TSTF

TECHNICAL SPECIFICATIONS TASK FORCE A JOINT OWNERS GROUP ACTIVITY

November 2, 2016

TSTF-16-10 PROJ0753

Attn: Document Control Desk U. S. Nuclear Regulatory Commission Washington, DC 20555-0001

SUBJECT:TSTF Comments on Draft Safety Evaluation for Traveler TSTF-542,
Revision 2, "Reactor Pressure Vessel Water Inventory Control," and
Transmittal of Editorial Corrections to TSTF-542, Revision 2.

REFERENCE: Letter from Kevin Hsueh (NRC) to the TSTF, "Draft Safety Evaluation of Technical Specifications Task Force Traveler TSTF-542, Revision 2, 'Reactor Pressure Vessel Water Inventory Control'," dated October 6, 2016 (ADAMS Accession No. ML16175A394).

On March 14, 2016, the TSTF submitted traveler TSTF-542, Revision 2, "Reactor Pressure Vessel Water Inventory Control," to the Nuclear Regulatory Commission (NRC) for review (Agencywide Documents Access and Management System (ADAMS) Accession No. ML16074A448). In the referenced letter, the NRC provided the draft Safety Evaluation (SE) for TSTF-542 for comment.

Attachment 1 contains a summary table and mark-up providing the TSTF's comments on the draft SE for TSTF-542. Attachment 2 contains a summary table and mark-up providing the TSTF's comments on the draft model SE for plant-specific adoption. Attachment 3 contains a summary table and revised pages of editorial corrections for TSTF-542, Revision 2, identified during the SE review. A complete copy of TSTF-542, Revision 2, with the editorial corrections incorporated is enclosed.



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Should you have any questions, please do not hesitate to contact us.

James R. Morris (PWROG/W)

Otto W. Gustafson (PWROG/CE)

Williams (BWROG) Ryan Joyce for/Lisa L

Michael K. Leisure (PWROG/B&W)

Jason Redd (APOG)

Attachment 1	TSTF Com	ments on	the '	TSTF-542	Draft	Safety	Evalua	tion	
A 1 0			.1 /			X 1 1	0 0 1	F 1	

- Attachment 2 TSTF Comments on the TSTF-542 Draft Model Safety Evaluation for Plant-Specific Adoption
- Attachment 3 Editorial Corrections to TSTF-542, Revision 2

Enclosure TSTF-542, Revision 2 (with editorial corrections incorporated)

cc: Michelle Honcharik, Technical Specifications Branch, NRC Alex Klein, Technical Specifications Branch, NRC

Attachment 1

TSTF Comments on the TSTF-542 Draft Safety Evaluation

Summary Table of TSTF Comments

Section and	Comment
Location	
	Generic Comment 1: The traveler revises NUREG-1433, BWR/4 STS, and NUREG-1434, BWR/6 STS. However, the NUREGs are also applicable to BWR/2, BWR/3, and BWR/5 plants. For clarity, the TSTF-542 SE, which considers only the changes to the STS NUREGs, is revised in multiple locations to refer only to the NUREGs and not to BWR/4 and BWR/6 plants.
1, page 1,	Revised the introduction to refer to a "Traveler" instead of a "Change
lines 10-14	Traveler," consistent with the traveler titles.
1, page 1, line 24	Made "specifications" lower case
2.2, page 2, line 34	Added missing word "fuel"
2.2, page 2, line 34	Changed from "control blades" to "control rods" to be consistent with the STS nomenclature.
2.2, page 2,	Revised the sentence to not imply that a potential to drain will result in loss of
line 35-36	core cooling.
2.2, page 2, line 40-44	The Modes for BWRs are combinations of reactor mode switch position and reactor coolant temperature. Changes are made to be consistent with Table 1-1 of the STS. Mode 3 for a BWR is Hot Shutdown, not Hot Standby.
2.2, page 3, line 1	Changed "can" to "will" to be consistent with the system design.
2.2, page 3, line 6-8	The Modes for BWRs are combinations of reactor mode switch position and reactor coolant temperature. Changes are made to be consistent with Table 1-1 of the STS.
2.3.1, page 4, lines 25 and 27	Revised the definition to be consistent with TSTF-542.
2.3.2.6, page 11	Formatted Note b to SR 3.5.2.2 to be consistent with TSTF-542.
2.3.2.6, page 14	Revised SR 3.5.2.3 to number subparts a and b vice c and d.
2.3.3, page	Added "licensee-controlled" Setpoint Control Program for accuracy. The
15, line 10	Allowable Value is in the licensee-controlled program, not the TS program.
2.3.3.2, page	Changed "current TS" to "current STS" as the SE is evaluating a change to the
16, line 22	STS.

