduced to  $\overline{31'6''}$  to lift and set the UGS.

<sup>(3)</sup>CS can be credited to backup LPSI for DHR in MODES 6 and Defueled per calculation ENG-04223M2, Rev. 0, Addendum 9. If CS is placed in service, no Core Alterations are allowed per Tech Specs.

<sup>(4)</sup>In Modes 5 and 6, <u>IF</u> RCS is vented <u>AND</u> Refuel Pool is less than full (< 35'6"), an Operator must be stationed in the vicinity of the SW/Fire Water Supply valves to the EDG to be ready to take action to shift cooling water to Fire Water if directed by the SM.</li>

DOMINION

Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

	OU-M2-201 – Attach	ment 1 Page 3 of 9				
Section 3 Decay Heat Removal (Continued)						
BEYOND DESIGN BASIS						
Mode 5:		NA for Mode 0				
Steam Generator available for Decay Heat Removal: 1 2 AC Independent Aux Feedwater Pump: TDAFW Pump Ø BDB AFW Pump	OR	<ul> <li>Pressurizer Vent Port Removed</li> <li>BDB AFW Pump Available for RCS Injection</li> </ul>				
Mode 6:	RCS					

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#### DOMINION

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#### Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

OU-M2-201 – Attachment 1 Page 4 of 9

Point Value (1) (0,½, 1) (0,½, 1) (1) (1) (1) (1) ** (1)**		Total       0       1       2       <≥ 3	Condit (Circle) RED ORAN YELLO GREE	) GE
$(1) \\ (0, \frac{1}{2}, 1)^{2} \\ (0, \frac{1}{2}, 1) \\ (1) \\ (1) \\ (1)^{**} \\ (1)^{**} \\ (1)^{**} \\ \end{array}$	* <u>1</u> * <u>1</u>	1 2	RED ORAN YELLO	GE
0, ½, 1) (1) (1) (1)** (1)**		1 2	ORAN YELLC	GE
(1) (1) (1)** (1)**		2	YELLO	
(1) (1)** (1)**				W
(1)** (1)**		<≥3	GREE	
(1)**				
Γ				
Total				
	3			
ev. 0, dem g 2R23. ump as 1 ed as 1 po e <70 °F, CS supp	nonstrates that point for SFPC oint provided th with excursions lying SFP cooli	TRM 3.9.3.3 v c. For 2R23, 0 tat fuel mover a allowed for < ng independe	will be met at $\geq$ Calculation EN nent begins $\geq$ <3 hours, if the ent of Shutdown	17 days IG- 150 hours moving n Cooling.
	(Check)			
A 🗸	В 🗌 ́	C√	Required	
A 🗸	В	C 🗸	Equipment NOT met	RE
A 🗸	В	C 🗸		
y is in the S	SFP during offloa	d, additional ec		
	(Check)			
	. ,			
				ORANG
A 🗌	в С		NOT met	
A 🗌	В			
	e transferr h SFPC p culation E of SFPC e reload, www.0, dem g 2R23. ump as 1 ed as 1 p e <70 °F, CS supp ling with 2 A A A A A A A SDC y is in the s f for Cycle	e transferred to the SFP, h SFPC pumps are one v culation ENG-04223M2, of SFPC provided the re e reload, each available s ev. 0, demonstrates that g 2R23. ump as 1 point for SFPC ed as 1 point provided th le <70 °F, with excursions CS supplying SFP cooli ling with 2-RW-280 oper (Check) A $\square$ B $\square$ A $\square$ B $\square$ A SFPC HX $\square$ B SF A SDC H	e transferred to the SFP, each availabl h SFPC pumps are one viable source culation ENG-04223M2, Rev. 0, Adde of SFPC provided the reactor has bee e reload, each available SFPC pumps g 2R23. ump as 1 point for SFPC. For 2R23, 1 ed as 1 point provided that fuel mover e <70 °F, with excursions allowed for	ump as 1 point for SFPC. For 2R23, Calculation EN         ed as 1 point provided that fuel movement begins ≥ '         ed as 1 point provided that fuel movement begins ≥ '         ed as 1 point provided that fuel movement begins ≥ '         ed as 1 point provided that fuel movement begins ≥ '         ed as 1 point provided that fuel movement begins ≥ '         ed as 1 point provided that fuel movement begins ≥ '         ce <70 °F, with excursions allowed for <3 hours, if the

#### OU-M2-201 REVISION 18 PAGE 26 OF 57

## Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

	OU-M2-201	I – Attachme	nt 1	<b>Page</b> 5 of 9
Section 4 Inventory Control				
RCS Inventory Control				
Check boxes for available equipment:	Point Value	Score	Total	Condition
I I A' HPSI pump	(1)	_1		(Circle)
· B' HPSI pump	(1)	1		
· · C' HPSI pump	(1)	1	0	RED
$\checkmark$ 'A' Charging pump via $\checkmark$ RWST or $\square$ BAST <sup>(1,2)</sup>	$(^{1}/_{2}, 1)^{(3)}$	1/2	1	ORANGE
✓ 'B' Charging pump via ✓ RWST or □ BAST <sup>(1,2)</sup>	$(^{1}/_{2}, 1)^{(3)}$	1/2	2	YELLOW
$\Box$ 'C' Charging pump via $\Box$ RWST or $\Box$ BAST <sup>(1,2)</sup>	$(^{1}/_{2}, 1)^{(3)}$		<≥3	GREEN
RCS Inventory Control not required if DEFUELED AND RCS isolated from SFP by				
one of the following:				
2-RW-280 CLOSED				
OR West SFP Gate INSTALLED				
			_ ΝΔ i	
RCS Inventory Con	rol Total	4		AND
RCS Inventory Con	lioi iolai	4	RCS isc	plated from SFP
Required during RIO (minimum):			-	
One HPSI pump			Require	ed
			Equipme	ent RED
SFP Inventory Control			NOT m	el
Check boxes for available equipment:	Point			<b>a</b>
	Value	Score	Total	Condition
One AFW pump aligned to CST	(1)		(	(Circle)
One Refuel Purification pump	(1)		0	RED
One PMW pump	(1)		1	ORANGE
Makeup available from Fire Protection System	(1)		2	YELLOW
(e.g., hoses)	(1)		<u>≥</u> 3	GREEN
	1			MODE 5, 6, or
SFP Inventory Con	trol Total	N/A		I Pool <u>&gt;</u> 36'4"
Requirements for RCS drain down conditions:	I			
SFP Cooling System vent and drain paths, which could	1			
affect SFP inventory, are identified and safety tagged prior to release of impacting work.	Tagout	Number:		
Controls are in place to ensure safety tags are in place				
during RCS drain down. (1)T.S. 3.1.1.3.b. allows only two charging pumps capable of inject	-			

<sup>(2)</sup>RWST ≥ 57,300 gallons (12%) or BAST > 3,750 gallons (65.8%) to be available per TRM 4.1.2.7a and SP 2601F, "Borated Water Sources Verification, MODE 5 or 6."

<sup>(3)</sup>If ≤ 384 hrs (16 days) since shutdown, at least two Charging pumps with suction from the RWST or BAST and aligned to RCS to be credited as ONE viable makeup source.

#### OU-M2-201 REVISION 18 PAGE 27 OF 57

## Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

OU-M2-2	201 – Attachmer	nt 1	Page 6 of 9
Section 5 Reactivity Control			
Reactivity Control while in MODEs 5 or 6			
Check boxes for available equipment and conditions:	Point Value	Score	Total Condition
RCS <u>AND</u> SFP boron concentrations greater than required by applicable Tech Specs	d (1)	1	<i>(Circle)</i> 0-2 RED
Dilution flowpaths identified (procedurally controlled <u>or</u> tagged) Tagout Number: 2207X99-0007	(1)	1	3 ORANGE
✓ Inventory Flow Paths	(0-2)	2	4 YELLOW 5 GREEN
✓ 'A' HPSI pump			GILLEN
B' HPSI pump			
C' HPSI pump			
$\checkmark$ 'A' Charging pump aligned to $\checkmark$ RWST or $\square$ BAST <sup>(1,2)</sup>			
B' Charging pump aligned to  RWST or  BAST <sup>(1,2)</sup>			
C' Charging pump aligned to RWST or BAST <sup>(1,2)</sup>			
$\boxed{\checkmark} \ge 2$ Source Range Monitor	(1)	_1	
RCS Reactivity Control while in MODE 5 or 6 Total	5	NA	if DEFUELED
Required Equipment (minimum):	(Check)		
$\checkmark$ $\geq$ 2 Source Range MonitorsA $\checkmark$		Z Equ	equired uipment RED
✓ Inventory Flow Paths		NC	DT met
RCS <u>AND</u> SFP Boron concentrations greater than required by applicable Tech Specs			
Reactivity Control while DEFUELED			
Check boxes for available equipment and conditions <b>Point</b> Value	Score	Total	Condition
RCS <u>AND</u> SFP boron concentrations greater than required by applicable Tech Specs (1)		0	(Circle) RED
Dilution flowpaths identified (procedurally		1	YELLOW
controlled or Safety Tagging) (1)		2	GREEN
Tagout Number:2207X99-0007			
	[	1	
RCS Reactivity Control while DEFUELED Total	N/A	NA if	in MODE 5 or 6

<sup>(1)</sup>Only two charging pumps must be capable of injecting based on T.S. 3.1.1.3.b., "Boron Dilution."

<sup>(2)</sup>RWST ≥ 57,300 gallons (12%) or BAST > 3,750 gallons (65.8%) to be available per TRM 4.1.2.7a and SP 2601F, "Borated Water Sources Verification, MODE 5 or 6."

#### OU-M2-201 **REVISION 18 PAGE 28 OF 57**

#### Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

	ties Affecting Containment eet, for status of containment Total Condition (Circle) 0 RED 1 ORANGE 2 YELLOW 2 GREEN
racking She Score	eet, for status of containment          Total       Condition         (Circle)       0         0       RED         1       ORANGE         2       YELLOW
	<i>(Circle)</i> 0 RED 1 ORANGE 2 YELLOW
2	0 RED 1 ORANGE 2 YELLOW
	1 ORANGE 2 YELLOW
1	
1	
4	NA if DEFUELED

<sup>(2)</sup> This item is scored a "1" if no significant fuel failures are indicated by radiochemistry sampling. For the purposes of the SDR

assessment, identification from radiochemistry samples and confirmation from NAF of significant fuel rod/pin failures is necessary

<sup>(4)</sup> This item is scored a "1" if the RCS is intact or a "0" if any RCS opening exists.

<sup>(5)</sup> After 8 days (from the start of the outage), it is assumed that the short-lived, volatile isotopes that are principally responsible for

early health effects have decayed sufficiently such that the event would not contribute to Large Early Release Frequency (LERF). <sup>(6)</sup> No Activities are in progress to preclude mitigation to a fuel handling accident. This item is scored a "1" if Decay Heat Removal, Inventory Control, and Power Availability are <u>NOT</u> Orange/Red. This item is scored a "0" if Decay Heat Removal, Inventory Control, and Power Availability are Orange/Red.

to score this item as "0." <sup>(3)</sup> No Core Alterations in progress in Containment is an indicator of the susceptibility to a fuel handling event. This item is scored a "1" if no Core Alterations are in progress or a "0" if Core Alterations are in progress.

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## Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

	OU-M2-201	- Attachmei	nt 1	Page 8 of 9
Section 7 Power Availability				
<ul> <li>Check boxes for available equipment and conditions:</li> <li>✓ Power Availability</li> <li>✓ Bus 24E aligned to:</li> <li>✓ 24C 24D</li> <li>On-site Power Source:</li> <li>✓ 'A' EDG with 'A' SW pump <u>or</u> 'B' SW pump supplied b the 'A' EDG</li> <li>✓ 'B' EDG with 'C' SW pump <u>or</u> 'B' SW pump supplied b</li> </ul>	(1)	<b>Score</b>	Total (C 0-1 2 3 €4	Condition Fircle) RED ORANGE YELLOW GREEN
the 'B' EDG SBO Diesel via 24E (Time to Boil > 60 min)	(1)			
Off-site Power Source: Unit 2 RSST Unit 2 NSST Unit 3 Ø RSST or NSST via 34A/B Power Source S	(1) (1) (1)	<u>1</u> <u>1</u> <u>4</u>	]	
		4		
Required Equipment:         ✓       One EDG + One Unit 2 Controlled Off-site Source         ✓       IF in RIO at least one additional on site power source:         •       SBO Diesel and Calculated Time to Boil > 60 minu         •       Additional Unit 2 EDG	tes		Required Equipment NOT met	RED
Off-Site GRID Risk Penalty Factor         Environmental Conditions <sup>(1)</sup> Avg sustained wind speed $\geq$ 75 mph         Salt contamination buildup or arcing in the 345 kV swite         OR         Switchyard Activities <sup>(1)</sup> Trip Testing affecting more than one 345 kV line	chyard			
Two 345 kV lines out of service OR ISO-NE/CONVEX Alerts <sup>(1)</sup>				
Abnormal transmission network conditions with potenti- loss of grid <u>OR</u> Planned Maintenance or Projects <sup>(2)</sup>	al for			
SUBTRACT from Power Sub-Total	1) _	( )	Penalty	
Power Availabi	lity Total	4		

<sup>(1)</sup>Apply offsite power source sub-total
<sup>(2)</sup>If 345 kV or main transformer switchyard work is in progress which jeopardizes off-site sources, then deduct points equivalent to the number of offsite sources that could be affected.

## Millstone Unit 2 Shutdown Safety Assessment (SSA) Checklist

	OU-M2-2	01 – Attachment 1	Page 9 of 9
Assessment Completion			
Conflicts between the availability reflected in the outag	e	Conflicts?	Initial
schedule and this checklist have been brought to the attention of the SM.		YES 🗌 / NO 🕅	RO
Remarks:			
Shutdown Safety Assessment (SSA) Checklist Perform	ned	Reactor Operator	
By:		Signature (Licensed C	Operator or STA)
SSA Equipment Status Board(s) / PPC Programs Upd	ated.	RO	
		minak	2
OMOC and Maintenance Rule Coordinator Notification made for <i>unplanned</i> RED or ORANGE.	IS	N/A	
		Initial	S
CR written to address unplanned entries into RED or ORANGE conditions		CR Number:N/A	
The SSA Checklist items have been reviewed and the			
Protected Equipment signs are in place based on SSA		Shift Technical Adviso	
		Signature (Licensed C	Operator or STA)
Shift Manager Review			
		Signatu	ire
Completed SSA Checklist maintained with the Shift			
Turnover Report.		 Initials	5

## Attachment 2 Personnel Designated for Containment Closure

(Sheet 1 of 1)

Date and Time: +2 Days 0000

Department: Maintenance

Beeper No.	Individual Assigned	Hours Available	Assigned Penetration (Penetration Name & No.)
x4576	Scott Getman	1800-0600	#48
		_	
		_	

Operations personnel required?

No No

Approved and sent to Operations:

Guy Blackburn

Applicable Department Manager or designee

*Note:* Operations Department's retention of this Attachment is only required until ALL associated work has been completed or until a new Attachment is provided containing any on-going work previously listed and any new work to be started.



Attachment 3
<b>Closure Plan for Containment Penetration Work Activities</b>

(Sheet 1 of 1)

Penetration #:48			
	Pressure Test Bound	dary Spare	
Closure Plan: Re-bolt flange	e after removing tempora	ary instrumentation	
Estimated Time to Establ	ish Containment Cl	osure (min): 30 minutes	
Estimated Time to Establ	ish Containment Clo	osure (min): <u>30 minutes</u>	
Estimated Time to Establ	ish Containment Clo	osure (min): <u>30 minutes</u>	
Prepared By: Scott Getn	nan	osure (min): <u>30 minutes</u> Department: <u>Main</u>	
	nan		
Job Supervi	nan SOT		
Prepared By: <u>Scott</u> Getm Job Supervi Approved By: Gerry Bake	nan SOT	Department: <u>Main</u>	
Prepared By: <u>Scott</u> Getm Job Supervi Approved By: Gerry Bake	nan ISOT	Department: <u>Main</u>	
Prepared By: <u>Scott</u> Getm Job Supervi Approved By: <u>Gerry</u> Bake Shift Manag	nan ISOT	Department: <u>Main</u>	
Prepared By: <u>Scott</u> Getr Job Supervi Approved By: Gerry Bake	nan ISOT	Department: <u>Main</u>	

### JOB PERFORMANCE MEASURE APPROVAL SHEET

David Jacobs Developer	Revision: 0 06/01/2016 Date
David Jacobs Developer Reviewed: Robert L. Cimmino, Jr.	
Developer Reviewed: Robert L. Cimmino, Jr.	
Developer Reviewed: Robert L. Cimmino, Jr.	
Reviewed: Robert L. Cimmino, Jr.	Date
Robert L. Cimmino, Jr.	
Technical Reviewer	07/12/2016
	Date
Approved:	
Supervisor, Nuclear Training	Date

## **SUMMARY OF CHANGES**

DATE	DESCRIPTION	<b>REV/CHANGE</b>
06/01/2016	Modified from JPM-218 for 2016 ILT NRC Exam	0

Facility: <b>MP2</b>	Examinee	2:			
JPM Number: J	PM-297-R-SRO		evision: 0		
Task Title: AEAS	Broken Boundary Doo	or			
System: Conduc	t of Operations / Fuel H	landling			
Time Critical Task:	🗌 YES 🖂 N	10			
Validated Time (minutes)	: 15				
Task Number(s):	119-01-086				
Applicable To:	SRO <u>X</u> STA	۱ <u> </u>	RO	PEO	
K/A Number: 2	.1.42 K/A I	Rating: 2	2.5 / 3.4		
Method of Testing: Si	imulated Performance:		Actual	Performance:	X
Location: C	lassroom: X	Simula		In-Plant:	
Task Standards:	At the completion of affected and state the			¥ 1	of door
Required Materials: (procedures, equipment, etc.)	MP-PROC-OPS-OP 2 MP-PROC-OPS-OPS		Doors 00] Spent Fuel Ha	ndling Operation	IS
General References:	MP-PROC-OPS-OP 2 MP-PROC-OPS-OPS		Doors 00] Spent Fuel Ha	ndling Operation	IS

#### \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

JPM Nur	JPM Number: JPM-297-R-SRO		Revision :	0
Initial Conditions:		e currently on watch as the Cont ng outage that's in its 10 <sup>th</sup> day wa		
Initiating Cues:	Aco	e Aux Building Watch reports th cess From Aux. bldg. to Railway ges and will not close.		
		cord any procedural actions requ tch's report.	ired to respond to the A	Aux Building
Simulator Requirements:	N/A			

#### \* \* \* \* <u>NOTES TO TASK PERFORMANCE EVALUATOR</u> \* \* \* \*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

## **PERFORMANCE INFORMATION**

	JPM Number: JPM-297-R-SRO	Revision	n: <b>0</b>	
	Task Title: <b>AEAS Broken Bounda</b>	ry Door		
			START TIME: _	
STEP #1	<ul> <li>Performance: Examinee refers to OP 2356 Doors: Section 4.1 Door Class Determination:</li> <li>4.1.1 WHEN any Unit 2 Door is, or will be, <i>not</i> OPERABLE, or <i>not</i> FUNCTIONAL, PERFORM the following:</li> <li>a. OBTAIN the following information for each affected door: <ul> <li>Door ID number and location</li> <li>Nature of inoperability (blocked open, does <i>not</i> latch, etc.)</li> <li>If door is being blocked open, AWO/clearance number/activity</li> <li>If known, expected duration of inoperability</li> </ul> </li> <li>b. SUBMIT a CR.</li> </ul>	Standard: Examinee determines the following: ID Door 205-14-007 AB 14' 6" RR Access Not Operable does not Latch Submits a CR	Critical: Y 🗌 N 🔀	Grade S 🗌 U 🗌
	Comments:			
S T E P # 2	<ul> <li>4.1.2 Refer To Attachment 1, "Unit 2 Door Attributes," and DETERMINE whether affected door is classified as <i>any</i> of the following:</li> <li>Spent Fuel Pool Ventilation Boundary</li> <li>Cue:</li> </ul>	Standard: Examinee refers OP 2356 Doors Attachment 1 page 7 of 15 and notes the following Attributes: • Non TRM Fire Door • SFP Ventilation Boundary (AEAS)	Critical: Y 🗌 N 🔀	Grade S 🗌 U 🗌
	Comments:			

#### **PERFORMANCE INFORMATION**

### JPM Number: JPM-297-R-SRO

Revision: 0

Task Title:AEAS Broken Boundary Door

STEP #3	<ul> <li>Performance:</li> <li>Examinee refers to OP 2356 Doors:</li> <li>Section 4.7 Spent Fuel Pool Ventilation</li> <li>Boundary (AEAS) (A):</li> <li>4.7.1 IF a SFP boundary door <i>cannot</i> be closed and latched, PERFORM the following:</li> <li>a. SUBMIT a CR.</li> <li>b. DEVELOP a closure plan and TRACK as specified in OPS-FH 216 section "Maintaining SFP Boundary Integrity," until door is repaired.</li> </ul>	<ul> <li>Standard:</li> <li>Examinee determines the following for the Door:</li> <li>Will not latch</li> <li>CR is submitted</li> <li>Refers to OPS-FH 216 section 4.10 Maintaining SFP Boundary Integrity</li> </ul>	Critical: Y □ N ⊠	Grade S 🗌 U 🗌
	Cue: Comments:			
STEP #4	Performance: Examinee refers to OPS-FH 216 Section 4.10 Maintaining SFP Boundary Integrity 4.10.2 IF an unplanned breach in the SFP area boundary is identified, PERFORM the following: a. STOP any movement of irradiated fuel or Cask operation in the SFP.	Standard: Examinee directs the Stopping of Fuel Movement in the Spent Fuel Pool Ventilation Boundary Area	Critical: Y ⊠ N □	Grade S 🗌 U 🗌
	Cue:			
	Comments:			

TERMINATION CUE: The evaluation for this JPM is concluded.

