U.S. NUCLEAR REGULATORY COMMISSION DOCKET 50-410 LICENSE NPF-69

NINE MILE POINT NUCLEAR STATION UNIT 2

UPDATED SAFETY ANALYSIS REPORT

OCTOBER 2016

REVISION 22

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6.3-30	REACTOR VESSEL PRESSURE FOLLOWING A 1.0 SQ FT BREAK (SBM) RECIRCULATION SUCTION BREAK, HPCS FAILURE
6.3-31	FUEL ROD CONVECTIVE HEAT TRANSFER COEFFICIENT FOLLOWING A 1.0 SQ FT BREAK (SBM) RECIRCULATION SUCTION BREAK, HPCS FAILURE
6.3-32	PEAK CLADDING TEMPERATURE FOLLOWING A 1.0 SQ FT BREAK (SBM) RECIRCULATION SUCTION BREAK, HPCS FAILURE
6.3-33	WATER LEVEL INSIDE SHROUD FOLLOWING A 0.09 SQ FT BREAK (HIGHEST TEMPERATURE SMALL BREAK) RECIRCULATION SUCTION BREAK, HPCS FAILURE
6.3-34	REACTOR VESSEL PRESSURE FOLLOWING A 0.09 SQ FT BREAK (HIGHEST TEMPERATURE SMALL BREAK) RECIRCULATION SUCTION BREAK, HPCS FAILURE
6.3-35	FUEL ROD CONVECTIVE HEAT TRANSFER COEFFICIENT FOLLOWING A 0.09 SQ FT BREAK (HIGHEST TEMPERATURE SMALL BREAK) RECIRC. SUCTION BREAK, HPCS FAILURE

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Figure <u>Number</u>	<u>Title</u>
6.3-36	PEAK CLADDING TEMPERATURE FOLLOWING A 0.09 SQ FT BREAK (HIGHEST TEMPERATURE SMALL BREAK) RECIRCULATION SUCTION BREAK, HPCS FAILURE
6.3-37	DELETED
6.3-38	DELETED
6.3-39	DELETED
6.3-40	DELETED
6.3-41	WATER LEVEL INSIDE SHROUD FOLLOWING A MAXIMUM HPCS LINE BREAK, LPCS DIESEL GENERATOR FAILURE
6.3-42	REACTOR VESSEL PRESSURE FOLLOWING A MAXIMUM HPCS LINE BREAK, LPCS DIESEL GENERATOR FAILURE
6.3-43	FUEL ROD CONVECTIVE HEAT TRANSFER COEFFICIENT FOLLOWING A MAXIMUM HPCS LINE BREAK, LPCS DIESEL GENERATOR FAILURE
6.3-44	PEAK CLADDING TEMPERATURE FOLLOWING A MAXIMUM HPCS LINE BREAK, LPCS DIESEL GENERATOR FAILURE
6.3-45	WATER LEVEL INSIDE SHROUD FOLLOWING A MAXIMUM FEEDWATER LINE BREAK, HPCS FAILURE
6.3-46	REACTOR VESSEL PRESSURE FOLLOWING A MAXIMUM FEEDWATER LINE BREAK, HPCS FAILURE
6.3-47	FUEL ROD CONVECTIVE HEAT TRANSFER COEFFICIENT FOLLOWING A MAXIMUM FEEDWATER LINE BREAK, HPCS FAILURE
6.3-48	PEAK CLADDING TEMPERATURE FOLLOWING A MAXIMUM FEEDWATER LINE BREAK, HPCS FAILURE
6.3-49	WATER LEVEL INSIDE SHROUD FOLLOWING A MAXIMUM MAIN STEAM LINE BREAK INSIDE CONTAINMENT, LPCI DIESEL GENERATOR FAILURE
6.3-50	REACTOR VESSEL PRESSURE FOLLOWING A MAXIMUM MAIN STEAM LINE BREAK INSIDE CONTAINMENT, LPCI DIESEL GENERATOR FAILURE

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Figure <u>Number</u>	<u>Title</u>
6.3-51	FUEL ROD CONVECTIVE HEAT TRANSFER COEFFICIENT FOLLOWING A MAXIMUM MAIN STEAM LINE BREAK INSIDE CONTAINMENT, LPCI DIESEL GENERATOR FAILURE
6.3-52	PEAK CLADDING TEMPERATURE FOLLOWING A MAXIMUM MAIN STEAM LINE BREAK INSIDE CONTAINMENT, LPCI DIESEL GENERATOR FAILURE
6.3-53	WATER LEVEL INSIDE SHROUD FOLLOWING A MAXIMUM MAIN STEAM LINE BREAK OUTSIDE CONTAINMENT, HPCS FAILURE
6.3-54	REACTOR VESSEL PRESSURE FOLLOWING A MAXIMUM MAIN STEAM LINE BREAK OUTSIDE CONTAINMENT, HPCS FAILURE
6.3-55	FUEL ROD CONVECTIVE HEAT TRANSFER COEFFICIENT FOLLOWING A MAXIMUM MAIN STEAM LINE BREAK OUTSIDE CONTAINMENT, HPCS FAILURE
6.3-56	PEAK CLADDING TEMPERATURE FOLLOWING A MAXIMUM MAIN STEAM LINE BREAK OUTSIDE CONTAINMENT, HPCS FAILURE
6.3-57	NMP-2 DBA-HPCS DG FAILURE-APPENDIX K, HOT AND AVERAGE CHANNEL WATER LEVEL
6.3-58	NMP-2 DBA-HPCS DG FAILURE-APPENDIX K, REACTOR VESSEL PRESSURE
6.3-59	NMP-2 DBA-HPCS DG FAILURE-APPENDIX K, HEAT TRANSFER COEFFICIENT
6.3-60	NMP-2 DBA-HPCS DG FAILURE-APPENDIX K, PEAK CLADDING TEMPERATURE
6.3-61	NMP-2 0.07 FT ² -HPCS DG FAILURE-APPENDIX K, HOT AND AVERAGE CHANNEL WATER LEVEL
6.3-62	NMP-2 0.07 FT ² -HPCS DG FAILURE-APPENDIX K, REACTOR VESSEL PRESSURE
6.3-63	NMP-2 0.07 FT ² -HPCS DG FAILURE-APPENDIX K, HEAT TRANSFER COEFFICIENT
6.3-64	NMP-2 0.07 FT ² -HPCS DG FAILURE-APPENDIX K, PEAK CLADDING TEMPERATURE
6.5-1	STANDBY GAS TREATMENT SYSTEM LOGIC DIAGRAM (SHEETS 1 THROUGH 8)