Section and	Comment
Location	
2.3.3.2.2.1,	Added missing word "pressure"
page 17, line	
25	
2.3.3.2.4,	Change capitalization of "Required Actions" and "Completion Times" to be
page 25,	consistent with the rest of the document. Missing word "Required" added.
lines 5, 8,	
22-23	
3.1, page 29,	Clarifies that the Drain Time assumes that the licensee takes no action to
line23	mitigate the event.
3.4.3, page	Clarified the discussion of Action B. Note (b) under Applicability for these
34, lines 15-	functions specifically says they are required when credited in calculating Drain
17	Time.
3.4.3, page	Change capitalization of "Required Actions" and "Completion Times" to be
34, lines 46-	consistent with the rest of the document.
47	
3.4.4, page	Clarified the discussion of Action B. Note (b) under Applicability for these
35, lines 21-	functions specifically says they are required when credited in calculating Drain
23	Time.
3.4.4, page	Change capitalization of "Required Actions" and "Completion Times" to be
36, lines 28-	consistent with the rest of the document.
29	
3.5, page 39,	Change capitalization of "Required Action" to be consistent with the rest of the
line 14	document.
3.7, page 44,	The 200°F temperature for Modes 4 and 5 is bracketed in the STS. Brackets
line 8	are added to the discussion.
Bases	Revised discussion to match the Bases.
discussion,	
3.1, page 4,	
lines 16-18,	
25-26	
Bases	Capitalized the initial letter in "Pump" to reflect the function title.
discussion,	
3.1, page 4,	
line 21	
Bases	Changed "valves" to "valve." There is only a single suction valve for CST and
discussion,	for Suppression Pool.
3.1, page 4,	
line 41 and	
page 5, line	
1	

Section and	Comment
Location	
Bases	Revised discussion to match the Bases.
discussion,	
3.1, page 5,	
lines 10-12	
Bases	Revised Function numbers to match the Bases.
discussion,	
3.1, page 5,	
line 41	
Bases	Corrected reference from "minimum flow valves" to "injection valves." The
discussion,	Bases do not describe the minimum flow valves for this action.
3.1, page 6,	
line 14 and	
page 7, line	
8	
Bases	Corrected document title capitalization.
discussion,	
3.1, page 7,	
line 38	
Bases	Corrected capitalization of 10 CFR 50.54(f).
discussion,	
3.2, page 10,	
line 40	
Bases	Revised introduction as not all Bases changes on the list were related to the two
discussion,	listed topics. 3.3.6.1, Primary Containment Isolation Instrumentation, was
3.3.4, page	discussed in Section 3.2.2. 3.3.6.2, Secondary Containment Isolation
12, lines 22-	Instrumentation, was not discussed and should be listed. TS 3.3.7.1, MCREC
26	System Instrumentation, should be listed.

Draft Safety Evaluation Mark-Up

Technical Specifications Task Force 11921 Rockville Pike, Suite 100 Rockville, MD 20852

SUBJECT: DRAFT SAFETY EVALUATION OF TECHNICAL SPECIFICATIONS TASK FORCE TRAVELER TSTF-542, REVISION 2, "REACTOR PRESSURE VESSEL WATER INVENTORY CONTROL" (TAC NO. MF3487)

Dear Members of the Technical Specifications Task Force:

By letter dated March 14, 2016 (Agencywide Documents Access and Management System Accession No. ML16074A448), the Technical Specifications Task Force submitted to the U.S. Nuclear Regulatory Commission (NRC) for review and approval traveler TSTF-542, Revision 2, "Reactor Pressure Vessel Water Inventory Control." The NRC staff's draft safety evaluation (SE) of the traveler and a draft model SE are enclosed.