**STOP TIME**: \_\_\_\_\_

#### **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-297-R-SRO

Revision: 2
-------------

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		TYes X No	0		
Validated Time (minutes):	15	Actual Time to 0	Complete (minutes):		
Work Practice Performance:		SAT	UNSAT		
Operator Fundamentals:		SAT	UNSAT		
JPM Question Portion Overall [NLO only]:		SAT	UNSAT	[] I	N/A
Attached Question #1		SAT	UNSAT		
Attached Question #2		SAT	UNSAT		
Overall Result of JPM:		SAT	UNSAT		

**Evaluator:** 

Print / Sign

Areas for Improvement / Comments:

## JPM QUESTIONS

Question #1:				
Answer #1:				
<u>Examinee</u> <u>Response</u> :				
Grade:	SAT	UNSAT		

Question #2:		
Answer #2:		
<u>Examinee</u> <u>Response</u> :		
Grade:	SAT	UNSAT

## **STUDENT HANDOUT**

JPM Number:	JPM-297-R-SRO	<u>Revision</u> :	2
Initial Conditions:	You are currently on watch as the Conrefueling outage that's in its $10^{th}$ day	ntrol Room Unit Supervis with a Core Offload in pro	or during a ogress.
Initiating Cues:			
	The Aux Building Watch reports that From Aux. bldg. to Railway Access h not close.		
	Record any procedural actions require Watch's report.	ed to respond to the Aux I	Building
Procedure #	Requirement		

### JOB PERFORMANCE MEASURE APPROVAL SHEET

PM Number:	JPM-294-R-SRO	Revision: 0/1
nitiated:		
	David Jacobs	07/12/2016
	Developer	Date
eviewed:		
	Robert L. Cimmino, Jr.	07/13/2016
	Technical Reviewer	Date
pproved:		
	Supervisor, Nuclear Training	Date

## **SUMMARY OF CHANGES**

DATE	DESCRIPTION	REV/CHANGE	
06/01/2016	New JPM developed for NRC exam 2016	0	
08/18/2016 Incorporated NRC comments		0/1	

Facility: MP2	Exa	aminee:						
JPM Number: J	PM-294-R-SRC	)	Revision: 0/1					
Task Title: <b>TECH</b>	Task Title:     TECH SPEC Evaluation LTOP							
System: Generic Equipment Control								
Time Critical Task:	Time Critical Task: YES X NO							
Validated Time (minutes)	):20							
Task Number(s):	*MP2* 119-(	029-01-02						
Applicable To:	SRO X	STA	RO	PEO				
K/A Number: 2	.2.40	K/A Rating:	3.4 / 4.7					
Method of Testing: S	imulated Perforn	nance:	Actua	al Performance:	X			
Location: C	lassroom:	<u>X</u> Sim	ulator:	In-Plant:				
Task Standards:			th PORVs are NOT ction Statement and					
Required Materials:		S-U2-14-OPS-BA	I					
(procedures, equipment, etc.)	MP-PROC-OP	S-OP 2207[r039]	Plant Cooldov	wn				
General References:		S-U2-14-OPS-BA	· · · · · · · · · · · · · · · · · · ·					
		S-OP 2207[r039] S-ARP 2590B-209	Plant Cooldov Alarm Respon	wn nse C02/03 A-37 L	.T/OP T115			

#### \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

	JPM Number:	JPM-294-R-SRO	Revision :	0/1
Initial Conditio	<u>ns</u> : OP 22 • • •	207 cooldown is in progress with the RCS temperature at 275°F Pressure at 375# P-103-1 and P-1 2 PORVs with "LT/OP SETPOIN 2 Charging pump available to inject	03 NT SELECTOR" in <sup>4</sup>	
Initiating Cues:	D	etermine any required actions for th	e current condition	

Simulator Requirements: N/A

## \* \* \* \* <u>NOTES TO TASK PERFORMANCE EVALUATOR</u> \* \* \* \*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

## **PERFORMANCE INFORMATION**

	JPM Number: JPM	-293-R-SRO	Revision:	0/1	
	Task Title: TEC	H SPEC Evaluati	on LTOP		
			S	TART TIME: _	
STEP #1	Performance: After reviewing the Initial Co determines the applicable acti 2207 Cooldown.		Standard: SRO determines the need to SUSPEND the COOLDOWN OP 2207 Section 4.14 Establishing LTOP Protection Step 4.18.8	Critical: Y 🛛 N 🗌	Grade S 🗌 U 🗌
	Cue: Comments:				
S T E P # 2	Performance: After reviewing the Initial Co determines applicable Tech. S required for the conditions.		<ul> <li>Standard: Technical Specification Actions of 3.4.9.3a action</li> <li>"c" Declare BOTH Channels of LTOP <u>not</u> OPERABLE</li> <li>Perform either of the following: <ul> <li>Depressurize and vent the RCS through a ≥ 2.2 square inch vent within 8 hours.</li> </ul> </li> <li>OR <ul> <li>Raise T<sub>COLD</sub> &gt;275°F (MODE above where PORV's are not required)</li> </ul> </li> </ul>	Critical: Y ⊠ N □	Grade S 🗌 U 🗌
	Cue: Comments:				

TERMINATION CUE: The evaluation for this JPM is concluded.

**STOP TIME**: \_\_\_\_\_

#### **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-294-R-SRO

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🖾 No	)		
Validated Time (minutes):	20	Actual Time to Complete (minutes):			
Work Practice Performance:		SAT	UNSAT		
Operator Fundamentals:		SAT	UNSAT		
JPM Question Portion Overall [ <i>NLO only</i> ]:		SAT	UNSAT		N/A
Attached Question #1		SAT	UNSAT		
Attached Question #2		SAT	UNSAT		
Overall Result of JPM:		SAT	UNSAT		

**Evaluator:** 

Print / Sign

Areas for Improvement / Comments:

## JPM QUESTIONS

Question #1:	What action in the Plant Cooldown cannot be accomplished if P103, "PZR PRES LO RGE" and P103-1, "PZR PRES LO RGE" are not within 30 psi of each other?
Answer #1:	Concurrent RCP and SDC operations
<u>Examinee</u> <u>Response</u> :	
<u>Grade:</u>	SAT UNSAT

Question #2:	In MODE 6 when is Low Temperature Over Pressure protection no longer required?
<u>Answer #2:</u>	Reactor vessel head has been removed or a vent of sufficient size has been established such that RCS pressurization is not possible. (T.S. Basis)
Examinee Response:	
<u>Grade:</u>	SAT UNSAT

## STUDENT HANDOUT

JPM Number:	JPM-294-R-SRO	Revision:	0/1
Initial Conditions:	OP 2207 cooldown is in progress wit	h the following conditions	:
	• RCS temperature at 275°F		
	• Pressure at 375# P-103-1 and	1 P-103	
	• 2 PORVs with "LT/OP SET]	POINT SELECTOR" in "I	HIGH"
	• 2 Charging pump available to	o inject	
	• 1 HPSI pump available to inj	ject	
Initiating Cues:			
	Determine any required actions for the	ne current conditions.	
Examinee			
Response:			

### JOB PERFORMANCE MEASURE APPROVAL SHEET

JPM-296-R-SRO	Revision: 1
David Jacobs	06/02/2016
Developer	Date
Robert L. Cimmino, Jr.	07/06/2016
Technical Reviewer	Date
Supervisor, Nuclear Training	Date
	David Jacobs Developer Robert L. Cimmino, Jr. Technical Reviewer

### **SUMMARY OF CHANGES**

DATE	DESCRIPTION	<b>REV/CHANGE</b>
12/07/10	Created JPM for LOIT 2011 NRC Exam	0/0
06/02/2016	Revised and modified for ILT 2016 NRC Exam	1/0

Facility: MP2 Examinee:					
JPM Number: JPM-296-R-SRO	Revision: <u>1</u>				
Task Title: Radiological Assessment and Task Sup	ervision				
System: Radiation Control 2.3					
Time Critical Task:					
Validated Time (minutes): 20					
Task Number(s):					
Applicable To:   SRO X   STA	RO PEO				
K/A Number: 2.3.4 K/A Rating:	3.3/3.7				
Method of Testing: Simulated Performance:	Actual Performance:				
Location: Classroom: X Sin	mulator: In-Plant:				
Task Standards:At the completion of this JPM, the SRO should analyze the given conditions and designate which PEO should perform each of the two specified tasks, based on the radiological concerns of each.					
Required Materials: • RPM 5.2.2 Basic Radiation	Worker Responsibilities				
General References:•RPM 5.2.2 Basic Radiation•MP-PROC-EP-MP-26-EPI-	Worker Responsibilities FAP09[r004] Radiation Exposure Controls				

#### \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

	JPM Number:	JPM-296-R-SRO	Revision :	1
Initial Conditio		re the Unit Supervisor currently m ng steps in EOP 2532 with the fol	6 6	e LOCA and
	<ul> <li>2-</li> <li>RI</li> <li>PF</li> <li>PF</li> <li>PF</li> <li>PF</li> <li>PF</li> </ul>	ift Manager has declared an SITE RB-30.1A has failed to Close rem BCCW Surge Tank is slowly Risin O #1 current year to date exposur O #2 current year to date exposur O #1 can restore the charging pur O #2 can restore the charging pur O #2 can manually close RB MO O #2 can manually close RB MO	otely ng re is 875 millirem re is 203 millirem np in 44 minutes np in 50 minutes IV in 18 minutes	
	rem/hı Task #	<ul> <li><sup>4</sup>1 restore the "B" charging pumpy</li> <li><sup>5</sup>2 manually close MOV 2-RB-30.</li> <li><sup>7</sup> with general area dose rate of 12</li> </ul>	1A "RBCCW CTMT ]	
Initiating Cues:	event. Deterr Deterr	nine the allowable Exposure for P nine the dose each PEO will recei- nine which PEO will perform the logical requirements for the PEOs	ve for each task. individual tasks based	-

Simulator Requirements: N/A

### \*\*\*\* NOTES TO TASK PERFORMANCE EVALUATOR \*\*\*\*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

## **PERFORMANCE INFORMATION**

	JPM Number:	JPM-296-R-SRO	Revision:	1	
	Task Title:	Radiological Assessme	nt and Task Supervision		
			S	TART TIME: _	
STEP #1	Using Emergency Exp limits and the expected	aditions and Initiating Cue. osure Control Guidance d exposure rate in the area, n dose available for each	<ul> <li>Standard: Using Emergency Exposure Control Guidance at an ALERT level and higher classification dose limits are automatically extended to 4.5 Rem minus their current dose.</li> <li>PEO #1 = 4.5R - 0.875R = 3.625R</li> <li>PEO #2 = 4.5R - 0.203R = 4.297R</li> </ul>	Critical: Y 🛛 N 🗌	Grade S 🗌 U 🗌
STEP #2	Performance: Determine the expecter receive for the stated t times required to accor Cue:	ask, the dose rates and	<ul> <li>Standard:</li> <li>PEO #1 task #1 = 3.667R ± 0.001R</li> <li>PEO #1 task #2 = 3.600R ± 0.001R</li> <li>PEO #2 task #1 = 4.167R ± 0.001R</li> <li>PEO #2 task #2 = 4.000R ± 0.001R</li> </ul>	Critical: Y ⊠ N □	Grade S 🗌 U 🗌
STEP #3	Comments: Performance: Based on available Do must perform each tas		Standard: • PEO #1 task #2 = 3.600R ± 0.001R • PEO #2 task #1 = 4.167R ± 0.001R	Critical: Y 🛛 N 🗌	Grade S 🗌 U 🗌
	Cue: Comments:				

TERMINATION CUE: The evaluation for this JPM is concluded.

**STOP TIME**: \_\_\_\_\_

#### **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-296-R-SRO

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🖾 No	)		
Validated Time (minutes):	20	Actual Time to Complete (minutes):			
Work Practice Performance:		SAT	UNSAT		
Operator Fundamentals:		SAT	UNSAT		
JPM Question Portion Overall [NLO only]:		SAT	UNSAT		N/A
Attached Question #1		SAT	UNSAT		
Attached Question #2		SAT	UNSAT		
Overall Result of JPM:		SAT	UNSAT		

Evaluator:

Print / Sign

Areas for Improvement / Comments:

## JPM QUESTIONS

Question #1:				
Answer #1:				
<u>Examinee</u> <u>Response</u> :				
Grade:	SAT	UNSAT		

Question #2:		
Answer #2:		
<u>Examinee</u> <u>Response</u> :		
Grade:	SAT	UNSAT

## STUDENT HANDOUT

JPM Number:	JPM-2	96-R-SRO	Revisio	<u>on: 1</u>
Initial Conditions:	You are the Unit Supervisor currently mitigating a medium size LOCA and directing steps in EOP 2532 with the following conditions:			
	• Shift Manager has declared an SITE AREA CHARLIE 2			
	• 2-RB-30.1A has failed to Close remotely			
	RBCCW Surge Tank is slowly Rising			
	• PEO #1 current year to date exposure is 875 millirem			
	• PEO #2 current year to date exposure is 203 millirem			
	• PEO #1 can restore the charging pump in 44 minutes			
	• PEO #2 can restore the charging pump in 50 minutes			
	• PEO #1 can manually close RB MOV in 18 minutes			
	• PEO #2 can manually close RB MOV in 20 minutes			
	Task #1 restore the "B" charging pump with general area dose rate of 5 rem/hr.			
		Task #2 manually close MOV 2-RB-30.1A "RBCCW CTMT ISOL HDR A SPLY" with general area dose rate of 12 rem/hr.		
Initiating Cues:				
	Determine the event.	ne allowable Exposure	e for PEO #1 and PEC	#2 for the given
	Determine the dose each PEO will receive for each task.			
		which PEO will perfor	m the individual tasks PEOs.	based on the
	Exposure Limit	Task #1 Dose	Task #2 Dose	Task to perform

	Exposure Limit	Task #1 Dose	Task #2 Dose	Task to perform
PEO #1				
PEO #2				

### JOB PERFORMANCE MEASURE APPROVAL SHEET

	Emergency Classification	
PM Number:	JPM-298-R-SRO	Revision: 0
nitiated:		
	Robert Royce	6 /14/2016
	Developer	Date
eviewed:		
	David Jacobs	07/05/2016
	Technical Reviewer	Date
pproved:		
	Supervisor, Nuclear Training	Date

## **SUMMARY OF CHANGES**

DATE	DESCRIPTION	<b>REV/CHANGE</b>

Facility:   Millstone 2   Examinee:
JPM Number: JPM-298-R-SRO Revision: 0
Task Title:   Emergency Classification
System:
Time Critical Task: X YES NO
Validated Time (minutes): 10
Task Number(s):
Applicable To:   SRO   X   RO
K/A Number: <u>GEN.2.4.41</u> K/A Rating: <u>2.9 / 4.6</u>
Method of Testing: Simulated Performance: X Actual Performance:
Location:     Classroom:     X     Simulator:     In-Plant:
Task Standards:At the completion of this JPM, the examinee has correctly classified the proposed event and provided the appropriate Protective Action Recommendation.
Required Materials: (procedures, equipment, etc.)MP-26-EPI-FAP06-002, Millstone Unit 2 Emergency Action Levels MP-26-EPI-FAP06-005, Control Room Protective Action Recommendations. MP-26-EPI-FAP06, Classification and PARs
<ul> <li><u>General References</u>:</li> <li>MP-26-EPI-FAP06-002, Millstone Unit 2 Emergency Action Levels</li> <li>MP-26-EPI-FAP06-005, Control Room Protective Action Recommendations.</li> </ul>

#### \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

JPM Number:	JPM-298-R-SRO	Revision :0
Initial Conditions:	• The plant is operating at 100% power.	
	• Bus 24E is aligned to Bus 24C.	
	<ul> <li>The "B" LPSI Pump is OOS.</li> <li>Wind at the site is from 15 degrees at 3</li> </ul>	mnh
	• Wind at the site is from 15 degrees at 3	, mpn.
Time = 0 Minutes:	The following sequence of events occurs:	
	• The Reactor trips,	
	• The Turbine trips,	
	• The crew enters EOP 2525, <i>Standard P</i>	Post Trip Actions.
Time + 17 Minutes:	• EOP 2525 is complete	
	• The BOP reports secondary conditions	as follows:
	• Buses 25A/B, 24A/B, and 24C de-e	
	• Bus 24D energized by the 'B' D/G	C
	• S/G press: #1 is 745 psia, #2 is 740	psia, both slowly lowering
	• $T_{hot}$ is 289°F, $T_{cold}$ is 262°F, both slo	
	• S/G levels: #1 is 18%, #2 is 16%, bo	oth rising slowly
	• "B" AFP supplying both S/Gs	
Time + 19 Minutes:	• RO reports primary conditions as follo	ws:
	• Pressurizer level is 0%	
	• Reactor vessel level (RVLMS) is 09 inoperable)	%. (Both #8 string HJTCs are
	• Pressurizer pressure is 53 psia and s	lowly lowering
	• CETS are 847°F and slowly rising	
	• Subcooling (CET) indicates -478°F	
	• Facility 2 SIAS, CIAS, EBFS have	
	Isolation Valves, CH-515, 516, and	089, indicate open and will
	NOT close from C-02	
	• CTMT pressure is 42 psig, rising slo	owly
	• CTMT temperature is not available	

Time + 22 Minutes:	<ul> <li>STA reports the following:</li> <li>Main Steam Line RM-4299A and B indicate 1.6 R/hr, RM-4299C indicates 1.8 R/hr, all rising</li> <li>CTMT Hi Range, RM-8240 / 8241 is 20,000R/hr / 21,000 R/hr, both rising</li> <li>CTMT Personnel Access Area, RM-7890, off scale high</li> <li>Facility 2 CTMT atmosphere, RM-8262A/B show pre CIAS spikes and alarm</li> <li>The Kaman Rad Monitor, RM-8168 is reading 5E+02µCi/cc, rising, and in ALARM.</li> <li>All other RMs outside CTMT are elevated, but NOT in alarm</li> <li>Main Steam Line RM-4299A/B were reading 0.7 R/hr, RM-4299C was reading 0.9 R/hr, 15 minutes ago</li> <li>CTMT Hi Range, RM-8240 / 8241 were reading 350R/hr / 370 R/hr, 15 minutes ago</li> <li>All other Rad Monitor, RM-8168 was reading 1.2E+01µCi/cc 15 minutes ago</li> <li>All other Rad Monitor, RM-8168 was reading 1.2E+01µCi/cc 15 minutes ago</li> </ul>
Time + 25 Minutes:	Crew transitions to EOP 2532, Loss of Coolant Accident
Initiating Cues:	You are the on-duty SM.
	Your task is to determine the NRC and state posture code classification for this

## \*\*\* TIME CRITICAL JPM\*\*\*

### \*\*\*\* NOTES TO TASK PERFORMANCE EVALUATOR \*\*\*\*

event, and as required, provide any additional recommendations.