Twenty working days are provided to you to comment on any factual errors or clarity concerns contained in the draft SE. The final SE will be issued after making any necessary changes. The NRC staff's disposition of your comments on the draft SE will be discussed in the final SE. To facilitate the NRC staff's review of your comments, please provide a marked-up copy of the draft SE showing proposed changes and provide a summary table of the proposed changes.

If you have any questions, please contact Michelle Honcharik at 301-415-1774 or via e-mail at <u>Michelle.Honcharik@nrc.gov</u>.

Sincerely,

Kevin Hsueh, Chief Licensing Processes Branch Division of Policy and Rulemaking Office of Nuclear Reactor Regulation

Project No. 753

Enclosures: As stated

cc: See next page

Technical Specifications Task Force 11921 Rockville Pike, Suite 100 Rockville, MD 20852

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ADAMS Accession No.: Package: ML16250A231, Cover letter and draft traveler SE:	ML16175A394, draft
model SE: ML16250A206: *concurred via e-mail	NRR-106

model SE: I	nodel SE: ML16250A206; "concurred via e-mail NRR-106						
OFFICE	DPR/PLPB*	DPR/PLPB*	DSS/SRXB*	DSS/STSB*	DE/EICB*	DORL/BC	
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DATE	6/27/2016	06/27/2016	9/23/16	09/19/2016	09/23/2016	9/23/16	
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DATE	9/23/16	09/22/2016	09/21/2016	9/26/2016			

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Technical Specifications Task Force

CC:

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1	DRAFT SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
2	TECHNICAL SPECIFICATIONS TASK FORCE TRAVELER
3	TSTF-542, REVISION 2
4	"REACTOR PRESSURE VESSEL WATER INVENTORY CONTROL"
5 6 7	1.0 INTRODUCTION
, 9 10 11 12 13 14 15 16 17 18 19 20 21	By letter dated December 31, 2013 (Agencywide Document Access and Management System (ADAMS) Accession No. ML14002A112), the Technical Specifications (TS) Task Force (TSTF) submitted Change Traveler TSTF-542, "Reactor Pressure Vessel Water Inventory Control," Revision 0, for U.S. Nuclear Regulatory Commission review and approval. By letter dated September 15, 2015, the TSTF submitted Revision 1 to Change Traveler TSTF-542 (ADAMS Accession No. ML15258A850), and by letter dated March 14, 2016, submitted Revision 2 to Change Traveler TSTF-542 (ADAMS Accession No. ML15258A850), and by letter dated March 14, 2016, submitted Revision 2 to Change Traveler TSTF-542 (ADAMS Accession No. ML16074A448). Traveler TSTF-542 proposes changes to the Standard Technical Specifications (STS) and Bases for boiling water reactor (BWR) designs BWR/4 and BWR/6. ¹ The changes would be incorporated into future revisions of NUREG-1433, Volumes 1 and 2 and NUREG-1434, Volumes 1 and 2. <i>NUREG-1433 is based on the BWR/4 plant design, but is also representative of the BWR/2, /3, and, in some cases, BWR/5 designs. NUREG-1434 is based on the BWR/6 plant design, and is representative, in many cases, of the BWR/5 design.</i>
22 23 24 25	The proposed changes would replace the existing BWR/4 and BWR/6 Specifications related to "operations with a potential for draining the reactor vessel" (OPDRVs) with revised Specifications specifications for Reactor Pressure Vessel Water Inventory Control (RPV WIC).
26 27 28 29 30	Throughout this safety evaluation (SE), items that are enclosed in square brackets signify plant-specific nomenclature or values. Individual licensees would furnish site-specific nomenclature or values for bracketed items when submitting a license amendment request (LAR) to adopt the changes described in this SE.

¹ U.S. Nuclear Regulatory Commission, "Standard Technical Specifications, General Electric BWR/4 Plants," NUREG-1433, Vol. 1, "Specifications," Rev. 4.0, April 2012, ADAMS Accession No. ML12104A192.

U.S. Nuclear Regulatory Commission, "Standard Technical Specifications, General Electric BWR/4 Plants," NUREG-1433, Vol. 2, "Bases," Rev. 4.0, April 2012, ADAMS Accession No. ML12104A193.

U.S. Nuclear Regulatory Commission, "Standard Technical Specifications, General Electric BWR/6 Plants," NUREG-1434, Vol. 1, "Specifications," Rev. 4.0, April 2012, ADAMS Accession No. ML12104A195.

U.S. Nuclear Regulatory Commission, "Standard Technical Specifications, General Electric BWR/6 Plants," NUREG-1434, Vol. 2, "Bases," Rev. 4.0, April 2012, ADAMS Accession No. ML12104A196.

- 2 -

1 2.0 REGULATORY EVALUATION

3 2.1 <u>TECHNICAL SPECIFICATIONS</u>

4

5 Section IV, "The Commission Policy," of the Final Policy Statement on Technical Specifications 6 Improvements for Nuclear Power Reactors (58 Federal Register 39132), dated July 22, 1993, 7 states in part: 8 9 10 The purpose of Technical Specifications is to impose those 11 conditions or limitations upon reactor operation necessary to 12 obviate the possibility of an abnormal situation or event giving rise 13 to an immediate threat to the public health and safety by 14 identifying those features that are of controlling importance to 15 safety and establishing on them certain conditions of operation 16 which cannot be changed without prior Commission approval. 17 [T]he Commission will also entertain requests to adopt portions of 18 the improved STS [(e.g., TSTF-542)], even if the licensee does 19 not adopt all STS improvements... 20 The Commission encourages all licensees who submit Technical 21 Specification related submittals based on this Policy Statement to 22 emphasize human factors principles... 23 In accordance with this Policy Statement, improved STS have 24 been developed and will be maintained for [the BWR/4 and 25 BWR/6 designs]. The Commission encourages licensees to use 26 the STS as the basis for plant-specific Technical Specifications... 27 [I]t is the Commission intent that the wording and Bases of the 28 improved STS be used [] to the extent practicable. 29 30 31 2.2 SYSTEM DESCRIPTION 32 33 The BWR reactor pressure vessels have a number of penetrations located below the top of 34 active *fuel* (TAF). These penetrations provide entry for control bladesrods, recirculation flow. 35 and shutdown cooling. Since these penetrations are below the TAF, this creates a gives 36 potential to drain the reactor vessel water inventory and thus-lose effective core cooling. The 37 loss of water inventory and effective core cooling can potentially lead to fuel cladding failure and 38 radioactive release. 39 40 During operation in Modes 1 (Power Operation with reactor mode switch position - Reactor 41 Mode Switch in runRun), 2 (Startup with reactor mode switch position- Reactor Mode Switch in 42 refuel Refuel (with all reactor vessel head closure bolts fully tensioned) or startup/hot standbyStartup/Hot Standby), and 3 (Hot Standby with reactor mode switch positionShutdown -43 44 -Reactor Mode Switch in shutdown). Run and average reactor coolant temperature > [200] °F).

45 the TS for instrumentation and emergency core cooling systems (ECCS) require operability of

1 sufficient equipment to ensure large quantities of water can-will be injected into the vessel

- 2 should level decrease below the preselected value. These requirements are designed to
- 3 mitigate the effects of a loss-of-coolant accident (LOCA), but also provide protection for other
- 4 accidents and transients that involve a water inventory loss.
- 5

6 During BWR operation in Mode 4 (Cold Shutdown - Reactor Mode Switch in Shutdown with all 7 reactor vessel head closure bolts fully tensioned and with average reactor coolant temperature 8 ≤ [200] °F), and Mode 5 (Refueling - with oOne or more reactor vessel head closure bolts less 9 than fully tensioned and Reactor Mode Switch in Shutdown or Refuel), the pressures and 10 temperatures that could cause a LOCA are not present. During certain phases of refueling 11 (Mode 5) a large volume of water is available above the RPV (i.e., the RPV head is removed, the water level is ≥ [23 feet] over the top of the RPV flange, and the spent fuel storage pool 12 13 gates are removed for BWR/4 plants in NUREG-1433, or the upper containment pool is 14 connected to the RPV for BWR/6 plants in NUREG-1434.