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

JPM Number:	JPM-298-R-SRO	

Revision: 0

Task Title: Emergency Classification

START TIME: \_\_\_\_\_

STEP	Performance:	Standard:	Critical:	Grade
#1	Obtain Millstone 2 Emergency Action Levels,	The examinee reads the student Handout and	Y[]N[X]	S[]U[]
	MP -26-EPI-FAP06-002.	obtains Millstone 2 Emergency Action Levels,		
		MP -26-EPI-FAP06-002.		
		ency Action Levels, MP-26-EPI-FAP06-002 and MP	-26-EPI-FAP06, 0	Classification
		with EALs may be provided at the start of the JPM.		
	Comments: The 15 minute clock starts when the ends when the classification is made.	examinee obtains Millstone 2 Emergency Action Level	s, MP -26-EPI-FA	AP06-002, and
STEP	Performance:	Standard:	Critical:	Grade
#2	Classify the event as a GENERAL	Using the barrier reference table, examinee	Y[X] N[]	S[]U[]
	EMERGENCY, State Posture Code, ALPHA,	determines the event as a GENERAL		
	within 15 minutes, based on Barrier Failure,	EMERGENCY, State Posture Code, ALPHA,		
	BG1, any three barriers failed.	based on;		
		• Fuel Clad Barrier failed FCB3 (L)		
		• RCS Barrier failed RCB2 (L),		
		• CTMT Barrier, CNB3(P) or CNB4 (P)		
		Could also arrive at the same classification from		
		Off Site Releases OG1 or In-Plant Radiation RG1.		
		• OG1 MP2 Kaman Vent Monitor (RM-8168)		
		reading $\geq 2\mu$ ci/cc for > 15 minutes. Current		
		reading 5E +02 $\mu$ ci/cc (500 $\mu$ ci/cc) and was 1.2		
		E+01 $\mu$ ci/cc (12 $\mu$ ci/cc) 15 minutes ago.		
		• RG1 RM-8240/8241 reading > 1,200 R/hr.		
		Currently reading 20,000 R/hr and 21,000 R/hr.		
	Cue:			
	Comments:			

## JPM Number: JPM-298-R-SRO

Revision: 0

## Task Title:Emergency Classification

STEP #3	<ul> <li>Performance:</li> <li>For Control Room PARs, Refer to EPI-FAP06-005, "Control Room Protective Action recommendations" and determines the PAR recommendation as;</li> <li>Evacuate Zones A and B and Plum Island.</li> <li>Shelter all other zones.</li> </ul>	<ul> <li>Standard:</li> <li>Using the Control Room PAR Process Flowchart, determine that the present wind direction (between 340° - 029°) requires the examinee to recommend an evacuation of Zones A, B, and Plum Island, and to shelter all other zones.</li> <li>General Emergency – Yes</li> <li>General Emergency – Alpha – Yes</li> <li>Rapidly Progressing Event – No (Clad &lt;1200°F)</li> <li>Does CTMT Radiation Exceed Table 1 Values – Yes (20,000 and 21,000 R/hr are &gt;19,000R)</li> <li>GE-ALPHA PAR</li> <li>Evacuate 5 mile radius</li> <li>Evacuate 10 miles downwind: Sector 340-029: Zones to evacuate to 10 miles; A and B and Plum Island.</li> <li>Shelter all other zones.</li> </ul>	Critical: Y [ X ] N [ ]	Grade S [ ] U [ ]
		06-05, Control Room Protective Action Recommendation bre, issuance of KI tablets should not be recommended.	ons.	

TERMINATION CUE: The evaluation for this JPM is concluded.

**STOP TIME**: \_\_\_\_\_

## **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-298-R-SRO

Date Performed:

Candidate:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		Yes 🗌 No		
Validated Time (minutes):	15 minutes for EAL PAR within 15 minutes after Classification	Actual Time to C	Complete (minutes):	
Overall Result of JPM:		SAT	UNSAT	

Evaluator:

Print / Sign

Areas for Improvement / Comments:

## JPM HANDOUT

JPM Number:	JPM-298-R-SRO	Revision:	0
Initial Conditions:	<ul> <li>The plant is operating at 100% power</li> <li>Bus 24E is aligned to Bus 24C.</li> <li>The "B" LPSI Pump is OOS.</li> <li>Wind at the site is from 15 degrees at</li> </ul>		
Time = 0 Minutes:	<ul> <li>The following sequence of events occurs:</li> <li>The Reactor trips,</li> <li>The Turbine trips,</li> <li>The crew enters EOP 2525, <i>Standard</i></li> </ul>	Post Trip Actions.	
Time + 17 Minutes:	<ul> <li>EOP 2525 is complete</li> <li>The BOP reports secondary condition</li> <li>Buses 25A/B, 24A/B, and 24C de-</li> <li>Bus 24D energized by the 'B' D/G</li> <li>S/G press: #1 is 745 psia, #2 is 740</li> <li>T<sub>hot</sub> is 289°F, T<sub>cold</sub> is 262°F, both si</li> <li>S/G levels: #1 is 18%, #2 is 16%, #</li> <li>"B" AFP supplying both S/Gs</li> </ul>	energized ) psia, both slowly lowe lowly lowering	ring
Time + 19 Minutes:	<ul> <li>RO reports primary conditions as following the pressurizer level is 0%</li> <li>Reactor vessel level (RVLMS) is 0 inoperable)</li> <li>Pressurizer pressure is 53 psia and</li> <li>CETS are 847°F and slowly rising</li> <li>Subcooling (CET) indicates -478°F</li> <li>Facility 2 SIAS, CIAS, EBFS have Isolation Valves, CH-515, 516, and NOT close from C-02</li> <li>CTMT pressure is 42 psig, rising s</li> <li>CTMT temperature is not available</li> </ul>	9%. (Both #8 string HJT slowly lowering F and becoming more ne actuated; however, Let 1 089, indicate open and lowly	egative down

Time + 22 Minutes:	<ul> <li>STA reports the following:</li> <li>Main Steam Line RM-4299A and B indicate 1.6 R/hr, RM-4299C indicates 1.8 R/hr, all rising</li> <li>CTMT Hi Range, RM-8240 / 8241 is 20,000R/hr / 21,000 R/hr, both rising</li> <li>CTMT Personnel Access Area, RM-7890, off scale high</li> <li>Facility 2 CTMT atmosphere, RM-8262A/B show pre CIAS spikes and alarm</li> <li>The Kaman Rad Monitor, RM-8168 is reading 5E+02µCi/cc, rising, and in ALARM.</li> <li>All other RMs outside CTMT are elevated, but NOT in alarm</li> <li>Main Steam Line RM-4299A/B were reading 0.7 R/hr, RM-4299C was reading 0.9 R/hr, 15 minutes ago</li> <li>CTMT Hi Range, RM-8240 / 8241 were reading 350R/hr / 370 R/hr, 15 minutes ago</li> <li>All other Rad Monitor, RM-8168 was reading 1.2E+01µCi/cc 15 minutes ago</li> <li>All other Rad Monitor, RM-8168 was reading 1.2E+01µCi/cc 15 minutes ago</li> </ul>
Time + 25 Minutes:	Crew transitions to EOP 2532, Loss of Coolant Accident
Initiating Cues:	You are the on-duty SM.
	Your task is to determine the NRC and state posture code classification for this event, and as required, provide any additional recommendations.
	<u>*** TIME CRITICAL JPM***</u>

## JOB PERFORMANCE MEASURE APPROVAL SHEET

IPM Number:	JPM-270	Revision:	0/1
n winder.	J1W-270		0/1
Initiated:			
	David J. Jacobs	07/1	5/2014
	Developer	I	Date
Reviewed:			
	Lenny E. Mausteller	07/1	5/2014
	Technical Reviewer	Date	
Approved:			
	Mike J. Cote	07/1	7/2014
	Supervisor, Nuclear Training	Г	Date

## **SUMMARY OF CHANGES**

DATE	DESCRIPTION	<b>REV/CHANGE</b>
07/15/2014	New JPM for 2013-2014 ILT Audit Exam	0
08/16/2016	Incorporate NRC comments	0/1

		JPM WORKS	SHEET			
Facility: Millstone U	nit 2 Ex	aminee:				
JPM Number:	JPM-270		Revision:	0/1	-	
Task Title: LOCA	Cooldown "A" S	Steam Dump lose	Vacuum go	to ADVs		
System: <b>039 Ma</b>	ain and Reheat	Steam System			-	
Time Critical Task:	() YES	( <b>X</b> ) NO				
Validated Time (minutes	):30					
Task Number(s):	599-05	-011	-			
Applicable To:	SRO X	STA	RO	X	PEO	
K/A Number:	42.01	K/A Rating:	3.1/3.2			
Method of Testing: S	imulated Perform	nance:	_	Actual	Performance:	X
Location: C	Classroom:	Sin	mulator:	X	In-Plant:	
Task Standards:	Condenser Va		red Cooling	down for	recognized a loss the "A" Steam Du	
Required Materials: (procedures, equipment, etc.)	MP-PROC-OF	PS-EOP 2532				
General References:	Ensure S/G lev	vels > 50%				

#### \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

	JPM WORKSHEET		
JPM Num	ber: JPM-270	Revision :	0/1
Initial Conditions:	The Plant was manually tripped du Accident.	e to a Loss of Primary	Coolant
	All actions from EOP 2525 SPTA Loss of Coolant Accident up to Ste	-	and EOP 2532
Initiating Cues:	You are directed to commence con with Step 17 of EOP 2532 LOCA	trolled cool down In A	Accordance
Simulator Requirements:	Reset to IC 290 password "2013>lo	oit"	
	Insert Malfunction RC04 at 500 gp	m	
	Trigger #1 Malfunction FW33 at 7	.5 BP@ 10 mins	

## \* \* \* \* <u>NOTES TO TASK PERFORMANCE EVALUATOR</u> \* \* \* \*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

	JPM Number: JPM-270	Revisi	on: <b>0/1</b>	
	Task Title:   LOCA Cooldown "A" Ste	am Dump lose Vacuum go to ADVs	START TIME:	
STEP #1	<ul> <li>Performance:</li> <li>NOTE</li> <li>1. RCS cooldown should be initiated within one hour after the event to conserve condensate inventory and comply with the Long Term Cooling Analysis.</li> <li>2. RCS cooldown rate greater than 40_F/hr should be maintained until the steam dump/bypass valves or atmospheric dump valves are full open.</li> <li>3. The starting point for the RCS cooldown should be the TC or CET temperatures where RCS has stabilized.</li> <li>4. TC should be used for monitoring RCS cooldown if in forced or natural circulation. CETs should be used for all other cases.</li> </ul>	Standard: Examinee reads NOTE and understands the requirements for RCS Cooldown	Critical: Y [ ] N [ <b>X</b> ]	Grade S[]U[]
	Cue: Comments:			

STEP	Performance:	Standard:	Critical:	Grade				
#2	NOTE Technical Specification cooldown rates should be observed during the cooldown. The cooldown rates are as follows: 1. RCS TC greater than 220 <sup>0</sup> F the cooldown rate is 100 <sup>0</sup> F/hr. 2. RCS TC less than or equal to 220 <sup>0</sup> F the cooldown rate is 50 <sup>0</sup> F/hr. Cue:	Examinee reads and complies with Tech. Spec. Limits	Y[] N[ <b>X</b> ]	S[]U[]				
	Comments:							
STEP #3	Performance: <b>Perform Controlled Cooldown</b> *17.INITIATE a controlled cooldown using the steam dumps to establish shutdown cooling entry conditions.	Standard: Examinee Locates and Places "A" Steam Dump to Manual, increases the Output approximately by 10% then monitors Cooldown Rate.	Critical: Y [ <b>X</b> ] N [ ]	Grade S [ ] U [ ]				
	Cue:	In rate > $40^{\circ}$ F per hour and less then TS limit INSERT 1	Malfunction FW3	33 at 7.5 BP@				
S T E P # 4	Performance: C06/07 A-37 "COND VACUUM LO" alarms, the Examinee refers to the Alarm Response 2590E-185 Confirms loss of the Main Condenser.	Standard: Examinee uses diverse indications to confirm a loss of the Main Condenser. Examinee understands that the "A" Steam Dump Valve will close when Condenser Vacuum rises to 10"	Critical: Y [ ] N [ <b>X</b> ]	Grade S [ ] U [ ]				
	Cue:							
		Comments:						

S T E P #5	Performance: <b>Perform Controlled Cooldown</b> *17.1 INITIATE a controlled cooldown using the ADVs to establish shutdown cooling entry conditions.	Standard: Examinee refers back to EOP 2532 Loss of Coolant Accident Step 17.1 Contingency Actions and transfers Cooldown to the ADVs	Critical: Y [X] N [ ]	Grade S [ ] U [ ]	
	Cue: Comments:				

TERMINATION CUE: The evaluation for this JPM is concluded.

STOP TIME:	
------------	--

## **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-270

Revision: 0/1

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade. As necessary, refer to TIG-04 for additional Pass/Fail criteria.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🖾 No	)		
Validated Time (minutes):	30	Actual Time to Complete (minutes):			
Work Practice Performance:		SAT	UNSAT		
Operator Fundamentals:		SAT	UNSAT		
JPM Question Portion Overall [NLO only]:		SAT	UNSAT	1	N/A
Attached Question #1		SAT	UNSAT		
Attached Question #2		SAT	UNSAT		
Overall Result of JPM:		SAT	UNSAT		

Evaluator:

Print / Sign

Areas for Improvement / Comments:

# JPM QUESTIONS

Question #1:		
Answer #1:		
<u>Examinee</u> <u>Response</u> :		
Grade:	SAT	UNSAT

Question #2:				
Answer #2:				
Examinee <u>Response</u> :				
Grade:	SAT	UNSAT		

# STUDENT HANDOUT

JPM Number:	JPM-270	Revision:	0/1
Initial Conditions:	The Plant was manually tripped due to a Lo Accident.	oss of Primary Coo	olant
	All actions from EOP 2525 SPTA have bee Loss of Coolant Accident up to Step #17	en completed and l	EOP 2532
Initiating Cues:	You are directed to commence controlled c Step 17 of EOP 2532 LOCA	ool down In Acco	rdance with

## JOB PERFORMANCE MEASURE APPROVAL SHEET

PM Title:	Respond to a 10 Step CEA Misalignmen	ι	
PM Number:	JPM-284-S-1	Revision:	0/1
nitiated:			
	Robert L. Cimmino, Jr.	03/	15/2016
	Developer		Date
Reviewed:			
	David J. Jacobs	07/0	)5/2016
	Technical Reviewer	]	Date
Approved:			
	Supervisor, Nuclear Training	]	Date

## **SUMMARY OF CHANGES**

DATE	DESCRIPTION	REV/CHANGE
03/15/2016	New JPM for 2015-16 NRC License Exam	0
08/15/2016	Incorporated NRC comments	0/1

#### JPM WORKSHEET

Facility: Millstone Un	nit 2 E	xaminee:				
JPM Number:	JPM-284-S-1		Revision:	0/0	-	
Task Title: Respon	d to a 10 Step C	EA Misalignme	nt			
System: CEDS					-	
Time Critical Task:	YES	NO NO				
Validated Time (minutes)	):20					
Task Number(s):	000-04-	097	-			
Applicable To:	SRO	STA	RO	X	PEO	
K/A Number: 001	/A2.03	K/A Rating:	3.5/4.2			
Method of Testing: Si	imulated Perform	ance:	_	Actual	Performance:	X
Location: C	lassroom:	Si	mulator:	X	In-Plant:	
Task Standards:	recognize CEA	<b>1</b>	as slipped 10	steps to 1	start the downpov 70 steps withdrav ith the group.	
Required Materials: (procedures, equipment, etc.)	MP-PROC-OPS	S-AOP 2575, Raj S-ARP 2590C-11 S-ARP 2590C-14 S-OP 2302A, Co	1, ACTM T 0, CEA GP	ROUBLE DEV BK/	'UP	
General References:		S-OP 2302A, Co for ~ 30%/hr po		•	ystem	

## \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

## JPM WORKSHEET

JPM Nu	mber:	JPM-284-S-1	Revision :	0/1
55%. The c		nt is stable at 100 % power, abo v has entered AOP 2575, Rapic orcing Pressurizer Sprays.		•
	The BOI	P is standing by to reduce turbin	ne load.	
Initiating Cues:	You are Insert G	the RO. Froup 7 rods 10 steps in accorda	ance with step 3.4 of A	OP 2575.
Simulator Requirements:	Insert M given an {Conside	ower, steady state, ARO, forcin alfunction RD0301 (10) to slip insert command. er creating a Boolean trigger ba <b>POS(01) &lt; 180</b> ]}	CEA #1 10 steps whe	

## \* \* \* \* <u>NOTES TO TASK PERFORMANCE EVALUATOR</u> \* \* \* \*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

	JPM Number:	JPM-284-S-1	Revi	ision:	0/1	
	Task Title:	Respond to a 10 Step CE	A Misalignment			
				S	TART TIME: _	
STEP #1	Performance: <u>AOP 2575</u> CAUTION In the case of a dropped used to initiate downpow	CEA, rod motion is <i>not</i> wer.	<ul> <li>Standard:</li> <li>Examinee obtains a copy of AOP 2575.</li> <li>Examinee turns to the Caution stater proceeding step 3.4 on page 8.</li> <li>Examinee reads and acknowledges Caution.</li> </ul>		Critical: Y □ N ⊠	Grade S 🗌 U 🗌
	Cue: Provide the Examinee w Comments:	rith a marked up copy of A	OP 2575 "RAPID DOWNPOWER"			
S T E P # 2	AND Reactor pow	ring due to a dropped rod, ver is greater than 99% SERT Group 7 CEAs 10 e downpower.	<ul><li>Standard:</li><li>CEA are inserted</li></ul>		Critical: Y 🛛 N 🗌	Grade S 🗌 U 🗌
	Cue: Ensure malfunction	n <b>RD0301</b> triggers (causing	g CEA #1 to slip to 170 steps) when Group 7 (	CEAs be	gin to insert.	
		or CEA insertion as directe	0 10 steps which would require entering AOP d by an AOP are considered "Skill Of The Cr			
S T E P # 3	-		<ul> <li>Standard:</li> <li>Examinee performs the following:</li> <li>CEA control switch released.</li> <li>C-04 alarms acknowledged.</li> <li>US notified of CEA misalignment.</li> </ul>		Critical: Y □ N ⊠	Grade S 🗌 U 🗌
	Cue:	for to AOD 255(1)				
	Comments: Examinee m	hay refer to AOP 2556 but	does not meet entry conditions.			

JPM Number:

Revision: 0/1

Task Title: Respond to a 10 Step CEA Misalignment

JPM-284-S-1

STEP	Performance:	Standard:	Critical:	Grade		
#4	ARP 2590C-136 CEA Motion Inhibit (BA-18)	Examinee references ARP 2590C-136 and notes	Y 🗌 N 🖂	S 🗌 U 🗌		
	1. VERIFY CEA motion has stopped (C-04,	the need to reference additional ARPs pertaining to				
	PPC).	the actual cause of the alarm.				
	2. To determine cause of alarm, OBSERVE any					
	associated CEDS annunciators lit (C-04).					
	3. IF other associated CEDS annunciators are					
	lit, Refer To applicable alarm response					
	section and PERFORM necessary corrective					
	actions.					
	4. IF necessary, SUBMIT Trouble Report to					
	I&C Department.					
	Cue: If Examinee expresses the need to have I&C and/or a PEO investigate CEA #1, state that you have already made the calls and both					
	are enroute to the East DC switchgear room.					
	Acknowledge any suggestion to submit a Trouble Report and state another operator will submit one.					
	Comments: Examinee may not reference this ARP as the cause is known based on C-04 annunciator DA-18 and the actions are to simply					
	verify CEA motion has stopped and then reference	the applicable ARP that caused the CMI.				
		**				

JPM Number:

JPM-284-S-1

Revision: **0/1** 

Task Title: Respond to a 10 Step CEA Misalignment

STEP	Performance:	Standard:	Critical:	Grade
# 5	ARP 2590C-111 ACTM Trouble (AB-15)	Examinee references ARP 2590C-136 and notes	Y 🗌 N 🔀	S 🗌 U 🗌
	1. STOP all CEA motion.	the need to reference additional ARPs pertaining to		
	2. REQUEST I&C Department investigate	the actual cause of the alarm.		
	ACTM trouble condition.			
	3. IF alarm is momentary, PERFORM the			
	following:			
	3.1. MONITOR all ACTMs and REPORT			
	all abnormal red lights (East DC			
	switchgear room).			
	3.2. OPEN the "ACTM TROUBLE			
	ALARMS" display on CEAPDS.			
	3.3. RESET ACTMs.			
	3.4. IF ACTM alarm on CEAPDS display			
	clears (is not flashing or steady red),			
	CEA motion may be resumed.			
	3.5. IF ACTM alarm on CEAPDS display			
	does not clear, NOTIFY SM.			
		and/or a PEO investigate CEA #1, state that you have a	already made the	calls and both
	are enroute to the East DC switchgear room.			
	Comments: Examinee may not reference this ARP	as the cause is known based on the other C-04 annun	ciators, the alarm	n is
	momentary and the actions are not critical at thi	s time.		