15

16 The large volume of water available in and above the RPV (during much of the time when in 17 Mode 5) provides time for operator detection and manual operator action to stop and mitigate an 18 RPV draining event. However, typically at other times during a refueling outage, during cold 19 shutdown (Mode 4) or refueling (Mode 5), there may be a potential for significant drainage paths 20 from certain outage activities, human error, and other events when it is more likely to have some 21 normally available equipment, instrumentation, and systems inoperable due to maintenance and 22 outage activities. There may not be as much time for operator action as compared to times 23 when there are large volumes of water above the RPV.

24

In comparison to Modes 1, 2, and 3, with typical high temperatures and pressures (especially in
Modes 1 and 2), Modes 4 and 5 generally do not have the high pressure and temperature
considered necessary for a LOCA envisioned from a high energy pipe failure. Thus, while the
potential sudden loss of large volumes of water from a LOCA are not expected, operators
monitor for BWR RPV water level decrease from potential significant or even unexpected
drainage paths. These potential drainage paths in Modes 4 and 5 generally would require less
water replacement capability to maintain water above TAF.

32

To address the drain down potential during Modes 4 and 5, the current BWR STS contain specifications that are applicable during an OPDRV, or require suspension of OPDRVs if certain equipment is inoperable. The term OPDRV is not specifically defined in the TS and historically has been subject to inconsistent application by licensees. The changes discussed in this SE are intended to resolve any ambiguity by creating a new RPV water inventory control TS with attendant equipment operability requirements, required actions and surveillance requirements (SR) and deleting references to OPDRVs throughout the TS.

40 41

2.3 CHANGES TO THE STS

The proposed changes would (1) provide a definition of a new term, DRAIN TIME; (2) revise
and rename STS 3.5.2 as "Reactor Pressure Vessel Water Inventory Control;" (3) provide a new
TS 3.3.5.2, "Reactor Pressure Vessel Water Inventory Control Instrumentation;" and (4) delete
existing references to "operations with the potential to drain the reactor pressure vessel"
throughout the STS. The descriptions of the proposed changes are provided in this section.

Corresponding changes are proposed to the STS Bases. A summary of the revised STS Bases 1 2 and the staff's evaluation of the revised Bases are provided in an attachment of this SE. 3 4 5

2.3.1 Insertion of New Definition of DRAIN TIME

6 The following definition of "DRAIN TIME" would be added to Section 1.1, "Definitions" Section of the STS: 7

8				
9				
10	Th	e DRAIN TIME is the time it would take for the water inventory		
11	in a	nd above the Reactor Pressure Vessel (RPV) to drain to the		
12	top	of the active fuel (TAF) seated in the RPV assuming:		
13				
14	a)	The water inventory above the TAF is divided by the limiting		
15		drain rate;		
16				
17	b)	The limiting drain rate is the larger of the drain rate through a		
18		single penetration flow path with the highest flow rate, or the		
19		sum of the drain rates through multiple penetration flow paths		
20		susceptible to a common mode failure (e.g., seismic event,		
21		loss of normal power, single human error), for all penetration		
22		flow paths below the TAF except:		
23				
24		1. Penetration flow paths connected to an intact closed		
25		system, or isolated by manual or automatic valves <i>that</i> are		
26		locked, sealed, or otherwise secured in the closed position,		
27		blank flanges, or other devices that prevent flow or of		
28		reactor coolant through the penetration flow paths;		
29				
30		2. Penetration flow paths capable of being isolated by valves		
31		that will close automatically without offsite power prior to		
32		the RPV water level being equal to the TAF when actuated		
33		by RPV water level isolation instrumentation; or		
34				
35		3. Penetration flow paths with isolation devices that can be		
36		closed prior to the RPV water level being equal to the TAF		
37		by a dedicated operator trained in the task, who is in		
38		continuous communication with the control room, is		
39		stationed at the controls, and is capable of closing the		
40		penetration flow path isolation device without offsite power.		
41				
42	c)	The penetration flow paths required to be evaluated per		
43		paragraph b) are assumed to open instantaneously and are		
44		not subsequently isolated, and no water is assumed to be		
45		subsequently added to the RPV water inventory;		
46				
47	d)	No additional draining events occur; and		
48				

		- 0 -
1 2		e) Realistic cross-sectional areas and drain rates are used.
2 3 4 5 6		A bounding DRAIN TIME may be used in lieu of a calculated value.
7	2.3.2 <u>Chan</u>	ges to STS Section 3.5:
8 9 10	2.3.2.1	Title of TS 3.5
11 12 13	Reactor Core	ection 3.5 is being revised from "Emergency Core Cooling System (ECCS) and e Isolation Cooling System (RCIC)" to "Emergency Core Cooling Systems (ECCS), nventory Control, and Reactor Core Isolation Cooling (RCIC) System."
14 15 16	2.3.2.2	Title of TS 3.5.2
17 18 19		S 3.5.2 is being revised from "ECCS – Shutdown" to "Reactor Pressure Vessel Inventory Control."
20 21	2.3.2.3	LCO 3.5.2
22 23 24 25	injection/spra subsystem m	condition for operation (LCO) 3.5.2 currently states "Two low pressure ECCS ay subsystems shall be OPERABLE." The LCO note currently states: "One LPCI hay be considered OPERABLE during alignment and operation for decay heat pable of being manually realigned and not otherwise inoperable."
26 27 28	STS LCO 3.5	5.2 for NUREG-1433 (BWR/4-STS) -would be revised to state:
29 30 31 32		DRAIN TIME of RPV water inventory to the top of active fuel (TAF) shall be \geq 36 hours.
32 33 34		AND
35 36 37		One low pressure ECCS injection/spray subsystem shall be OPERABLE.
38 39 40	The note for	LCO 3.5.2 would be revised to state:
41 42 43 44 45 46 47		A Low Pressure Coolant Injection (LPCI) subsystem may be considered OPERABLE during alignment and operation for decay heat removal if capable of being manually realigned and not otherwise inoperable.

For NUREG-1434 (BWR/6) STS, the phrase "low pressure" is omitted because the BWR/6-high
 pressure core spray system may be used to satisfy this requirement.

4 2.3.2.4 Applicability of TS LCO 3.5.2

For NUREG-1433-(BWR/4), LCO 3.5.2 is currently applicable in MODE 4 and in MODE 5,
except with the spent fuel storage pool gates removed and water level ≥ [23 ft] over the top of
the reactor pressure vessel flange.

10 For NUREG-1434 (BWR/6), LCO 3.5.2 is currently applicable in Mode 4 and Mode 5 except 11 with the upper containment [cavity to dryer] pool [gate] removed and water level \ge [22 ft 12 8 inches] over the top of the reactor pressure vessel flange.

13

14 The applicability would be revised to be MODES 4 and 5, with no exceptions.

15 16 2.3.2.5 Actions Table of TS 3.5.2

18 The existing Actions Table of TS 3.5.2 for NUREG-1433 (BWR/4) states:

19

17

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One required ECCS	A.1 Restore required ECCS	4 hours
injection/spray	injection/spray subsystem to	
subsystem inoperable.	OPERABLE status.	
B. Required Action and	B.1 Initiate action to suspend	Immediately
associated Completion	operations with a potential for	
Time of Condition A not	draining the reactor vessel	
met	(OPDRVs).	luono diotaly
C. Two required ECCS injection/spray	C.1 Initiate action to suspend OPDRVs	Immediately
subsystems inoperable.	OFDRVS	
subsystems moperable.	AND	
	C.2Restore one ECCS injection/spray	4 hours
	subsystem to OPERABLE status	
D. Required Action C.2 and	D.1 Initiate action to restore	Immediately
associated Completion	[secondary] containment to	
Time not met	OPERABLE status.	
	AND	
	D.2[Initiate action to restore one	Immediately]
	standby gas treatment subsystem	
	to OPERABLE status.	
	AND	
		Immediately
		mmediately

D.3 Initiate action to restore isolation	
capability in each required [secondary] containment	
penetration flow path not isolated.	