JPM Number:

JPM-284-S-1

Revision: **0/1** 

Task Title: Respond to a 10 Step CEA Misalignment

CTED	Doutourson ool	Standard	Critical	Crada		
STEP #6	<ul> <li>Performance: <u>ARP 2590C-140</u> (DA-18)</li> <li>1. IDENTIFY misaligned CEA and DETERMINE actual steps misaligned.</li> <li>2. IF deviation is greater than 10 steps, Go To AOP 2556, "CEA Malfunctions."</li> <li>3. IF deviation is less than or equal to 10 steps, Refer To OP 2302A, "Control Element Drive System" and PERFORM applicable actions to align all CEAs in affected group to within one step of each other, using manual individual mode in conjunction with bypassing CMI.</li> <li>4. IF necessary, RESET affected group CEA positions on PPC as follows: 4.1. SELECT "CEA POSITION" on PPC. 4.2. SELECT "CEA POSITION EDITOR." 4.3. PERFORM directions as indicated on</li> </ul>	<ol> <li>Standard:</li> <li>Identifies CEA #1 as being misaligned with Group 7 by (less than or equal to) 10 steps.</li> <li>Informs US of the need to realign CEA #1 with Group 7 using OP 2302A (based on magnitude of CEA misalignment).</li> </ol>	Critical: Y ⊠ N □	Grade S 🗌 U 🗍		
	are enroute to the East DC switchgear room. When	&C and/or a PEO investigate CEA #1, state that you have already made the calls and both When later questioned by the Examinee about the status of CEA #1, state that I&C is on sed the cause of the slipped CEA. I&C and a PEO will remain on station in the East DC T the CEDS during subsequent CEA motion.				
	Comments:					

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	Task Title: Respond	to a 10 Step CEA Misalignment			
S T E P # 7	Performance: <u>OP 2302A</u> <u>Section 4.2</u> CEA Operation in Ma Individual Mode		opy of OP 2302A, turns to d acknowledges the Note.	Critical: Y □ N ⊠	Grade S 🗌 U 🗌
	<b>NOTE:</b> When operating in this mode, any CEA in any group can be moved u control switch and is primarily use CEAs and testing.	sing CEA			
	Cue:				
	Comments:				
STEP #8	Performance: 4.2.1 IF at any time it is necess CMI during CEA movem Section 4.5 PERFORM a actions in conjunction wi section.	pplicable Section 4.5 of OP 20.	acknowledges the need to use 32A to realign CEA #1.	Critical: Y □ N ⊠	Grade S 🗌 U 🗌
	Cue:				
	Comments:				

JPM Number:

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Task Title: Respond to a 10 Step CEA Misalignment

JPM-284-S-1

Grade S 🗌 U 🗌						
S L U L						
Cue: When asked, state that Examinee has permission to bypass the CMI for the purpose of realigning CEA #1 with Group 7 and that all						
applicable Tech Spec have been entered.						
Comments: Examinee may review Section 4.5 up to step 4.5.7, or in its entirety to be prepared to utilize it in concert with Section 4.2						
he CMI are						
h						

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Task Title: Respond to a 10 Step CEA Misalignment

STED	Performance	Standard	Critical	Grade
STEP #10	<ul> <li>Performance:</li> <li>4.5.1 REQUEST SM/US authorization to bypass CMI.</li> <li>4.5.2 CHECK annunciator C-04, window BA-18, "CEA MOTION INHIBIT," lit.</li> <li>4.5.3 Refer To the following and LOG entry in Shift Turnover Log (CMI bypassed): <ul> <li>TSAS 3.1.3.1, ACTION b</li> <li>TSAS 3.1.3.1, ACTION c</li> <li>TSAS 3.1.3.6, ACTION d</li> </ul> </li> <li>4.5.4 PRESS appropriate group "INHIBIT BYPASS" pushbutton and CHECK the following: <ul> <li>Appropriate group red "INHIBIT BYPASS" pushbutton, lit</li> <li>Annunciator C-04, window BA-19, "CEA MOTION INHIBIT BYP," lit</li> </ul> </li> <li>4.5.5 PRESS and HOLD system "CEA MOTION INHIBIT BYPASS" pushbutton.</li> <li>4.5.6 CHECK system red "CEA MOTION INHIBIT BYPASS," lit.</li> </ul>	<ul> <li>Standard:</li> <li>1. "Inhibit Bypass" pushbutton for Group 7 is pressed and verified lit, and annunciator C-04/BA-19 alarm is acknowledged.</li> <li>2. "CEA Motion Inhibit Bypass" button is pressed and held, button verified lit.</li> <li>Note: When the CMI for Group 7 is bypassed, the CEA MOTION INHIBIT annunciator will "reset". This is a quirk in the system design and totally expected. The Examinee may or may not explain.</li> </ul>	Critical: Y ⊠ N □	Grade S 🗌 U 🗍
	Cue: If not already done, state that Examinee has p that all applicable Tech Spec have been entered.	bermission to bypass the CMI for the purpose of realign	ning CEA #1 with	Group 7 and
		hally address the applicable TSAS that are affected what address them and the RO is to focus on correcting the second sec		

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Task Title: Respond to a 10 Step CEA Misalignment

STEP #11	<ul> <li>Performance:</li> <li>4.5.7 Using applicable Section and PERFORM necessary actions to operate CEA(s): <ul> <li>IF operating in Manual Individual mode, Section 4.2</li> <li>IF operating in Manual Group mode, Section 4.3</li> <li>IF operating in Manual Sequential mode, Section 4.4</li> </ul> </li> <li>NOTE <ul> <li>CMI should remain bypassed for at least three seconds after CEA motion is stopped to allow CPP operations to be completed.</li> <li>4.5.8 WHEN CEA motion has been stopped for at least three seconds, RELEASE system "CEA MOTION INHIBIT BYPASS"</li> </ul> </li> </ul>	<ul> <li>Standard:</li> <li>Based on the guidance of step 4.5.7, Examinee returns to Section 4.2 of OP 2302A, if not being used in concert with Section 4.5.</li> <li>Examinee may review the Note proceeding step 4.5.8 at this time, or wait until returning to this section of the procedure after the CEA is realigned.</li> </ul>	Critical: Y □ N ⊠	Grade S 🗌 U 🗌
	CEA MOTION INHIBIT BYPASS" pushbutton.			

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Task Title: Respond to a 10 Step CEA Misalignment

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<b>EP</b> Perfor	rmance:	Standard:	Critical:	Grade
<b>12</b> Section Mode 4.2.2 4.2.3	<ul> <li>PRESS "MANUAL INDIVIDUAL, MI" pushbutton and CHECK light, lit.</li> <li>SELECT applicable group for CEA to be moved on one of the following scales ("CEAPDS MONITOR"):</li> <li>"FULL RANGE"</li> <li>"+/- 15"</li> </ul>	<ol> <li>Standard:</li> <li>"MI" button pressed and light verified lit.</li> <li>Group 7 "Group Select" button pressed and verified lit.</li> <li>CEA #1 button pressed and verified lit.</li> </ol>	Critical: Y ⊠ N □	Grade S □ U □
4.2.4 4.2.5	SELECTION" pushbutton for CEA to be moved and CHECK light, lit.			
4.2.6	<ul> <li>SELECTION pushoution for CEA to be moved and CHECK light, lit.</li> <li>MONITOR CEA movement on the following:</li> <li>"CEAPDS MONITOR"</li> <li>PPC (desired display)</li> <li>Core mimic</li> </ul>			
Cue:		imic for CEA motion until the CEA reaches the fully v	ith drown a coitin	-

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Task Title: Respond to a 10 Step CEA Misalignment

STEP	Performance:	Standard:	Critical:	Grade		
#13	<ul> <li>4.2.7 WHEN desired to initiate CEA movement, PERFORM applicable action: <ul> <li>IF desired to insert CEA, PLACE and HOLD CEAcontrol switch to "INSERT."</li> <li>IF desired to withdraw CEA, PLACE and HOLD CEA control switch to "WITHDRAW."</li> </ul> </li> <li>4.2.8 WHEN movement of selected CEA is <i>no</i> longer desired, <i>slowly</i> RELEASE CEA control switch and CHECK CEA movement has stopped.</li> <li>Cue: If stated, US acknowledges the restoration of</li> </ul>	<ol> <li>Examinee moves the CEA control switch to the "withdraw" position and monitors CEA movement.</li> <li>CEA motion is stopped when CEA #1 is aligned with the rest of Group 7 CEAs.</li> </ol>	Y N			
	Comments:					
STEP #14	<ul> <li>Performance:</li> <li><u>Section 4.5</u> Bypass CMI Interlocks</li> <li>NOTE</li> <li>CMI should remain bypassed for at least three seconds after CEA motion</li> <li>is stopped to allow CPP operations to be completed.</li> <li>4.5.8 WHEN CEA motion has been stopped for at least three seconds, RELEASE <i>system</i> "CEA MOTION INHIBIT BYPASS" pushbutton.</li> </ul>	<ul> <li>Standard:</li> <li>Examinee may have already reviewed this Note and step 4.5.8, performing it immediately after releasing the CEA control switch.</li> <li>Examinee <i>may</i> verify the CEA Motion Inhibit Bypass button light goes out at this time.</li> </ul>	Critical: Y □ N ⊠	Grade S 🗌 U 🗌		
	Cue: Comments:					

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STEP	Performance:	Standard:	Critical:	Grade
#15	<ul> <li>4.5.9 CHECK system red "CEA MOTION INHIBIT BYPASS," light not lit.</li> <li>4.5.10 PRESS appropriate group "INHIBIT BYPASS" pushbutton and CHECK the following:</li> <li>Appropriate group red "INHIBIT</li> </ul>	<ol> <li>Examinee performs the following:</li> <li>Verifies CEA Motion Inhibit Bypass button not lit when released.</li> <li>Inhibit Bypass button is pressed, button is verified not lit and annunciator C-04/BA-19 is verified cleared.</li> </ol>	Y 🗌 N 🔀	S 🗌 U 🗌
	<ul> <li>BYPASS" button, not lit</li> <li>Annunciator C04, window BA 19, "CEA MOTION INHIBIT BYP," not lit</li> <li>4.5.11 IF CMI is no longer required to be bypassed, LOG exit of the following in Shift Turnover Log (CMI not bypassed):</li> <li>TSAS 3.1.3.1, ACTION b</li> <li>TSAS 3.1.3.1, ACTION c</li> <li>TSAS 3.1.3.6, ACTION d</li> </ul>	<ul> <li>3.1.3.1, ACTION c</li> <li>3.1.3.6, ACTION d</li> </ul>		
	Cue:	as been realigned with the rest of Group 7 and the CMI	circuit is no longe	er bypassed.

TERMINATION CUE: The evaluation for this JPM is concluded.

**STOP TIME**: \_\_\_\_\_

## **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-284-S-1

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🖾 No	)	
Validated Time (minutes):	20	Actual Time to Complete (minutes):		
Work Practice Performance:		SAT	UNSAT	
Operator Fundamentals:		SAT	UNSAT	
JPM Question Portion Overall [NLO only]:		SAT	UNSAT	N/A
Attached Question #1		SAT	UNSAT	
Attached Question #2		SAT	UNSAT	
Overall Result of JPM:		SAT	UNSAT	

Evaluator:

Print / Sign

Areas for Improvement / Comments:

# JPM QUESTIONS

Question #1:			
Answer #1:			
<u>Examinee</u> <u>Response</u> :			
<u>Grade:</u>	SAT	UNSAT	

Question #2:				
Answer #2:				
<u>Examinee</u> <u>Response</u> :				
Grade:	SAT	UNSAT		

# STUDENT HANDOUT

JPM Number:	JPM-284-S-1	Revision:	0/1
Initial Conditions:	The plant is stable at 100 % power, about to commence a down power to 55%. The crew has entered AOP 2575, Rapid Downpower, and has completed steps to 3.3, Forcing Pressurizer Sprays. The BOP is standing by to reduce turbine load.		
Initiating Cues:	You are the RO. Insert Group 7 rods 10 steps in accordance	with step 3.4 of A	OP 2575.

## JOB PERFORMANCE MEASURE APPROVAL SHEET

PM Number:	JPM 243	Revision: 0
nitiated:		
	David J. Jacobs	07/14/2014
	Developer	Date
Reviewed:		
	Doug M. Funk	07/14/2014
	Technical Reviewer	Date
Approved:		
	Mike J. Cote	07/17/2014
	Supervisor, Nuclear Training	Date

## **SUMMARY OF CHANGES**

DATE	DESCRIPTION	<b>REV/CHANGE</b>
07/14/2014	New JPM for Audit Exam ILT 2013-2014	0
08/15/2016	Incorporated NRC comments	0/1

JPM WORKSHEET				
Facility:   Millstone Unit 2   Examinee:				
JPM Number: JPM 243 Revision: 0/1				
Task Title:   Actuation Test ESF Components				
System: Engineered Safety Features Actuation System				
Time Critical Task: ( ) YES (X) NO				
Validated Time (minutes): 30				
Task Number(s):NUTIMS 013-01-002				
Applicable To:         SROX_         STA         ROX_         PEO				
K/A Number:013 A4.03 K/A Rating:4.5 / 4.7				
Method of Testing: Simulated Performance: <u>X</u>				
Location:     Classroom:     Simulator:     X     In-Plant:				
Task Standards:At the completion of the JPM the Examinee will have Tested an Engineered Safety Features Actuation System Actuation Module AM518 per SP 2604T "Actuation Tests of Various ESF Components"				
Required Materials:SP 2604T "Actuation Tests of Various ESF Components" Section 4.2 AM518(procedures, equipment, etc.)MP-PROC-OPS-SP 2604T-001				
General References:SP 2604T "Actuation Tests of Various ESF Components"Technical Specifications				

#### \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

	JPM WORKSHE	ET	
JPM Nur	ber: JPM 243	Revision :	0/1
Initial Conditions:	•	power with the following s in service and aligned pe 169B aligned for Standby	r OP 2330C
Initiating Cues:	You are directed to perform a to Perform SP 2604T Section 4.2 Record data on SP 2604T-001 All Alarms for this activity will	for Actuation Module AM	1518
Simulator Requirements:	<ul> <li>100% IC</li> <li>Following Overrides for F-54A</li> <li>CHHS8871_1 INSE</li> <li>CHHS8871_2 INSE</li> <li>11A1S5 DC SWGR F</li> </ul>	RT NR ( Not Red Light) RT G (Green Light Lit)	an to OFF:

#### \* \* \* \* <u>NOTES TO TASK PERFORMANCE EVALUATOR</u> \* \* \* \*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

	JPM Number:JPM 243Task Title:Actuation Tests of Vario	us ESF Components	0/1	
		S	START TIME: _	
STEP #1	<ul> <li>Performance:</li> <li>1. Refer To OP 2330C, "Chilled Water System," and ENSURE vital chiller, X-169A is aligned for standby operation AND X- 169B is <i>not</i> in operation.</li> </ul>	Standard: Examinee reads the NOTE and refers to the initial conditions for the Chilled Water System status.	Critical: Y [ ] N [ <b>X</b> ]	Grade S[]U[]
	Cue: ALL Alarms will addressed by the Reactor Operat Comments: Normal alignment Chiller X169A and X169B is n	or ot normally running "No Red Light" , but in Standby		
S T E P # 2	Performance: 2. STOP "F-54A, A DC SWGR RM A/C FAN," (C-80).	Standard: Examinee locates panel C-80 and the Handswitch for F-54A and places it to OFF and allows it to spring return to AUTO	Critical: Y [X] N [ ]	Grade S [ ] U [ ]
	Cue: Insert following Override Digital Outputs for F54 • CHHS8871_1 INSERT NR ( Not R • CHHS8871_2 INSERT G (Green L • 11A1S5 DC SWGR Fan-54A STOP		ed in STOP	
	Comments:	F-54A is place in OFF and PPC Alarm may clear whe	en the Actuation N	Iodule is

S T E P	Performance:	Standard:	Critical:	Grade
#3	3. Depending on current condition, PERFORM applicable action:	Examinee checks RCS pressure > 1800 psi and SIAS is not Blocked on C01 or ESF Cabinets.	Y[] N[ <b>X</b> ]	<b>S[]</b> U[]
	• IF SIAS is <i>not</i> "Blocked" (*PZR pressure greater than or equal to 1,850 psia), CHECK all "1/5" lights or bistable "TRIP" lights <i>not</i> lit for SIAS, CIAS, EBFAS, CSAS, or SG on Facility 1 and Facility 2 ESAS actuation and sensor cabinets.	Verifies no 1/5 lights lit on ESF actuation modules.		
	Cue: Comments:			
STEP	Performance:	Standard:	Critical:	Grade
#4	4. PLACE "S-501, TEST PERMISSIVE SWITCH" in "TEST SIAS." (ESAS actuation cabinet 5)	Examinee locates S-501 and places in test SIAS	Y [X] N [ ]	S[]U[
	Cue:			
	Comments:			
STEP	Performance:	Standard:	Critical:	Grade
#5	5. PLACE "S-502, TEST GROUP SWITCH" in "GROUP 5." (ESAS actuation cabinet 5)	Examinee locates S-502 and places in test Group 5	Y [X] N [ ]	<b>S</b> []U[]
	Cue:			
	Comments:			
STEP	Performance:	Standard:	Critical:	Grade
#6	6. PLACE "S-102, TRIP TEST" switch, in "CONT PRESS SIAS/CIAS/EBFAS/MSI." (ESAS Sensor Cabinet 'A')	Examinee locates S-102 and places in Containment Pressure SIAS/CIAS/EBFAS/MSI	Y [X] N [ ]	S[]U[
	Cue:			
	Comments:			

S T E P	Performance:	Standard:	Critical:	Grade
#7	<b>NOTE</b> When the next step is performed, many "1/5" lights illuminate on the actuation modules.	Examinee locates and <u>Presses</u> and <u>Holds</u> the trip test button on the correct Bistable BA101.	Y [ <b>X</b> ] N [ ]	S[]U[]
	7. PRESS and HOLD "TRIP TEST" button on bistable, BA101. (ESAS Sensor Cabinet 'A')			
	Cue:			
	Comments: Most Operators will find and place flag on the cor holding the trip test button.	rect Actuation Module prior to this step so you are not	t searching for the	AM while
S T E P # 8	<ul> <li>Performance:</li> <li>8. CHECK <i>lower</i> "1/5" light lit on actuation module, AM518. (ESAS actuation cabinet 5)</li> </ul>	Standard:	Critical: Y [ ] N [ <b>X</b> ]	Grade S [ ] U [ ]
	Cue:			
	Buttons are next to each other. The Simulator has	attons between the Simulator and the Plant. In the plant the both 1/5 lights above the two Trip Test Buttons. T		
	work for both the Simulator and the Plant.			
S T E P # 9	<ul> <li>work for both the Simulator and the Plant.</li> <li>Performance:</li> <li>9. To initiate start signal, PRESS <i>lower</i> "1/5" "TEST" button on actuation module, AM518. (ESAS actuation cabinet 5)</li> </ul>	Standard: Examinee presses the Test #2 Button to initiate equipment on AM518	Critical: Y [ <b>X</b> ] N [ ]	Grade S[]U[]
	<ul> <li>Performance:</li> <li>9. To initiate start signal, PRESS <i>lower</i> "1/5" "TEST" button on actuation module, AM518. (ESAS actuation cabinet 5)</li> <li>Cue:</li> </ul>	Examinee presses the Test #2 Button to initiate		
	<ul> <li>Performance:</li> <li>9. To initiate start signal, PRESS <i>lower</i> "1/5" "TEST" button on actuation module, AM518. (ESAS actuation cabinet 5)</li> <li>Cue:</li> <li>Comments:</li> <li>There is a difference in 1/5 lights and Trip Test Button</li> </ul>	Examinee presses the Test #2 Button to initiate	Y [ <b>X</b> ] N [ ]	S [ ] U [ ]
<b>#9</b>	<ul> <li>Performance:</li> <li>9. To initiate start signal, PRESS <i>lower</i> "1/5" "TEST" button on actuation module, AM518. (ESAS actuation cabinet 5)</li> <li>Cue:</li> <li>Comments:</li> <li>There is a difference in 1/5 lights and Trip Test Bu Buttons are next to each other. The Simulator has</li> </ul>	Examinee presses the Test #2 Button to initiate equipment on AM518	Y [ <b>X</b> ] N [ ]	S [ ] U [ ]
STEP	<ul> <li>Performance:</li> <li>9. To initiate start signal, PRESS <i>lower</i> "1/5" "TEST" button on actuation module, AM518. (ESAS actuation cabinet 5)</li> <li>Cue:</li> <li>Comments: There is a difference in 1/5 lights and Trip Test Bu Buttons are next to each other. The Simulator has work for both the Simulator and the Plant.</li> <li>Performance:</li> <li>10. RELEASE "TRIP TEST" button on bistable</li> </ul>	Examinee presses the Test #2 Button to initiate equipment on AM518 attons between the Simulator and the Plant. In the plan the both 1/5 lights above the two Trip Test Buttons. T	Y [X] N [ ] at the 1/5 lights and he procedure Steps Critical:	S[]U[] I Trip Test s as written

S T E P	Performance:	Standard:	Critical:	Grade			
STEP #11	<ul> <li>Performance:</li> <li>11. OBSERVE the following and INITIAL for the "Results" on SP 2604T-001:</li> <li>Actuation module, AM518, red "TRIP" light is lit (ESAS actuation cabinet 5)</li> <li>"F-54A, A DC SWGR RM A/C FAN," started (C-80)</li> <li>"P-122A, VITAL CHILL WTR PP," started (C-80)</li> <li>"X-169A, VITAL CHILLER," red indicating light is lit (C-80)</li> <li>"CHW-3, CHIL WTR SPLY TO DC SWGR A/C X-84A," is open (C-80)</li> <li>"CHW-11, CHIL WTR PPS P-122A/123 SPLY HDRS X-TIE," is closed (C-80)</li> </ul>	Standard: Examinee should locate and observe all the listed equipment running with the exception of <b>F-54A "A DC SWGR RM A/C FAN"</b> Examinee should make a report to the Unit Supervisor of the discrepancy. <u>Critical action</u> of this step is that the Examinee recognizes <b>F-54A "A DC SWGR RM A/C FAN"</b> <b>fails to Start.</b> Examinee should also identify the LCO for DC Switchgear Operability.	Critical: Y [ <b>X</b> ] N [ ]	Grade S[]U[]			
	• "CHW-13, CHILL WATER PUMPS (P122A/P123) RETURN HEADER CROSSTIE," is closed (C-80)						
	Cue: As the Unit Supervisor acknowledge the Report from the Examinee.						
	Comments: Ask the Examinee for any recommendations regarding the Fan and if any LCO should be entered and any T.S.A.S. required.						
	Examinee should recommend continuing with the procedure to restore cooling to the DC Switch gear Room by Starting the Fan and that the DC Switch gear maybe considered in-operable.						

TERMINATION CUE: The evaluation for this JPM is concluded.

**STOP TIME**: \_\_\_\_\_

#### **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM 243

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade. As necessary, refer to TIG-04 for additional Pass/Fail criteria.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🖾 No	)	
Validated Time (minutes):	30	Actual Time to 0	Complete (minutes):	
Work Practice Performance:		SAT	UNSAT	
Operator Fundamentals:		SAT	UNSAT	
JPM Question Portion Overall [NLO only]:		SAT	UNSAT	N/A
Attached Question #1		SAT	UNSAT	
Attached Question #2		SAT	UNSAT	
Overall Result of JPM:		SAT	UNSAT	

Evaluator:

Print / Sign

Areas for Improvement / Comments:

# JPM QUESTIONS

Question #1:		
Answer #1:		
<u>Examinee</u> <u>Response</u> :		
<u>Grade:</u>	SAT	UNSAT

Question #2:				
Answer #2:				
Examinee <u>Response</u> :				
Grade:	SAT	UNSAT		

# STUDENT HANDOUT

JPM Number:	JPM 243	<u>Revision</u> :	0/1
<u>nitial Conditions:</u>	<ul> <li>The Plant is operating at 100% power w</li> <li>Chilled Water System is in serv</li> <li>Chiller X-169A and X-169B ali</li> </ul>	ice and aligned per OP 2	2330C
nitiating Cues:	You are directed to perform a test of A Perform SP 2604T Section 4.2 for Actu Record data on SP 2604T-001 All Alarms for this activity will be add	ation Module AM518	

### JOB PERFORMANCE MEASURE APPROVAL SHEET

PM Number:	JPM-211	Revision: 1/1
nitiated:		
	Robert L. Cimmino, Jr.	03/22/2016
	Developer	Date
Reviewed:		
	David J. Jacobs	07/05/2016
	Technical Reviewer	Date
Approved:		
	Supervisor, Nuclear Training	Date

### **SUMMARY OF CHANGES**

DATE	DESCRIPTION	REV/CHANGE
03/22/2016	Updated to the latest format	1/0
08/15/2016	Incorporated NRC comments	1/1

#### JPM WORKSHEET

Facility: MP2	Examinee:			
JPM Number:	JPM-211	Revision: <u>1/1</u>	_	
Task Title: Pumpir	ng the Containment Sump - F	'aulted		
System: Station	Sumps and Drains		_	
Time Critical Task:	🗌 YES 🖾 NO			
Validated Time (minutes)	):15			
Task Number(s):	NUTIMS # 092-01-021			
Applicable To:	SRO STA	RO <u>X</u>	PEO	
K/A Number: 103	K/A Rating:	3.9/4.1		
Method of Testing: Si	imulated Performance:	Actua	ll Performance: X	
Location: C	lassroom:	Simulator: X	In-Plant:	
Task Standards:	At the completion of this JPM until receipt of the "CTMT N stop the pump(s) and report th	ORM SUMP DIS PRES	S HI" annunciator and	<b>.</b>
Required Materials: (procedures, equipment, etc.)	OP 2336, Station Sumps and ARP 2590E-108, (BB-21) "C		S PRESS HI"	
General References:	OP 2336, Section 4.2, Opera	tion of the Containmer	ıt Sump	

#### \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

#### JPM WORKSHEET

JPM Nu	mber:	JPM-211	Revision :	1/1
Initial Conditions:		s not available. requires a CTMT sump san	nple.	
Initiating Cues:	I will notif	rected to pump the Contain y you when Chemistry has o ps should then be secured.		
<u>Simulator Requirements</u> :	Containme Malfunctio Malfunctio No SIAS o Ensure PPO When direc	o any IC with: ent sump at > 30% on <b>WD02B</b> to fail SSP-16.2 on <b>WD04</b> (100%, 20 sec.) to or CIAS in progress C monitors turned OFF cted after JPM step 2 Action sec. ramp).	o clog the CTMT Sump	

#### \* \* \* \* <u>NOTES TO TASK PERFORMANCE EVALUATOR</u> \* \* \* \*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

	JPM Number: JPM-211	Revi	ision:	1/1	
	Task Title: <b>Pumping the Containn</b>	nent Sump – Faulted			
			5	START TIME: _	
STEP #1	<ul> <li>Performance:</li> <li><u>OP 2336, "Station Sumps and Drains"</u></li> <li>Section 4.2</li> <li>CAUTION. <ol> <li>CTMT sump should be treated as contaminated liquid.</li> <li>Note the frequency of pumping.</li> </ol> </li> <li>Cue: As necessary, acknowledge the Chemist has lare tracking the frequency of pumping the CTMT sumplements.</li> </ul>		and	Critical: Y 🗌 N 🔀 contaminated and	Grade S 🗌 U 🗍
STEP #2	Comments: Performance: NOTE 1. Sump pumps must be manually stopped. 2. Pumping time > 3 minutes could indicate a clogged strainer basket. 3. Normal full range of pumping CTMT sump raises AWDT 6 – 7%. Cue: If questioned, Rad. Waste PEO has stated the sump. Comments:	Standard: Examinee reads and acknowledges the Note. in-service AWDT has sufficient room to rece		Critical: Y 🗌 N 🖾 ntire contents of t	Grade S 🗌 U 🗌 he CTMT

JPM Number: JPM-211

Revision: 1/1

#### Task Title:Pumping the Containment Sump – Faulted

STEP	Performance:	Standard:	Critical:	Grade
#3	4.2.1 IF desired, START containment sump	Examinee starts one of the CTMT sump pumps.	Y 🛛 N 🗌	S 🗌 U 🗌
	pump "A" or "B" as follows (C-06):			
	a. PLACE "CTMT SUMP PP A" OR			
	"CTMT SUMP PP B," control			
	switch(es) to "START."			
		e <u>full</u> open and sump level begins to decrease, then t	rigger malfuncti	ons WD02B to
	fail SSP-16.2 open and WD04 (100%, 20 sec. ra	mp), "CTMT Sump Filter Clogged".		
	Comments:			
STEP	Performance:	Standard:	Critical:	Grade
#4	4.2.1 [START containment sump pump]:	Examinee verifies SSP-16.1 and SSP-16.2 open.	$Y \square N \boxtimes$	
	b. ENSURE the following open:			
	• "CTMT SUMP ISOL INBOARD,			
	SSP-16.1"			
	"CTMT SUMP ISOL			
	OUTBOARD, SSP-16.2"			
	ENSURE associated sump pump starts.			
	Cue:	•		
	Comments:			
GEDD				
STEP	Performance:	Standard:	Critical:	Grade
# 5	4.2.1 [START containment sump pump]:	Examinee verifies applicable sump pump starts.	Y 🗌 N 🖾	S 🗌 U 🗌
	c. ENSURE associated sump pump			
	starts.			
	Cue:			
	Comments:			

JPM Number: JPM-211

Revision: 1/1

#### Task Title:Pumping the Containment Sump – Faulted

STEP	Performance:	Standard:	Critical:	Grade
<b># 6</b>	Annunciator alarm C06/7 BB-21, "CTMT	Examinee observes annunciator C06/7 BB-21 and	Y 🛛 N 🗌	S 🗌 U 🗌
	NORM SUMP DIS PRESS HI"	takes action to stop the containment sump pump(s).		
	<u>ARP 2590E-108:</u>	The examinee may take immediate action to stop		
	Corrective Actions:	the pump [when level stops going down] and then		
	1. IF alarm is in for greater than one minute OR	refer to the ARP or may refer to the ARP and then		
	sump level is not lowering as expected	stop the pump.		
	PERFORM the following:			
	1.1. PLACE P33A, "CTMT SUMP PP A,"			
	and P33B, "CTMT SUMP PP B"			
	switches to "STOP" (C_06).			
	Cue:			
		e the CTMT sump pumps. It is not important that the		
		ors. However, after taking the action to stop the runnin		
should return to OP2336A and complete the step for ensuring the sump isolation valves close. If examinee does no		e does not state the	nis, question as	
	to what they are monitoring when the pumps are se			
STEP	Performance:	Standard:	Critical:	Grade
	4.2.2 IF desired, STOP containment sump	Examinee performs the following:	Y 🛛 N 🗌	
#7				S 🗌 U 🗌
#7	pump "A" or "B" as follows(C $0\hat{6}$ ):	1. Any running sump pump has been stopped.		3 0 0
#7	pump "A" or "B" as follows(C06): a. WHEN CTMT sump level has lowered	<ol> <li>Any running sump pump has been stopped.</li> <li>SSP-16.1 is verified closed by its green light</li> </ol>		
#7	pump "A" or "B" as follows(C06): a. WHEN CTMT sump level has lowered to 10 percent, PLACE "CTMT SUMP	1. Any running sump pump has been stopped.		3 [] 0 []
#7	pump "A" or "B" as follows(C06): a. WHEN CTMT sump level has lowered to 10 percent, PLACE "CTMT SUMP PP A" OR "CTMT SUMP PP B,"	<ol> <li>Any running sump pump has been stopped.</li> <li>SSP-16.1 is verified closed by its green light only lit.</li> </ol>		3 [] 0 []
#7	pump "A" or "B" as follows(C06): a. WHEN CTMT sump level has lowered to 10 percent, PLACE "CTMT SUMP PP A" OR "CTMT SUMP PP B," control switch(es) to "STOP."	<ol> <li>Any running sump pump has been stopped.</li> <li>SSP-16.1 is verified closed by its green light only lit.</li> <li><u>CRITICAL STEP</u></li> </ol>		3 [] 0 []
#7	<ul> <li>pump "A" or "B" as follows(C06):</li> <li>a. WHEN CTMT sump level has lowered to 10 percent, PLACE "CTMT SUMP PP A" OR "CTMT SUMP PP B," control switch(es) to "STOP."</li> <li>b. ENSURE the following closed:</li> </ul>	<ol> <li>Any running sump pump has been stopped.</li> <li>SSP-16.1 is verified closed by its green light only lit.</li> <li><u>CRITICAL STEP</u></li> <li>SSP-16.2 is verified to NOT have closed by its</li> </ol>		3 [] 0 []
#7	<ul> <li>pump "A" or "B" as follows(C06):</li> <li>a. WHEN CTMT sump level has lowered to 10 percent, PLACE "CTMT SUMP PP A" OR "CTMT SUMP PP B," control switch(es) to "STOP."</li> <li>b. ENSURE the following closed:</li> <li>"CTMT SUMP ISOL INBOARD,</li> </ul>	<ol> <li>Any running sump pump has been stopped.</li> <li>SSP-16.1 is verified closed by its green light only lit.</li> <li><u>CRITICAL STEP</u></li> <li>SSP-16.2 is verified to NOT have closed by its red light only lit and the Examinee attempts to</li> </ol>		3 [] 0 []
#7	<ul> <li>pump "A" or "B" as follows(C06):</li> <li>a. WHEN CTMT sump level has lowered to 10 percent, PLACE "CTMT SUMP PP A" OR "CTMT SUMP PP B," control switch(es) to "STOP."</li> <li>b. ENSURE the following closed:</li> <li>"CTMT SUMP ISOL INBOARD, SSP-16.1"</li> </ul>	<ol> <li>Any running sump pump has been stopped.</li> <li>SSP-16.1 is verified closed by its green light only lit.</li> <li><u>CRITICAL STEP</u></li> <li>SSP-16.2 is verified to NOT have closed by its</li> </ol>		3 [] 0 []
#7	<ul> <li>pump "A" or "B" as follows(C06):</li> <li>a. WHEN CTMT sump level has lowered to 10 percent, PLACE "CTMT SUMP PP A" OR "CTMT SUMP PP B," control switch(es) to "STOP."</li> <li>b. ENSURE the following closed:</li> <li>"CTMT SUMP ISOL INBOARD, SSP-16.1"</li> <li>"CTMT SUMP ISOL</li> </ul>	<ol> <li>Any running sump pump has been stopped.</li> <li>SSP-16.1 is verified closed by its green light only lit.</li> <li><u>CRITICAL STEP</u></li> <li>SSP-16.2 is verified to NOT have closed by its red light only lit and the Examinee attempts to</li> </ol>		3 [] 0 []
#7	<ul> <li>pump "A" or "B" as follows(C06):</li> <li>a. WHEN CTMT sump level has lowered to 10 percent, PLACE "CTMT SUMP PP A" OR "CTMT SUMP PP B," control switch(es) to "STOP."</li> <li>b. ENSURE the following closed:</li> <li>"CTMT SUMP ISOL INBOARD, SSP-16.1"</li> <li>"CTMT SUMP ISOL OUTBOARD, SSP-16.2"</li> </ul>	<ol> <li>Any running sump pump has been stopped.</li> <li>SSP-16.1 is verified closed by its green light only lit.</li> <li><u>CRITICAL STEP</u></li> <li>SSP-16.2 is verified to NOT have closed by its red light only lit and the Examinee attempts to</li> </ol>		
#7	<ul> <li>pump "A" or "B" as follows(C06):</li> <li>a. WHEN CTMT sump level has lowered to 10 percent, PLACE "CTMT SUMP PP A" OR "CTMT SUMP PP B," control switch(es) to "STOP."</li> <li>b. ENSURE the following closed:</li> <li>"CTMT SUMP ISOL INBOARD, SSP-16.1"</li> <li>"CTMT SUMP ISOL OUTBOARD, SSP-16.2"</li> </ul>	<ol> <li>Any running sump pump has been stopped.</li> <li>SSP-16.1 is verified closed by its green light only lit.</li> <li><u>CRITICAL STEP</u></li> <li>SSP-16.2 is verified to NOT have closed by its red light only lit and the Examinee attempts to manually closed valve.</li> </ol>		3 [] 0 []
#7	pump "A" or "B" as follows(C06): a. WHEN CTMT sump level has lowered to 10 percent, PLACE "CTMT SUMP PP A" OR "CTMT SUMP PP B," control switch(es) to "STOP." b. ENSURE the following closed: • "CTMT SUMP ISOL INBOARD, SSP-16.1" • "CTMT SUMP ISOL OUTBOARD, SSP-16.2" Cue: Booth Operator insert trigger to Allow the Closure	<ol> <li>Any running sump pump has been stopped.</li> <li>SSP-16.1 is verified closed by its green light only lit.</li> <li><u>CRITICAL STEP</u></li> <li>SSP-16.2 is verified to NOT have closed by its red light only lit and the Examinee attempts to manually closed valve.</li> </ol>		3 [] 0 []
#7	<ul> <li>pump "A" or "B" as follows(C06):</li> <li>a. WHEN CTMT sump level has lowered to 10 percent, PLACE "CTMT SUMP PP A" OR "CTMT SUMP PP B," control switch(es) to "STOP."</li> <li>b. ENSURE the following closed:</li> <li>"CTMT SUMP ISOL INBOARD, SSP-16.1"</li> <li>"CTMT SUMP ISOL OUTBOARD, SSP-16.2"</li> </ul>	<ol> <li>Any running sump pump has been stopped.</li> <li>SSP-16.1 is verified closed by its green light only lit.</li> <li><u>CRITICAL STEP</u></li> <li>SSP-16.2 is verified to NOT have closed by its red light only lit and the Examinee attempts to manually closed valve.</li> </ol>		

JPM Number:	JPM-211	Revision:	1/1
		Revision:	1/1

### Task Title:Pumping the Containment Sump – Faulted

S T E P # 8	Performance: Examinee reports the status of SSP-16.2 to the US.	Standard: The failure of SSP-16.2 to close is reported to the US.	Critical: Y 🗌 N 🔀	Grade S 🗌 U 🗌
	Cue: As the US, acknowledge the report and state	that the applicable Tech. Spec. required actions will be	e taken.	
	Comments: After this step is completed, the JPM	A is considered complete.		

TERMINATION CUE: The evaluation for this JPM is concluded.