The revised TS 3.5.2 Actions Table for NUREG-1433-(BWR/4) would state:

CC	NDITION	REQUIRED ACTION	COMPLETION TIME	
	Required ECCS	A.1 Restore required ECCS	4 hours	
	injection/spray subsystem	injection/spray subsystem to		
	inoperable.	OPERABLE status.		
В.	Required Action and	B.1 Initiate action to establish a	Immediately	
	associated Completion	method of water injection capable		
	Time of Condition A not	of operating without offsite		
	met.	electrical power.		
C.	DRAIN TIME < 36 hours and \geq 8 hours.	C.1Verify [secondary] containment boundary is capable of being	4 hours	
		established in less than the		
		DRAIN TIME.		
		AND		
		C.2Verify each [secondary]	4 hours	
		containment penetration flow path	- nouis	
		is capable of being isolated in		
		less than the DRAIN TIME.		
		AND		
		C.3Verify one standby gas treatment	4 hours	
		subsystem is capable of being		
		placed in operation in less than		
_		the DRAIN TIME.		
D.	DRAIN TIME < 8 hours.	D.1NOTE		
		Required ECCS injection/spray subsystem or additional method		
		of water injection shall be capable		
		of operating without offsite		
		electrical power.		
		Initiate action to establish an	Immediately	
		additional method of water		
		injection with water sources capable of maintaining RPV water		
		level > TAF for \ge 36 hours.		

Г,		1
	AND	
	D.2Initiate action to establish [secondary] containment boundary	Immediately
	AND	
	D.3Initiate action to isolate each [secondary] containment penetration flow path or verify it can be manually isolated from the control room.	Immediately
	AND	
	D.4Initiate action to verify one standby gas treatment subsystem is capable of being placed in operation.	Immediately
E. Required Action and associated Completion Time of Condition C or D not met.	E.1 Initiate action to restore DRAIN TIME to ≥ 36 hours	Immediately
<u>OR</u>		
DRAIN TIME < 1 hour		

			<u>.</u>
CONDITION		REQUIRED ACTION	COMPLETION TIME
Α.	A. One required ECCS A.1 Restore required ECCS		4 hours
	injection/spray subsystem	injection/spray subsystem to	
	inoperable.	OPERABLE status.	
Β.	Required Action and	B.1 Initiate action to suspend	Immediately
	associated Completion	operations with a potential for	
	Time of Condition A not met	draining the reactor vessel	
		(OPDRVs).	
C.	Two required ECCS	C.1 Initiate action to suspend	Immediately
	injection/spray subsystems	OPDRVs	
	inoperable.		
		AND	
		C.2Restore one ECCS	4 hours
		injection/spray subsystem to	
		OPERABLE status	

The existing Actions Table of TS 3.5.2 for NUREG-1434 (BWR/6) states:

D. Required Action C.2 and associated Completion Time not met	D.1 Initiate action to restore [secondary containment] to OPERABLE status.	Immediately
	AND	
	D.2[Initiate action to restore one standby gas treatment subsystem to OPERABLE status.	Immediately]
	AND	
	D.3Initiate action to restore isolation capability in each required [secondary containment] penetration flow path not isolated.	Immediately

The revised TS 3.5.2 ACTIONS Table for NUREG-1434 (BWR/6) would state:

CC	ONDITION	COMPLETION TIME	
	Required ECCS injection/spray subsystem inoperable.	REQUIRED ACTION A.1 Restore required ECCS injection/spray subsystem to OPERABLE status.	4 hours
В.	Required Action and associated Completion Time of Condition A not met.	B.1 Initiate action to establish a method of water injection capable of operating without offsite electrical power	Immediately
C.	DRAIN TIME < 36 hours and ≥ 8 hours.	C.1Verify [secondary containment] boundary is capable of being established in less than the DRAIN TIME.	4 hours
		C.2Verify each [secondary containment] penetration flow path is capable of being isolated in less than the DRAIN TIME.	4 hours
		AND C.3[Verify one standby gas treatment subsystem is capable of being placed in operation in less than the DRAIN TIME.	4 hours]

D.	DRAIN TIME < 8 hours.	D.1 Required ECCS injection/spray subsystem or additional method of water injection shall be capable of operating without offsite electrical power.	
		Initiate action to establish an additional method of water injection with water sources capable of maintaining RPV water level > TAF for ≥ 36 hours.	Immediately
		AND	
		D.2Initiate action to establish [secondary containment] boundary	Immediately
		AND	
		D.3Initiate action to isolate each [secondary containment] penetration flow path or verify it can be manually isolated from the control room.	Immediately
		AND	
		D.4[Initiate action to verify one standby gas treatment subsystem is capable of being placed in operation.	Immediately]
E.	Required Action and associated Completion Time of Condition C or D not met.	E.1 Initiate action to restore DRAIN TIME to ≥ 36 hours.	Immediately
	<u>OR</u>		
	DRAIN TIME < 1 hour		