**STOP TIME**: \_\_\_\_\_

### **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-211

Revision: 1/1

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🖾 No	)		
Validated Time (minutes):	15	Actual Time to C	Complete (minutes):		
Work Practice Performance:		SAT	UNSAT		
Operator Fundamentals:		SAT	UNSAT		
JPM Question Portion Overal	l [NLO only]:	SAT	UNSAT	<b>1</b>	N/A
Attache	d Question #1	SAT	UNSAT		
Attache	d Question #2	SAT	UNSAT		
Overall Result of JPM:		SAT	UNSAT		

**Evaluator:** 

Print / Sign

Areas for Improvement / Comments:

# JPM QUESTIONS

Question #1:				
Answer #1:				
<u>Examinee</u> <u>Response</u> :				
Grade:	SAT	UNSAT		

Question #2:		
Answer #2:		
<u>Examinee</u> <u>Response</u> :		
Grade:	SAT	UNSAT

# STUDENT HANDOUT

JPM Number:	JPM-211	Revision:	1/1
Initial Conditions:	The PPC is not available. Chemistry requires a CTMT sump samp	ıle.	
Initiating Cues:	You are directed to pump the Containma I will notify you when Chemistry has ob sump pumps should then be secured.	Ĩ	

### JOB PERFORMANCE MEASURE APPROVAL SHEET

JPM Number:	JPM-285-S-6	Revision:	0/1
Initiated:			
	Robert L. Cimmino, Jr.	07/1	2/2016
	Developer	I	Date
Reviewed:			
	Dave Jacobs	07/1	2/2016
	Technical Reviewer	Ι	Date
A 1.			
Approved:			
	Supervisor Nuclear Training		Data
	Supervisor, Nuclear Training	1	Date

### **SUMMARY OF CHANGES**

DATE	DESCRIPTION	<b>REV/CHANGE</b>
03/17/2016	Created from JPM-158 for the 2016 NRC License Exam	0
08/15/2016	Incorporated NRC comments	0/1

#### JPM WORKSHEET

Facility: MP2	Examinee:			
JPM Number:	JPM-285-S-6	Revision: 0/1	_	
Task Title: EOP 25	41 Appendix 23-N Energizing B	us 24E from Unit 3		
System:	olt AC		_	
Time Critical Task:	🗌 YES 🖾 NO			
Validated Time (minutes)	:30			
Task Number(s):	062-01-356	_		
Applicable To:	SRO STA	ROX	PEO	
K/A Number: 062	-A2.12 K/A Rating:	3.2/3.6		
Method of Testing: Si	mulated Performance:	Actua	l Performance: _	X
Location: C	lassroom: S	imulator: X	In-Plant:	
Task Standards:	Upon energizing bus 24E from on bus 24E and then de-energiz		as recognized a fa	ult exists
Required Materials: (procedures, equipment, etc.)	<ul><li>EOP 2541 Appendix 23,</li><li>Attachment 23-N "Energiz</li><li>Attachment 23-U "3 MVA</li></ul>	e		
General References:	EOP 2541 Appendix 23			

### \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

### JPM WORKSHEET

JPM Nur	nber: <b>JPM-285-S-6</b>	Revision : 0
Initial Conditions:	The Unit is currently in EOP 2526 Re degrading. Unit 3 has given permission to power	eactor Trip Recovery and the RSST is
Initiating Cues:	You are directed to energize Bus 24E "Restoring Electrical Power".	from Unit 3 per EOP 2541 Appendix 23
Simulator Requirements:	<ul> <li>IC-144 or equivalent NOP/N</li> <li>EDA-2/2153-24E-2 1106 and</li> </ul>	

### \* \* \* \* <u>NOTES TO TASK PERFORMANCE EVALUATOR</u> \* \* \* \*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

JPM Number: JPM-285-S-6

Revision: **0/1** 

### Task Title: EOP 2541 Appendix 23-N Energizing Bus 24E from Unit 3

#### START TIME: \_\_\_\_\_

STEP #1	<ul> <li>Performance:</li> <li>EOP 2541, Attachment 23-N</li> <li>Energizing 4.16 kV Bus 24E From Unit 3</li> <li>NOTE: The following may indicate a fault on 4.16 kV Bus 24E:</li> <li>Annunciator "4KV BUS 24E/34B TIE BKR A505 TRIP" lit (A-9, C08)</li> <li>Annunciator "4KV BUS 24C/E TIE BKR A305 TRIP" lit (B-10, C08)</li> <li>Annunciator "4KV BUS 24D/E TIE BKR A408 TRIP" lit (D-10, C08)</li> <li>1. CHECK that no fault indications are present for 4.16 kV Bus 24E.</li> </ul>	Standard: Examinee reads and acknowledges the Note. Checks that no fault indications are present for 4.16 kV bus 24E by verifying associated annunciators are not lit.	Critical: Y □ N ⊠	Grade S 🗌 U 🗍
	Cue: Comments:			
S T E P # 2	Performance: 2. ENSURE 4.16 kV Bus 24E "SPLY VOLTS" voltage is indicated.	Standard: Checks bus 24E voltage indicated. (C-08)	Critical: Y 🗌 N 🔀	Grade S 🗌 U 🗌
	Cue:			
	Comments:			

JPM Number: JPM-285-S-6

Revision: 0/1

#### Task Title:EOP 2541 Appendix 23-N Energizing Bus 24E from Unit 3

STEP #3	<ul> <li>Performance:</li> <li>3. ENSURE ALL of the following load breakers on 4.16 kV Bus 24E are open:</li> <li>A502, Service Water Pump B</li> <li>A503, HPSI Pump B</li> <li>A504, RBCCW Pump B</li> <li>Cue:</li> <li>Comments:</li> </ul>	<ul> <li>Standard:</li> <li>Ensures all of the following load breakers on 4.16 kV Bus 24E are open:</li> <li>A502, Service Water Pump B</li> <li>A503, RBCCW Pump B</li> <li>A504, HPSI Pump B</li> </ul>	Critical: Y □ N ⊠	Grade S 🗌 U 🗌
S T E P # 4	<ul> <li>Performance:</li> <li>4. ENSURE ALL of the following breakers are open:</li> <li>A305, "24C/24E TIE BKR, 24C-2T-2"</li> <li>A408, "24D/24E TIE BKR, 24D-2T-2"</li> </ul>	<ul> <li>Standard:</li> <li>Opens breaker A305, "24C/24E TIE BKR, 24C-2T-2" by placing switch to trip, green light on, red light off. <i>Not</i> "Critical Step</li> <li>Verifies breaker A408, "24D/24E TIE BKR, 24D-2T-2" racked down (open), green light off, red light off.</li> </ul>	Critical: Y ⊠ N □	Grade S 🗌 U 🗌
STEP	Cue: Booth Operator <b>INSERT</b> Malfunction EDA-2/21: Comments: Performance:	53-24E-2 1106 amps	Critical:	Grade
#5	<ul> <li>5. REQUEST permission from Unit 3 Shift</li> <li>Manager or Unit Supervisor to energize Unit 2</li> <li>4.16 kV Bus 24E from Unit 3 4.16 kV Bus 34A/34B.</li> </ul>	Verifies initial Conditions	Y N X	
	Cue: as necessary respond as Unit 3 SM/US; "You Comments:	u have permission to energize Bus 24E from Unit 3, Bu	ıs 34B".	

JPM Number: JPM-285-S-6

Revision: 0/1

### Task Title:EOP 2541 Appendix 23-N Energizing Bus 24E from Unit 3

STEP	Performance:	Standard:	Critical:	Grade
81EP #6	NOTE: Due to the "Dead Bus" state of 4.16 kV	Examinee reads and acknowledges the Note.	$Y \boxtimes N \square$	
	<ul> <li>Bus 24E, the synchroscope will not move.</li> <li>6. PLACE "SYN SEL SW, 34B-24E-2 (A505)" to "ON" and CHECK "INCOMING" voltage indicated.</li> </ul>	Places "SYN SEL SW, 34B-24E-2 (A505)" to "ON" and checks "INCOMING" voltage indicated. (C-08).		
	Cue:			
	Comments: If timed out, acknowledges alarm for	"Sync Selector Switch On".		
STEP	Performance:	Standard:	Critical:	Grade
#7	7. CLOSE A505, "24E/34B TIE BKR, 34B-24E-2".	Closes A505, "24E/34B TIE BKR,34B-24E-2" by placing to close and releasing after red light on, green light off.	Y 🖾 N 🗌	S 🗌 U 🗌
	Cue: When A505 is closed, trigger the I/O that raises 24	1 7		
	Comments: The Examinee may or may not OPEN to address the malfunction.	the BREAKER and may not get to <b>JPM</b> steps #8 it is	irrelevant if this s	tep is skipped
STEP #8	Performance: 8. CHECK voltage indicated on "RUNNING" voltmeter. 9. PLACE "SYN SEL SW, 34B-24E-2 (A505)" to "OFF".	Standard: Checks voltage indicated on "RUNNING" voltmeter (C-08). Places "SYN SEL SW, 34B-24E-2 (A505)" to "OFF".	Critical: Y 🗌 N 🔀	Grade S 🗌 U 🗌
	Cue:			
	Comments:			

JPM Number: JPM-285-S-6

Revision: 0/1

### Task Title:EOP 2541 Appendix 23-N Energizing Bus 24E from Unit 3

STEP #9	Performance: 10. Refer To Attachment 23-U, "3 MVA Electrical Limit on Bus 34A/34B," and ENSURE that 3 MVA is not exceeded as loads are restored to service.	Standard: Examinee notes 24E/34B bus amps (C-08) are above the 3MVA limit and recommends OPENING supply breaker 34B-24E-2 (A505).	Critical: Y ⊠ N □	Grade S 🗌 U 🗌
	Cue: If at any time Examinee suggests tripping A505	, concur and/or direct.		
	Comments: Step #10 is listed only because the Exa However, its use is <i>not</i> relevant. <b>The JPM is completed when breaker A505 is m</b>	aminee may use it when addressing the indications of a anually tripped from C-08.	a problem with 24	Е.
STEP #10	<ul> <li>Performance:</li> <li>11. At the direction of the US, REALIGN as necessary for the applicable facility and PLACE the following pumps in service: <ul> <li>Service Water Pump B</li> <li>HPSI Pump B</li> <li>RBCCW Pump B</li> </ul> </li> </ul>	Standard: Places "B" SW pump in service removes "A" SW pump. Examinee notes 24E/34B bus amps (C-08) are above the 3MVA limit and recommends OPENING supply breaker 34B-24E-2 (A505).	Critical: Y ⊠ N □	Grade S 🗌 U 🗌
	Cue:	er DIRECT placing "B" Service water pump in service <b>a, concur and/or direct.</b>	Secure "A"SW p	ump
	Comments: The only critical part of this step is ope The JPM is completed when breaker A505 is m	•		

TERMINATION CUE: The evaluation for this JPM is concluded.

STOP TIME: \_\_\_\_\_

#### **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-285-S-6

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🖾 No	)	
Validated Time (minutes):	30	Actual Time to C	Complete (minutes):	
Work Practice Performance:		SAT	UNSAT	
Operator Fundamentals:		SAT	UNSAT	
JPM Question Portion Overall [NLO only]:		SAT	UNSAT	N/A
Attache	d Question #1	SAT	UNSAT	
Attache	d Question #2	SAT	UNSAT	
Overall Result of JPM:		SAT	UNSAT	

Evaluator:

Print / Sign

Areas for Improvement / Comments:

# JPM QUESTIONS

Question #1:				
Answer #1:				
<u>Examinee</u> <u>Response</u> :				
Grade:	SAT	UNSAT		

Question #2:		
Answer #2:		
<u>Examinee</u> <u>Response</u> :		
Grade:	SAT	UNSAT

# STUDENT HANDOUT

JPM Number:	JPM-285-S-6	Revision:	0/1
Initial Conditions:	The Unit is currently in EOP 2526 Reactor degrading.	Trip Recovery and t	the RSST is
	Unit 3 has given permission to power Bus 2	.4E	
Initiating Cues:	You are directed to energize Bus 24E from "Restoring Electrical Power".	Unit 3 per EOP 254	1 Appendix 23

### JOB PERFORMANCE MEASURE APPROVAL SHEET

PM Number:	JPM-288-S-2	Revision: 0
initiated:		
	David J. Jacobs	04/13/2016
	Developer	Date
Reviewed:		
	Robert L. Cimmino	07/05/2016
	Technical Reviewer	Date
Approved:		
	Supervisor, Nuclear Training	Date

# **SUMMARY OF CHANGES**

DATE	DESCRIPTION	<b>REV/CHANGE</b>	
04-13-2016	New JPM ILT Exam 2016	0	

#### JPM WORKSHEET

Facility: MP Unit 2 Examinee:								
JPM Number:         JPM-288-S-2         Revision:         0								
Task Title:   AOP 2551 SD from Outside the Control Room C21 PZR LVL								
System: 011 Pressurizer Level Control System								
Time Critical Task:								
Validated Time (minutes): 10								
Task Number(s): 000-04-155								
Applicable To:         SRO X         STA         RO X         PEO								
K/A Number: 011 A1.01 K/A Rating: 3.2/3.1								
Method of Testing: Simulated Performance: Actual Performance: X								
Location: Classroom: Simulator: X In-Plant:								
Task Standards:Operator restores and maintains PZR Pressure 2225-2300 psia and Level 35% to 70% from C-21 "Remote Shutdown Panel with a fault on the Controlling Channel of Pressurizer Level.								
Required Materials: MP-PROC-OPS-AOP 2551[r009.03] Shutdown from Outside the Control Room (procedures, equipment, etc.)								
General References: MP-PROC-OPS-AOP 2551[r009.03] Shutdown from Outside the Control Room								

# \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

# JPM WORKSHEET JPM-288-S-2 JPM Number: Revision : 0 **Initial Conditions:** The Unit was tripped from 100% and the Crew has evacuated the Control Room. The Unit Supervisor has entered AOP 2551 "Shutdown from Outside the Control Room" The Shift Manager has Classified the Event. All steps up to and including 3.10 have been completed. Initiating Cues: The Unit Supervisor has directed you to perform Steps 3.11 and 3.12 • Maintain Pressurizer Level 35% to 70% Maintain Pressurizer Pressure 2225 psia to 2300 psia • Simulator Requirements: **RCS at NOP/NOT** All Charging Pump H/S in NORM after START Malfunction for PZR Level RX04A (X Ch.) or RX04B (Y Ch.) to 100% with a 60 second Ramp

### \*\*\*\* NOTES TO TASK PERFORMANCE EVALUATOR \*\*\*\*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

	JPM Number:	JPM-288-S-2		Revision:	0	
	Task Title:	AOP 2551 SD from Ou	utside the Control Room C21 PZR	LVL		
				S	START TIME: _	
S T E P # 1	outside the Control Ro 3.11 DETERMINE tha acceptance criteria are of the following: a. CHECK that <b>BOTH</b> conditions exist: • Pressurizer lev • Pressurizer lev 70%. Cue:	at RCS Inventory Control met by performing ALL	Standard: Examinee should review the controlle available at C-21 to control PZR level • Letdown flow controller HIC • Charging Pumps in and out of	-110 manual	Critical: Y 🗌 N 🖾	Grade S 🗌 U 🗌
S T E P # 2	Performance: 3.12 DETERMINE RC acceptance criteria are following: • CHECK that F 2350 psia.	met by <b>BOTH</b> of the PZR pressure is 1900 to PZR pressure is trending to	Standard: Examinee should review the controlle available at C-21 to control PZR press • PZR backup Heaters in or out • Spray Valves in manual	sure.	Critical: Y 🗌 N 🔀	Grade S 🗌 U 🗌
	Comments:					

JPM Number: **JPM-288-S-2** 

Revision: 0

## Task Title:AOP 2551 SD from Outside the Control Room C21 PZR LVL

STEP #3		Standard: Locates and Monitors PZR level and RCS Pressure on C-21 en 35 and 70% And PZR pressure 2225 to 2300 psia or to insert RX04A or RX04B for the controlling chann	Critical: Y 🗌 N 🔀 nel of PZR level o	Grade S 🗌 U 🗌 on a 60 second
	Comments:			
STEP #4	<ul> <li>Performance:</li> <li>Examinee observes the following: <ul> <li>PZR Level Slowly rising on the Controlling CH.</li> <li>PZR Level slowly lowering on the NON Controlling CH.</li> <li>RCS Pressure slowly rising. (BU HTRS ON)</li> <li>@4% PZR level above program level all Backup Heaters energizing</li> <li>IF 2 charging pumps were operating 1 shuts off (BU SIG OFF)</li> <li>Letdown Flow rising HIC-110</li> </ul> </li> <li>Comments:</li> </ul>	<ul> <li>Standard:</li> <li>Examinee States the following: <ul> <li>PZR level and Trend</li> <li>RCS pressure and Trend</li> </ul> </li> <li>Examinee performs the following: <ul> <li>Takes manual control of HIC-110 and matches Charging and Letdown Flow.</li> </ul> </li> <li>Takes manual control of Spray valves and Initiates Spray to maintain RCS pressure. OR <ul> <li>Places PZR BU Heaters in PTL to maintain RCS pressure.</li> </ul> </li> </ul>	Critical: Y 🖾 N 🗖	Grade S 🗌 U 🗌

JPM Number: **JPM-288-S-2** 

Revision: 0

#### Task Title:AOP 2551 SD from Outside the Control Room C21 PZR LVL

STEP #5	<ul> <li>Performance:</li> <li>Examinee Reports RCS Pressure and PZR level in manual control maintaining the following:</li> <li>PZR level 35 to 70%</li> <li>RCS pressure 2225 to 2300 psia</li> </ul>	Standard: Examinee manually controls Pressure and Level with in the procedural guidance.	Critical: Y ⊠ N □	Grade S 🗌 U 🗌
	Cue: Comments:			

TERMINATION CUE: The evaluation for this JPM is concluded.

**STOP TIME**: \_\_\_\_\_

## **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-288-S-2

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🖾 No	0		
Validated Time (minutes):	10	Actual Time to 0	Complete (minutes):		
Work Practice Performance:		SAT	UNSAT		
Operator Fundamentals:		SAT	UNSAT		
JPM Question Portion Overall [NLO only]:		SAT	UNSAT	[] I	N/A
Attached Question #1		SAT	UNSAT		
Attached Question #2		SAT	UNSAT		
Overall Result of JPM:		SAT	UNSAT		

**Evaluator:** 

Print / Sign

Areas for Improvement / Comments:

## JPM QUESTIONS

Question #1:		
Answer #1:		
<u>Examinee</u> <u>Response</u> :		
Grade:	SAT	UNSAT

Question #2:		
Answer #2:		
Examinee <u>Response</u> :		
<u>Grade:</u>	SAT	UNSAT

## STUDENT HANDOUT

JPM Number:	JPM-288-S-2	Revision:	0
Initial Conditions:	The Unit was tripped from 100% and the Room.	Crew has evacuate	ed the Control
	The Unit Supervisor has entered AOP 25: Control Room"	51 "Shutdown from	n Outside the
	The Shift Manager has Classified the Eve	nt.	
	All steps up to and including 3.10 have be	en completed.	
Initiating Cues:	The Unit Supervisor has directed you to pe	rform Steps 3.11 a	nd 3.12
	• Maintain Pressurizer Level 35% to	70%	

• Maintain Pressurizer Pressure 2225 psia to 2300 psia

## JOB PERFORMANCE MEASURE APPROVAL SHEET

PM Title:	Evaluate RCP Problem	
PM Number:	JPM-011	Revision: 10
nitiated:		
	Robert L. Cimmino, Jr.	05/03/2016
	Developer	Date
Reviewed:		
	David J. Jacobs	07/05/2016
	Technical Reviewer	Date
Approved:		
	Supervisor, Nuclear Training	Date

## **SUMMARY OF CHANGES**

DATE	DESCRIPTION	<b>REV/CHANGE</b>
06/27/2003	Updated to reflect changes to OP 2301C and new 1500 psid criteria.	8
05/03/2016	Updated to reflect new format and procedure changes.	9
08/12/2016	Revised to NRC comments change to Upper Oil Reservoir LO LVL	10

#### JPM WORKSHEET

Facility: MP2	Examinee:		
JPM Number:	JPM-011	Revision: 10	-
Task Title: Evaluate	e RCP Problem		
System: <u>RCP</u>			-
Time Critical Task:	🗌 YES 🖾 NO		
Validated Time (minutes)	:20		
Task Number(s):	NUTIMS #003-01-033	_	
Applicable To:	SRO STA	RO <u>X</u>	PEO
K/A Number: 003	/A2.01 K/A Rating:	3.5/3.9	
K/A Number: 003	/A4.04 K/A Rating:	3.1/3.0	
Method of Testing: Si	mulated Performance:	Actua	Performance: X
Location: C	lassroom: S	Simulator: X	In-Plant:
Task Standards:	At the completion of this JPM, reported that the upper lube oil criteria for the "A" RCP and th	l level is lowering with	conditions meeting trip
Required Materials: (procedures, equipment, etc.)	AOP 2586 "RCP Malfunctions ARP 2590B-083 (BA-19)	3"	
General References:	ARP 2590B-083 (BA-19) AOP 2586 "RCP Malfunctions OP 2272C "Plant Operation in		tor Startup"

## \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

JPM WORKSHEET						
	JPM Number:	JPM-011	Revision :	10		
Initial Conditions	• ]	IF required allow pre-brief of OP prior to Reactor Startup" Plant is in MODE 3 OP 2272C "Plant Operation in M	-			
Initiating Cues:	•	You are the PPO				
Simulator Require	(	Normal Operating temperature ar equivalent Malfunction "RC12A" 2 minute r	•	-144) or		

## \* \* \* \* <u>NOTES TO TASK PERFORMANCE EVALUATOR</u> \* \* \* \*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

	JPM Number: JPM-011	Revision:	10	
	Task Title: Evaluate RCP Proble	m		
		S	START TIME: _	
STEP #1	Performance: <u>ARP 2590B-083</u> Respond to "RCP A UPR OIL RSVR LEVEL LO" annunciator (BB-19 on C-02/3). <u>Automatic Functions</u> 1. None	Standard: Examinee refers to ARP 2590B, BA-19 and reads NO AUTOMATIC FUNCTIONS SETPOINT 60%	Critical: Y □ N ⊠	Grade S 🗌 U 🗌
	Cue: Comments:			
S T E P # 2	Performance: <u>Corrective Actions</u> CHECK "A" RCP upper reservoir oil level indication and DETERMINE rate of level decrease (C-04R or PPC) (Normal range, 75 to 85%).	Standard: Examinee uses the PPC or the Indications behind C04R switches to determined "A" RCP Upper Oil Reservoir level and notes lowering level.	Critical: Y □ N ⊠	Grade S 🗌 U 🗌
	Cue:	o address all alarming annunciators. It is only Critical th arming annunciators.	nat the Examinee	refer to AOP
STEP #3	Performance: <b>3</b> IF "A" RCP upper reservoir oil level is lowering, Go To AOP 2586, "RCP Malfunctions."	Standard: Examinee obtain a copy of and refers to AOP 2586 "RCP Malfunctions."	Critical: Y 🗌 N 🔀	Grade S 🗌 U 🗌
	Cue:			
	Comments:			

JPM Number: JPM-011

Revision: 10

Task Title: Evaluate RCP Problem

STEP #4	Performance: <u>AOP 2586, RCP Malfunctions</u> <u>Action/Expected Response</u> NOTE: Foldout page shall be monitored throughout this procedure. Cue:	Standard: Examinee transitions to AOP 2586 and performs the following: Examinee opens "foldout page" for monitoring.	Critical: Y □ N ⊠	Grade S 🗌 U 🗌	
	Comments:				
STEP #5	Performance: 1. Check RCP Trip Criteria Met.	Standard: Examinee reviews PPC data for Upper Oil Level and acknowledges RCP trip criteria is met or is rapidly approaching the criteria.	Critical: Y 🛛 N 🗌	Grade S 🗌 U 🗌	
	Cue:				
	Comments:				
STEP #6	Performance: <b>THEN PERFORM</b> the following: a. <b>TRIP</b> Reactor b. <b>TRIP</b> affected RCP c. <b>GO TO</b> EOP 2525, Standard Post Trip Actions.	Standard: Examinee notes Trip Criteria for "A" RCP Upper Oil Reservoir Low Level on the PPC then performs the following: a. Notes the Reactor is already Tripped b. Trips the "A" RCP c. Notes EOP 2525 not required	Critical: Y ⊠ N □	Grade S 🗌 U 🗌	
	Cue:				
	Comments: Examinee may finish ANO actions by proceeding to Step 2.c. prior to stating any issues noted with the recorded RCI				

TERMINATION CUE: The evaluation for this JPM is concluded.

STOP TIME: \_\_\_\_\_

## **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-011

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🖾 No			
Validated Time (minutes):	20	Actual Time to C	Complete (minutes):		
Work Practice Performance:		SAT	UNSAT		
Operator Fundamentals:		SAT	UNSAT		
JPM Question Portion Overal	l [NLO only]:	SAT	UNSAT		N/A
Attache	d Question #1	SAT	UNSAT		
Attache	d Question #2	SAT	UNSAT		
Overall Result of JPM:		SAT	UNSAT		

Evaluator:

Print / Sign

Areas for Improvement / Comments:

## JPM QUESTIONS

Question #1:				
Answer #1:				
<u>Examinee</u> <u>Response</u> :				
Grade:	SAT	UNSAT		

Question #2:		
Answer #2:		
<u>Examinee</u> <u>Response</u> :		
Grade:	SAT	UNSAT

## STUDENT HANDOUT

JPM Number:	JPM-011	Revision: 10
Initial Conditions:	<ul><li>Plant is in MODE 3</li><li>OP 2272C "Plant Operation in MOD</li></ul>	E 3 prior to Reactor Startup"
Initiating Cues:	• You are the PPO	

## JOB PERFORMANCE MEASURE APPROVAL SHEET

PM Number:	JPM-230	Revision:	2/1
nitiated:			
	John W. Riley	08/	21/12
	Developer	D	ate
Reviewed:			
	Ken Truesdale	08/2	22/12
	Technical Reviewer	D	ate
Approved:			
	James V. Grogan	08/23	3/2012
	Supervisor, Nuclear Training	D	ate

## **SUMMARY OF CHANGES**

DATE	DESCRIPTION	<b>REV/CHANGE</b>
N/A, 08/04/2008	New JPM with new malfunction for LPSI Pump breaker failed closed alternate path. New malfunction validated on 08/11/2008 with IC-98. PSS.	0
N/A	Revised for new Trex system and selection of new IC #289 for set up.	1
JWR 08/21/2012	Revised JPM to new format. Modified JPM in response to NRC feedback for alternate path JPMs. Added steps in the beginning of JPM to take action to align charging pump suction to the RWST prior to checking SRAS. Per NRC feedback during the 2010 71111.11 inspection a good alternate path JPM has the operator perform actions prior to taking contingency action.	2/0
08/18/2016	Incorporate NRC comments	2/1

	JPM WORKS	HEET		
Facility: MP 2	Examinee:			
JPM Number:	JPM-230	Revision: 2/1		
Task Title: Respon	d to LPSI Pump failure to trip on S	SRAS Actuation		
System: ECCS/E	ESAS			
Time Critical Task:	() YES (X) NO			
Validated Time (minutes)	:15			
Task Number(s):	NUTIMS #000-05-222			
Applicable To:	SRO X RO X	PEO		
K/A Number: 013	-A4.01 K/A Rating:	4.5/4.8		
Method of Testing: St	mulated Performance:	Actual	Performance:	X
Location: C	lassroom: Sir	nulator: <u>X</u>	In-Plant:	
Task Standards:	At the completion of this JPM, th Pump to trip on SRAS and imple from EOP-2532, LOCA.			
Required Materials: (procedures, equipment, etc.)	EOP 2532, Loss of Coolant Acc	ident, steps 46 through	48 (Revision 29 o	change 01)
General References:	EOP 2532; Loss of Coolant Acc	ident.		

## \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

## JPM WORKSHEET

JPM Num	ber: JPM	-230	Revision :	2/1
Initiating Cues:		rm Steps 46, 47, and 48 lequate Suction for SI P		
Initial Conditions:	<ul><li>EOP 2525 was co</li><li>BAST levels are</li></ul>		ransitioned to EO roximately 9%.	
Simulator Requirements:	<ul> <li>Reset to IC-289: RWST lowering</li> <li>IC-289 inserts mathematical failure to trip</li> <li>Steps of EOP 253: suction to the RW imminent</li> <li>RCPs off</li> <li>2-SI-659/660 are</li> <li>CETs are &lt; 345 controls are </li> <li>Rx vessel level </li> <li>Pressurizer level</li> <li>RWST at ~14% (</li> <li>BAST levels at ~</li> </ul>	egrees F 43%	conditions; RWS Ts at ~9% and lo ne "A" LPSI Pum ep 46, aligning ch 9%) with SRAS in 7ST") LBAT8B)	wering. p breaker aarging pump
* * * * <u>NO</u> 1. Critical steps for this JP		RFORMANCE EVA		

- grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

	JPM Number: JPM-230 Task Title: Respond to LPSI Pump	Revision: Failure to Trip on SRAS Actuation	2/1	
		* *	START TIME:	
TEP #1	<ul> <li>Performance:</li> <li>46. <u>IF</u> BAST levels are less than 10% <u>OR</u> boration from the BASTs is not required, PERFORM the following:</li> <li>a. OPEN CH-192, RWST isolation.</li> <li>b. ENSURE CH-504, RWST to charging suction is open.</li> <li>c. STOP BOTH boric acid pumps.</li> </ul>	<ul> <li>Standard:</li> <li>BAST levels are &lt; 10% and the following actions must be taken to shift the charging pump suction from the BASTs to the RWST.</li> <li>a. Examinee opens CH-192, RWST isolation. Verifies red light lit and green light not lit.</li> <li>b. Examinee ensures CH-504, RWST to charging suction is open. Verifies red light lit and green light not lit</li> <li>c. Examinee stops both boric acid pumps by taking hand switches to "start" and then to "stop". Verifies green light lit, red light not lit. Discharge pressure may also be checked to verify pumps are off.</li> </ul>	Critical: Y [ X ] N [ ]	Grade S[]U[]
	Cue:			
	Comments: Pages 38 and 39 of EOP 2532 should taking the simulator to run.	be provided to the examinee. Allow the examinee to v	walk down the boa	rds prior to

	JPM Number:	JPM-230	Revision:	2/1	
	Task Title:	Respond to LPSI Pump	Failure to Trip on SRAS Actuation		
STEP #2	<ul> <li>closed:</li> <li>CH-514, boric a</li> <li>CH-509, gravity BAST A.</li> </ul>	STs is not required, ing ( <b>Cont</b> ): he following valves are acid isolation. y feed isolation from y feed isolation from putlet isolation.	<ul> <li>Standard:</li> <li>d. Examinee closes or verifies closed the following valves and verifies their green lights are lit and red lights are not lit:</li> <li>CH-514, boric acid isolation.</li> <li>CH-509, gravity feed isolation from BAST A.</li> <li>CH-508, gravity feed isolation from BAST B.</li> <li>CH-501, VCT outlet isolation.</li> <li>CH-196, VCT makeup bypass.</li> </ul>	Critical: Y [ X ] N [ ]	Grade S [ ] U [
		*	the RWST level to be ready to ensure SRAS initiation.		
5 T E P # 3	Performance: 47. Check containmer rising.	nt sump wide range level	Standard: Examinee checks containment sump wide range level rising. Uses level indicator on C101 (L8242)	Critical: Y [ ] N [ X ]	Grade S [ ] U [
	Cue: Comments:				

<b>ERFORMANCE INFORMATION</b>
-------------------------------

	JPM Number:	JPM-230	Revision:	2/1	
	Task Title:	Respond to LPSI Pump	Failure to Trip on SRAS Actuation		
5 T E P # 4	Performance:		Standard:	Critical:	Grade
" -		containment <u>AND</u> RWST ual to 9%, ENSURE the	Examinee identifies SRAS has actuated by SRAS annunciators on C01 (C-35 and D-35).	Y [ X ] N [ ]	S[]U[]
	a. SRAS has actuated.	(C01)	Examinee checks that both LPSI pumps have stopped. Examinee should use pump amp meters and red and green hand switch lights to determine	Y[X] N[]	S[]U[]
	b. BOTH LPSI pumps	s are stopped.	status. Examinee identifies that the "B" LPSI pump has stopped and the "A" LPSI pump is still running.		
			Examinee takes action to stop the "A" LPSI pump by taking it's hand switch to start and then to stop. This will not stop the "A" LPSI pump but should be attempted. If stopping the "A" LPSI pump is	Y[]N[X]	S[]U[]
			not attempted it does not constitute a failure of the JPM but should be identified as a weakness.		
	Cue: At this point, the breaker did not open w	• •	PEO to open the "A" LPSI Pump breaker locally. Wait	2-3 minutes and r	eport the

	JPM Number: JPM-23 Task Title: Respon		Revision: Failure to Trip on SRAS Actuation	2/1	
T E P # 5	Performance:	F	Standard:	Critical:	Grade
# 5	48.b.1 IF LPSI pumps cannot be PERFORM the following:	stopped,	Examinee determines that BOTH LPSI pumps are not stopped and goes to contingency action 48.b.1.	Y[] N[X]	S[]U[]
	<ol> <li>CLOSE SI-635, LPSI injection</li> <li>ENSURE TWO of the followi fully closed and only ONE of the open:</li> </ol>	ng valves are	Examinee closes SI-635, LPSI injection valve. Verifies valve closed by green light lit and red light not lit. Should also verify flow at zero gpm on associated flow meter.	Y[X] N[]	S[]U[]
	<ul> <li>SI-615, LPSI injection val</li> <li>SI-625, LPSI injection val</li> <li>SI-645, LPSI injection val</li> </ul>	ve	Examinee closes TWO of the other three LPSI injection valves. Closes two of the following: SI-615, SI-625, and/or SI-645. Verifies valves closed by green light lit and red light not lit.	Y[X] N[]	S[]U[]
	Cue: Comments:				

TERMINATION CUE: The evaluation for this JPM is concluded.

**STOP TIME**: \_\_\_\_\_

## **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-230

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade. As necessary, refer to TIG-04 for additional Pass/Fail criteria.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🛛 No	)	
Validated Time (minutes):	15	Actual Time to 0	Complete (minutes):	
Work Practice Performance:		SAT	UNSAT	
Operator Fundamentals:		SAT	UNSAT	
JPM Question Portion Overall [NLO only]:		SAT	UNSAT	N/A
Attached Question #1		SAT	UNSAT	
Attached Question #2		SAT	UNSAT	
Overall Result of JPM:		SAT	UNSAT	

Evaluator:

Print / Sign

Areas for Improvement / Comments:

## JPM QUESTIONS

Question #1:		
Answer #1:		
<u>Examinee</u> <u>Response</u> :		
<u>Grade:</u>	SAT	UNSAT

Question #2:				
Answer #2:				
Examinee <u>Response</u> :				
Grade:	SAT	UNSAT		

## STUDENT HANDOUT

JPM Number:	JPM-230	Revision:	2/1
Initiating Cues:	You are the RO perform Steps 46, 47, and 48 RWST" Ensure Adequate Suction for SI Pum	0 0 01 1	
Initial Conditions:	<ul> <li>The plant experienced a large-break LOC</li> <li>EOP 2525 was completed and the crew transmission in the second second</li></ul>	ansitioned to EOP 2532 oximately 9%.	

## JOB PERFORMANCE MEASURE APPROVAL SHEET

PM Number:	JPM-287-C-6	Revision: 0/1
nitiated:		
	David J. Jacobs	04/29/2010
	Developer	Date
Reviewed:		
	Robert L. Cimmino	07/05/2016
	Technical Reviewer	Date
Approved:		
	Supervisor, Nuclear Training	Date

## **SUMMARY OF CHANGES**

DATE	DESCRIPTION	REV/CHANGE
04/29/2016	New JPM for 2016 NRC ILT Exam	0
08/12/2016	Incorporate NRC comments	0/1

#### JPM WORKSHEET

Facility: MP Unit	2 Examinee:		
JPM Number:	JPM-287-C-6	Revision: 0/1	_
Task Title: EOP	2541 App. 26 EDG Operations Lo	w Oil pressure	
System: 064	Emergency Diesel Generators		-
Time Critical Task:	YES X NO		
Validated Time (minu	tes): <u>25</u>		
Task Number(s):	064-010-075		
Applicable To:	SRO X STA	ROX	PEO X
K/A Number:	A 1.01 K/A Rating:	3.0 / 3.1	
Method of Testing:	Simulated Performance: X	Actua	l Performance:
Location:	Classroom: Sin	nulator:	In-Plant: X
Task Standards:	Monitor EDG during operations low lube oil pressures and either	e e .	
<u>Simulator</u> <u>Requirements</u> :	None		
General References:	MP-PROC-OPS-EOP 2541-APP MP-PROC-OPS-ARP 2591A-00		
	MP-PROC-OPS-ARP 2591B-00	2[r006.00] C-39 Alar	m Response

## \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

# JPM WORKSHEET JPM-287-C-6 JPM Number: Revision : 0/1 Initial Conditions: The Reactor Tripped from 100% power due to a loss of the Grid. Both Emergency Diesels are running aligned to their respective Buses. The Crew is performing EOP 2528 Loss of Offsite Power. Initiating Cues: You have been directed to perform EOP 2541 Standard Appendix 26 "Emergency Diesel Operation" and locally check diesel operation. **Required Materials:** (procedures, equipment, etc.) MP-PROC-OPS-EOP 2541-APP26[r001.00] EDG Operations MP-PROC-OPS-ARP 2591A-033[r001.04] C-38 Alarm Response MP-PROC-OPS-ARP 2591B-002[r006.00] C-39 Alarm Response

## \* \* \* \* <u>NOTES TO TASK PERFORMANCE EVALUATOR</u> \* \* \* \*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

	JPM Number: <b>JPM-287-C-6</b>	Revision:	0/1
	Task Title: EOP 2541 App. 26 ED	G Operations Low Oil pressure	
		S	START TIME:
S T E P # 1	Performance: EOP 2541-APP26 Step #1 OBSERVE EDG alarms. (C-38, C-39)	Standard: Examinee Reviews the Appendix	Critical:Grade $Y \square N \boxtimes$ $S \square U \square$
	Cue: Provide Examinee with a copy of EOP 2541-APP. Comments:	26	
STEP #2	Performance: EOP 2541-APP26 Step #2 RESET and ACKNOWLEDGE the alarms.	Standard: The Examinee presses the Reset then Acknowledge button on C-38 or C-39 NOTEs that the C-38 / C-39 B-1 "LUBE OIL PRESSURE LOW" Alarm remains lit	Critical:GradeYNSU
	Cue: After the Examinee states that they press the Ackr B-1 Alarm remains lit. Comments:	nowledge and the Reset Button, Provide feedback that '	LUBE OIL PRESSURE LOW"
STEP #3	Performance: EOP 2541-APP26 Step #3 NOTIFY the Control Room of alarm panel status.	Standard: The Examinee calls the Control Room and reports C-38 or C-39 B-1 "LUBE OIL PRESSURE LOW" Alarm remains lit	Critical:GradeYNSU
	Cue: Acknowledges as Control room "Understand Low Comments:	Lube Oil Pressure Alarm is in, Perform local Alarm R	esponse Panel Procedure".
S T E P # 4	Performance: Reviews the Alarm Response for B-1 ARP 2591A/B-002 Cue:	Standard: Examinee removes and obtains the ARP for the Low Lube Oil Pressure Alarm	Critical:GradeYNSU
	Hand the Examinee a copy of ARP 2591A or B -0 Comments:	02 for the corresponding EDG	

JPM Number: **JPM-287-C-6** 

Revision: 0/1

## Task Title:EOP 2541 App. 26 EDG Operations Low Oil pressure

S T E P # 5	Performance: Reviews the ARP for Set Points, Automatic Functions and Corrective Actions	Standard: Examinee notes the Lube Oil Pressure Setpoints and the Automatic Functions	Critical: Y 🗌 N 🔀	Grade S 🗌 U 🗌
	Cue: Comments:			
STEP #6	Performance: CHECK oil sump level meets one of the following: IF DG is operating, between "ADD OIL" and "FULL" mark on dipstick	Standard: Examinee goes to the north side of the EDG and locates the Dipstick and states they would unscrew counter clockwise and remove the dipstick checking oil level	Critical: Y □ N ⊠	Grade S 🗌 U 🗌
	Cue: Inform the Examinee oil level is half way between Comments:	n the "ADD" and "FULL" mark on the dipstick		

JPM Number: **JPM-287-C-6** 

Revision: 0/1

## Task Title:EOP 2541 App. 26 EDG Operations Low Oil pressure

STEP #7	<ul> <li>Performance:</li> <li>IF DG is operating, OBSERVE the following: <ul> <li>Pressure indicated on PI-8755/8757 (engine skid) (normally 30 to 40 psig).</li> <li>Upstream and downstream lube oil filter pressure indicated on PI-8759/8760 (normally 62 to 72 psig)</li> <li>Upstream and downstream lube oil strainer pressure indicated on PI-8765/8766 (normally 45 to 60 psig)</li> </ul> </li> <li>Cue: When the Examinee examines the lube oil gages p</li> </ul>	turns selector handle for PI-8759 / 8760 to upstream and reads 10 psig and turns selector handle to downstream reads 8 psig Examinee goes to the Southside of the EDG skid and turns selector handle for PI-8765 / 8766 to upstream and reads 10 psig and turns selector handle to downstream reads 8 psig				
	<ul> <li>PI-8755 reads 10 psig</li> <li>PI-8759 reads upstream 10 psig and downstream 8 psig</li> <li>PI-8765 reads upstream 10 psig and downstream 8 psig</li> <li>Comments:</li> </ul>					
STEP #8	Performance: The Examinee NOTEs that pressure indications read less than 10 psig and are less than require for operations.	Standard: Examinee recommends to the Control Room the need to trip the EDG or Manually Trips the EDG by Depressing the Fuel Rack Trip or Tripping or isolating the Fuel Supply.	Critical: Y ⊠ N □	Grade S 🗌 U 🗌		
	Cue:					
	Acknowledge Report as Control room and then inf	form examinee that the EDG Trip and the RPMS are re	ducing.			
	Comments:					
	Rack or isolation of Fuel.	e Trip Button on C-38 / 39 the EDG does not TRIP, m	*			
	If the Examinee Trips the EDG this concludes the	JPM or JPM continues step #11 if the Examinee fails t	o locally trip the I	EDG.		

JPM Number: **JPM-287-C-6** 

Revision: 0/1

# Task Title:EOP 2541 App. 26 EDG Operations Low Oil pressure

S T E P	Performance:	Standard:	Critical:	Grade
<b># 9</b>	Visually INSPECT system for oil leakage or	Examinee walks around the EDG looking for oil	Y 🗌 N 🖂	S 🗌 U 🗌
	broken oil lines.	leaks.		
	Cue:			
	No Oil Leaking Comments:			
	Reason for the Low Lube Oil pressure is the Engin	e Driven Oil Pump shaft sheared		
STEP	Performance:	Standard:	Critical:	Grade
	WHEN alarming condition is clear, PRESS			
#10	"ALARM RESET" button (engine skid).	Examinee attempts to reset alarms on C-38 / C-39		
	Cue: Alarms do not reset			
	Comments:			
S T E P	Performance:	Standard:	Critical:	Grade
#11	SUBMIT CR.	Examinee state they would submit a CR	Y 🗌 N 🔀	S 🗌 U 🗌
	Cue:			
	Comments:			
STEP	Performance:	Standard:	Critical:	Grade
#12	Refer To Technical Specification LCOs,	States that the EDG is NOT OPERABLE	$Y \square N \boxtimes$	$S \square U \square$
	3.8.1.1 and 3.8.1.2, and DETERMINE			
	applicability.			
	Cue:			
	Comments:			

TERMINATION CUE: The evaluation for this JPM is concluded.

**STOP TIME**: \_\_\_\_\_

## **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-287-C-6

Revision: 0/1

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🖾 No			
Validated Time (minutes):	25	Actual Time to Complete (minutes):			
Work Practice Performance:		SAT	UNSAT		
Operator Fundamentals:		SAT	UNSAT		
JPM Question Portion Overall [ <i>NLO only</i> ]:		SAT	UNSAT		N/A
Attached Question #1		SAT	UNSAT		
Attached Question #2		SAT	UNSAT		
Overall Result of JPM:		SAT	UNSAT		

Evaluator:

Print / Sign

Areas for Improvement / Comments:

## JPM QUESTIONS

Question #1:	If the Emergency Diesel was started due to a loss of normal power signal, how many lube oil low pressure switches must actuate to trip the diesel?
Answer #1:	If an emergency start signal is present, two of three pressure switches must actuate to trip diesel. ARP 2591A-002 / ARP 2591B-002 "Automatic Functions"
<u>Examinee</u> <u>Response</u> :	
Grade:	SAT UNSAT

Question #2:	When the diesel engine is running what supplies lube oil pressure?
Answer #2:	Engine driven lube oil pump
<u>Examinee</u> <u>Response</u> :	
Grade:	SAT UNSAT

## STUDENT HANDOUT

JPM Number:	JPM-287-C-6	Revision:	0/1		
Initial Conditions:					
	The Reactor Tripped from 100% power due to a loss of the Grid.				
	Both Emergency Diesels are running aligned to their respective Buses.				
	The Crew is performing EOP 2528 Loss of Offsite Power.				
Initiating Cues:					
	You have been directed to perform EOP 2541 Standard Appendix 26 "Emergency Diesel Operation" and locally check diesel operation.				

# JOB PERFORMANCE MEASURE APPROVAL SHEET

PM Number:	JPM-245	Revision: 1/1
nitiated:		
	Robert L. Cimmino, Jr.	05/04/2016
	Developer	Date
Reviewed:		
	David J. Jacobs	07/05/2016
	Technical Reviewer	Date
Approved:		
	Supervisor, Nuclear Training	Date

# **SUMMARY OF CHANGES**

DATE	DESCRIPTION	<b>REV/CHANGE</b>
05/04/2016	Updated to latest template and procedure revision	1/0
08/11/2016	Incorporate NRC changes	1/1

#### JPM WORKSHEET

Facility: MP2	Examinee:			
JPM Number:	JPM-245	Revision: <u>1/1</u>	-	
Task Title: Placing	CAR RBCCW Valve In Manua	al Local Operation.		
System: RBCCV	V		-	
Time Critical Task:	🗌 YES 🖾 NO			
Validated Time (minutes)	):15			
Task Number(s):	NUTIMS # 000-04-104			
Applicable To:	SRO X STA	RO <u></u>	PEO X	
K/A Number: 022	2/A4.04 K/A Rating:	3.1/3.2		
Method of Testing: Si	imulated Performance: X	Actual	Performance:	
Location: C	lassroom: Sir	nulator:	In-Plant:	X
Task Standards:	At the completion of this JPM, the RBCCW value in manual local of		simulated placing a	in
Required Materials: (procedures, equipment, etc.)	OP 2330A, RBCCW System			
General References:	OP 2330A, Section 4.9			

### \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

### JPM WORKSHEET

JPM	Number:	JPM-245	Revision :	1/1
Initial Conditions:	<ul><li>"C" C</li><li>The C closin</li></ul>	Dant is in MODE 5. CAR Cooler has developed Control Room has attempted ing the inlet and outlet RB va -28.1C "C" CAR COOLER	d to isolate the "C" CAR alves.	cooler by
Initiating Cues:	close	JS has directed you to place 2-RB-28.1C "C" CAR CO A Section 4.9 "Manual Ope	OLER INLET ISOLATI	ON" IAW OP
Simulator Requirement	ts: N/A			

### \*\*\*\* NOTES TO TASK PERFORMANCE EVALUATOR \*\*\*\*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

	JPM Number:	JPM-245	Revision:	1/1	
	Task Title:	Placing CAR RBCCW Va	alve In Manual Local Ops.		
			2	START TIME: _	
STEP #1	Performance: 4.9.1 PERFORM the f valve in "MANUAL" a. Using Attachment 3 actions for the valve to Cue:	s, PERFORM applicable	Standard: Examinee obtains procedure OP 2330A Attachment 3	Critical: Y □ N ⊠	Grade S 🗌 U 🗌
	Cue: Comments:				
STEP #2	top of Attachment. 3.2 IF CAR cooler val ENSURE adequate RI available. 3.3 ENSURE CAR co	-	<ol> <li>Standard:</li> <li>Records valve number and function on Attachment 3.</li> <li>Examinee determines that repositioning the valve will not have an effect on RBCCW flow with the outlet valves closed. (initial conditions)</li> <li>Examinee either uses Initial Conditions or locates the valve to determine position.</li> </ol>	Critical: Y 🗌 N 🔀	Grade S 🗌 U 🗌
	in relation to the valve	e lever arm grove. For examp	etration Room, to locally determine position requires c ple if the handwheel position indicates OPEN and the te the same as the current valve position.		

JPM Number: JPM-245

Revision: 1/1

### Task Title: Placing CAR RBCCW Valve In Manual Local Ops.

STEP	Performance:	Standard:	Critical:	Grade
#3	<b>3.4</b> IF in MODE 1, 2, 3, or 4 LOG entry into the following:	Examinee determines TS and TRM not applicable. Initial Conditions plant is in MODE 5	Y 🗌 N 🔀	S 🗌 U 🗌
	3.4.1 TSAS 3.6.3.1 3.4.2 TRM 3.6.3.1			
	<b>3.5</b> IF in MODE 1, 2, or 3 (greater than or equal to 1,750 psia), AND valve to be positioned is a CAR cooler inlet valve or a CAR cooler emergency outlet valve, LOG ENTRY in TSAS 3.6.2.1.			
	Cue:			
	Comments:			
S T E P #4	Performance: <b>3.6</b> CLOSE instrument air isolation to air operator.	Standard: Examinee locates instrument air isolation near south wall and rotates handwheel in the clockwise direction.	Critical: Y ⊠ N □	Grade S □ U □
	Cue:			
	Valve is at a hard stop			
	Comments:			
S T E P # 5	Performance: <b>3.7</b> Refer to Table 5 and determine applicable fuseblock.	Standard: Examinee refers to Table 5 and identifies that fuse "CFN" on C-01R.	Critical: Y $\boxtimes$ N $\square$	Grade S 🗌 U 🗌
	Cue:			
		Control Room personnel, and that the sound of air rele	ease is heard, and	the valve is
	Comments:			
	The fuse block is located <i>inside</i> the main control b	oards, which have very limited access while at power.		

	JPM Number: JPM-245	Revision:	1/1	
	Task Title: Placing CAR RBCCW Va	lve In Manual Local Ops.		
S T E P # 6	Performance: <b>3.8</b> Loosen Allen head screw on lever arm of "air cylinder" operating shaft. Cue: Examiner states that the Allen screw is loose.	Standard: Examinee indicates that he would use the attached Allen wrench to loosen the screw.	Critical: Y 🛛 N 🗌	Grade S 🗌 U 🗌
STEP #7	Comments: Performance: 2.0 DEMOVE functions for value being placed	Standard: Examinees states they would call the Control	Critical: Y 🗌 N 🖂	Grade
# /	<b>3.9</b> REMOVE fuseblock for valve being placed in "MANUAL" (C-01R). Cue:	Room to remove Fuse Block "CFN"	ILNA	
	Respond as the Control Room "Fuse Block CFN is Comments:	s removed from C01R"		
S T E P # 8	Performance: <b>3.10</b> Operate the manual handwheel to align the manual operator shaft to valve stem for the lever arm insertion.	Standard: Examinee states that he would move the manual handwheel to align the shaft.	Critical: Y □ N ⊠	Grade S 🗌 U 🗌
	Cue: Examiner states that the shafts are now aligned.			
	Comments:		~	~ .
S T E P #9	Performance: <b>3.11</b> Loosen Allen screw on lever arm of "Manual" operating shaft and Engage arm.	Standard: Examinee states that he must access the area under the valve and loosen the allen screw. He then would engage the lever arm for the manual operator.	Critical: Y ⊠ N □	Grade S 🗌 U 🗌
	Cue: Examiner states that the lever arm is engaged.			
	Comments:			

JPM Number: JPM-245

Revision: 1/1

### Task Title: Placing CAR RBCCW Valve In Manual Local Ops.

STEP	Performance:	Standard:	Critical:	Grade				
#10	<b>3.12</b> Tighten the Allen screw for the manual lever arm.	Examinee states that he would turn Allen screw to tighten the lever arm for the manual handwheel.	Y 🖾 N 🗌	S U U				
	Cue:	ughten the level ann for the manual handwheel.						
	Examiner states that the Allen screw is tight.							
	Comments:							
S T E P	Performance:	Standard:	Critical:	Grade				
#11	<b>3.13</b> To prevent inadvertent engagement of lever arm and air operating shaft,	Examinee states that he would move the lever arm out of the way and that he may need to move the manual handwheel to relieve the tension on the arm to allow this. Also states that he would then tighten	Y 🛛 N 🗌	S U U				
	PERFORM the following:							
	<b>3.13.1</b> DISENGAGE lever arm from air cylinder shaft and HOLD in "DISENGAGED" position.	the Allen screw (clockwise) to prevent the movement of the lever arm.						
	<b>3.13.2</b> ROTATE Allen head screw <i>clockwise</i> until it maintains lever arm in "DISENGAGED" position.	OPERATES VALVE in the REVERSE direction to close valve (COUNTER CLOCKWISE)						
	<ul><li><b>3.13.3</b> RELEASE lever arm.</li><li><b>3.13.4</b> OPERATE handwheel to position valve as directed by SM or US.</li></ul>							
	Cue:							
	Examiner states that the lever arm is disengaged ar	nd Allen screw is tight.						
	Comments:							
	After this step is completed, the JPM is consider	red complete						

TERMINATION CUE: The evaluation for this JPM is concluded.

**STOP TIME**: \_\_\_\_\_

### **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-245

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🖾 No	)		
Validated Time (minutes):	15	Actual Time to C	Actual Time to Complete (minutes):		
Work Practice Performance:		SAT	UNSAT		
Operator Fundamentals:		SAT	UNSAT		
JPM Question Portion Overall [NLO only]:		SAT	UNSAT		N/A
Attached Question #1		SAT	UNSAT		
Attached Question #2		SAT	UNSAT		
Overall Result of JPM:		SAT	UNSAT		

Evaluator:

Print / Sign

Areas for Improvement / Comments:

# JPM QUESTIONS

Question #1:				
Answer #1:				
<u>Examinee</u> <u>Response</u> :				
Grade:	SAT	UNSAT		

Question #2:		
Answer #2:		
<u>Examinee</u> <u>Response</u> :		
Grade:	SAT	UNSAT

# STUDENT HANDOUT

JPM Number:	JPM-245	<u>Revision</u> :	1/1
Initial Conditions:	<ul> <li>The plant is in MODE 5.</li> <li>"C" CAR Cooler has developed a let</li> <li>The Control Room has attempted to closing the inlet and outlet RB value</li> </ul>	isolate the "C" CAR c	
Initiating Cues:	<ul> <li>2-RB-28.1C "C" CAR COOLER IN</li> <li>The US has directed you to place 2- close 2-RB-28.1C "C" CAR COOL 2330A Section 4.9 "Manual Operation</li> </ul>	RB-28.1C in manual lo ER INLET ISOLATIO	ocal control and N" IAW OP

# JOB PERFORMANCE MEASURE APPROVAL SHEET

PM Number:	JPM-265	Revision: 0	)/2
nitiated:			
	David J. Jacobs	02/14/20	)12
	Developer	Date	
Reviewed:			
	Joseph M. Amarello	02/14/20	12
	Technical Reviewer	Date	
Approved:			
	Mike J. Cote	02/24/20	12
	Supervisor, Nuclear Training	Date	

# **SUMMARY OF CHANGES**

DATE	DESCRIPTION	<b>REV/CHANGE</b>
06/26/2014 djj	Updated to new format	0/1
06/26/2016 djj	Incorporated NRC comments	0/2

JPM WORKSHEET				
Facility:   Millstone Unit 2   Examinee:				
JPM Number: JPM-265	Revision: 0/2			
Task Title:   EOP 2541 Appendix 34 Turbine Building S	ump Alignment			
System: 2336 Station Sumps and Drains				
Time Critical Task: ( ) YES (X) NO				
Validated Time (minutes):15				
Task Number(s):NUTIMS 092-01-006				
Applicable To:   SRO X   STA	RO <u>X</u> PEO			
K/A Number: 2.3.11 K/A Rating:	3.8/4.3			
Method of Testing: Simulated Performance: X	Actual Performance:			
Location: Classroom: Simu	ilator: In-Plant:X			
Task Standards:At the completion of this JPM, the examinee has simulated realigning Turbine Building Sumps to CPF.				
<u>Required Materials</u> : • MP-PROC-OPS-EOP 2541-A (procedures, equipment, etc.)	PP34 Turbine Building Sump Alignment			
General References: • MP-PROC-OPS-EOP 2541-A	PP34 REV. 000			

#### \*\*\* READ TO THE EXAMINEE \*\*\*

I will explain the initial conditions, which step(s) to simulate or discuss, and provide initiating cues. When you complete the task successfully, the objective for this JPM will be satisfied. With the exception of the questions at the end, you may use any approved reference material normally available in the Control Room, including logs. Make all written reports, oral reports, alarm acknowledgements, and log entries as if the evolution was actually being performed.

JPM Num	ıber:	JPM WORKSHEET JPM-265	Revision :	0/2
Initial Conditions:	ך • <i>ב</i>	The Unit has been Manually T Tube Rupture approximately 1 All Turbine building sumps we Steam Generator Tube Leak.	hour ago.	
Initiating Cues:	2	The Balance of Plant Operator 2541 Appendix 34 and pump to bit sump to the on service CPF	he Turbine Building E	
Simulator Requirements:	N/A			

### \* \* \* \* <u>NOTES TO TASK PERFORMANCE EVALUATOR</u> \* \* \* \*

- 1. Critical steps for this JPM are indicated by checking "Y". For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly.
- 2. When the student states what his/her simulated action/observation would be, read the appropriate "Cue".
- 3. If necessary, question student for details of simulated actions/observations (i.e. "What are you looking at?" or "What are you observing?").
- 4. Under <u>NO</u> circumstances must the student be allowed to manipulate any devices during the performance of this JPM (in-plant only).

	JPM Number: Task Title:	JPM-265	Revision: urbine Building Sump Alignment	0/2	
	Task Thie.	EOF 2341 Appendix 34 1		START TIME: _	
5 T E P #1	<ul><li>in-service CPF ta</li><li>Adequate tank influent.</li></ul>	of the following for the ank: (TK-10 or TK-11) volume exists to receive being discharged	<ul> <li>Standard:</li> <li>The examinee Verifies the following:</li> <li>TK-10 or TK-11 levels on panel "2CND-PNLCDX".</li> <li>Verifies no Discharge Placard for the TK in service to receive TB Sump effluent.</li> </ul>	Critical: Y [ ] N [ <b>X</b> ]	Grade S [ ] U [
	Comments: Indication for TK-10 a		TANK HAS ADEQUATE VOLUME" sate Demineralizer Waste Treating Panel", 2CND-PNL LVES to the TK	CDX and the on	service TK is
5 T E P # 2	tank: • "AR-81A, CC TO TK 10"(C	to the in-service CPF NDENSER PIT SUMP PF) NDENSER PIT SUMP	<ul> <li>Standard:</li> <li>Examinee states they would UNLOCK and OPEN either <ul> <li>"AR-81A, CONDENSER PIT SUMP TO TK 10"(CPF)</li> <li>"AR-81B, CONDENSER PIT SUMP TO TK 11" (CPF)</li> </ul> </li> <li>by unlocking and rotating the handwheel in the counter clockwise direction for the on service TK.</li> </ul>	Critical: Y [ <b>X</b> ] N [ ]	Grade S[]U[

S T E P	Performance:	Standard:	Critical:	Grade
#3	<ol> <li>PLACE ALL of the following Turbine Building Sump Pump handswitches in "STOP":</li> <li>Condenser Pit A, "P73A" (West)</li> <li>Condenser Pit A, "P73B" (West)</li> <li>Condenser Pit B, "P39A" (East)</li> <li>Condenser Pit B, "P39B" (East)</li> <li>Motor Driven Auxiliary SGFP Room, "P125"</li> <li>Turbine Driven Auxiliary SGFP Room, "P72A"</li> <li>Turbine Driven Auxiliary SGFP Room, "P72B</li> </ol>	Examinee refers to the initial conditions and states that Handswitches are OFF.	Y [ ] N [ <b>X</b> ]	S[]U[]
	Cue: Switch is in OFF and Green Light lit for applicable Comments:	e pumps		
S T E P # 4	<ul> <li>Performance:</li> <li>4. CLOSE "SS-25, CONDENSER PIT AND AFW SUMPS TO OIL SEPARATOR #2". (Northeast corner of condenser)</li> </ul>	Standard: Examinee locates SS-25, and closes by rotating handwheel in clockwise direction using the chain operator.	Critical: Y [ <b>X</b> ] N [ ]	Grade S[]U[]
	Cue: Valve is at a hard stop. Comments:	ing Duilding quarkand		
STEP	Located Northeast corner of condenser in the Turb Performance:	Standard:	Critical:	Grade
#5	<ul> <li>5. OPEN "AR-80, TURBINE BUILDING SUMPS TO CPF TK 10/11". (Northeast corner of condenser)</li> </ul>	Examinee locates AR-80, and opens by rotating handwheel in counter clockwise direction using the chain operator.	Y [ <b>X</b> ] N [ ]	
	Cue: Valve is at a hard stop. Comments:			
		ine Building overhead		

S T E P #6	<ul> <li>Performance:</li> <li>6. PERFORM the following to align Turbine Building Sumps for automatic operation:</li> <li>a. OBTAIN approval to operate the Turbine Building Sump Pumps in automatic.</li> </ul>	Standard: Examinee refers to the initial conditions and states that they turn the switch for the East Condenser pit sump pumps (P39A/B) to Start or Automatic.	Critical: Y [ <b>X</b> ] N [ ]	Grade S [ ] U [ ]
	Cue: Select pump Light turns RED, discharge pressure Comments:	ndicated and sump level is lowering		

TERMINATION CUE: The evaluation for this JPM is concluded.

<b>STOP TIME</b> :	
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### **VERIFICATION OF JPM COMPLETION**

JPM Number:

JPM-265

Revision: 0/2

Date Performed:

Student:

For the student to achieve a satisfactory grade, <u>ALL</u> critical steps must be completed correctly. If task is Time Critical, it <u>MUST</u> be completed within the specified time to achieve a satisfactory grade. As necessary, refer to TIG-04 for additional Pass/Fail criteria.

#### **EVALUATION SECTION:**

Time Critical Task?		🗌 Yes 🖾 No	)		
Validated Time (minutes):	15	Actual Time to Complete (minutes):			
Work Practice Performance:		SAT	UNSAT		
Operator Fundamentals:		SAT	UNSAT		
JPM Question Portion Overall [NLO only]:		SAT	UNSAT		N/A
Attached Question #1		SAT	UNSAT		
Attached Question #2		SAT	UNSAT		
Overall Result of JPM:		SAT	UNSAT		

Evaluator:

Print / Sign

Areas for Improvement / Comments:

# JPM QUESTIONS

Question #1:	What would be the Major consequence if the Turbine Building Sumps were to over flow and fill the Condenser Pit?Ref. CWS-00-C.R9Chg1
<u>Answer #1:</u>	Circulating Water pumps would trip causing a Reactor Trip if water level exceeded 10" inches above the Floor of the Condenser Pit
<u>Examinee</u> <u>Response</u> :	
<u>Grade:</u>	SAT UNSAT

Question #2:	What guidelines are used to minimize water inputs to the Condenser pit sumps during a Steam Generator Tube Leak event in progress? <b>Ref.</b> AOP 2569 Steam Generator Tube Leak Step 4.4
<u>Answer #2:</u>	<ul><li>a. ENSURE the SJAE after cooler drains are aligned to the condenser.</li><li>b. AVOID the use of mechanical vacuum pumps.</li><li>c. AVOID draining any tanks or lines to the condenser pit sumps.</li></ul>
Examinee <u>Response</u> :	
Grade:	SAT UNSAT

# STUDENT HANDOUT

JPM Number:	JPM-265	Revision: 0/2
Initiating Cues:	• The Balance of Plant Operator has dir Appendix 34 and pump the Turbine B to the on service CPF TK-10 or 11.	• •
Initial Conditions:	<ul> <li>The Unit has been Manually Tripped of Rupture approximately 1 hour ago.</li> <li>All Turbine building sumps were place Generator Tube Leak.</li> </ul>	