2.3.2.6 TS 3.5.2 Surveillance Requirements

The NUREG-1433-(BWR/4) TS 3.5.2 currently contains the following SRs:

SURVEILLANCE	FREQUENCY

- 10 -

	Verifie for each new inclusion processing and the	[40 h av ma
SR 3.5.2.1	Verify, for each required low pressure coolant	[12 hours
	injection (LPCI) subsystem, the suppression pool	
	water level is ≥ [12 ft 2 inches].	<u>OR</u>
		In accordance with the
		Surveillance Frequency
		Control Program]
SR 3.5.2.2	Verify, for each required core spray (CS)	[12 hours
SK 3.3.2.2	subsystem, the:	
	subsystem, me.	
	a Suppression peel water level in	<u>OR</u>
	 a. Suppression pool water level is ≥ [12 ft 2 inches] or 	In accordance with the
	<i>b.</i> NOTE	Surveillance Frequency
		Control Program]
	Only one required CS subsystem may take	
	credit for this option during OPDRVS.	
	Condensate storage tank water level is ≥ [12 ft].	
SR 3.5.2.3	Verify, for each required ECCS injection/spray	[31 days
01101012.0	subsystem, the piping is filled with water from the	[or days
	pump discharge valve to the injection valve.	OR
		<u></u>
		In accordance with the
		Surveillance Frequency
		Control Program]
SR 3.5.2.4	Verify each required ECCS injection/spray	[31 days
	subsystem manual, power operated, and automatic	
	valve in the flow path, that is not locked sealed, or	OR
	otherwise secured in position, is in the correct	
	position.	In accordance with the
	P.00110111	Surveillance Frequency
		Control Program]
		oonaon rogramj

The revised SRs for NUREG-1433 (BWR/4) would be:

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SURVEILLANCE		FREQUENCY	
SR 3.5.2.1	Verify DRAIN TIME ≥ 36 hours.	[12 hours	
		OR	
		In accordance with the Surveillance Frequency Control Program]	
SR 3.5.2.2	Verify, for a required low pressure ECCS injection/spray subsystem, the suppression pool	[12 hours	
	water level is ≥ [12 ft 2 inches].	OR	

-		1
		In accordance with the Surveillance Frequency Control Program]
SR 3.5.2.3	Verify, for a required Core Spray (CS) System, the:	[12 hours
	 a. Suppression pool water level is ≥ [12 ft 2 inches] or 	<u>OR</u>
	 b. Condensate storage tank water level is ≥ [12 ft]. 	In accordance with the Surveillance Frequency Control Program]
SR 3.5.2.4	Verify, for the required ECCS injection/spray subsystem, the piping is filled with water from the pump discharge valve to the injection valve.	[31 days
		In accordance with the Surveillance Frequency Control Program]
SR 3.5.2.5	Verify, for the required ECCS injection/spray subsystem each manual, power operated, and automatic valve in the flow path, that is not locked sealed, or otherwise secured in position, is in the	[31 days <u>OR</u>
	correct position.	In accordance with the Surveillance Frequency Control Program]
SR 3.5.2.6	Operate the required ECCS injection/spray subsystem through the recirculation line for ≥ 10 minutes.	[92 days <u>OR</u>
		In accordance with the Surveillance Frequency Control Program]
SR 3.5.2.7	Verify each valve credited for automatically isolating a penetration flow path actuates to the isolation position on an actual or simulated isolation signal.	[[18] months <u>OR</u>
		In accordance with the Surveillance Frequency Control Program]
SR 3.5.2.8	NOTENOTEVessel injection/spray may be excluded.	[[18] months
		OR
	Verify the required ECCS injection/spray subsystem actuates on a manual initiation signal.	In accordance with the Surveillance Frequency Control Program]

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The corresponding NUREG-1434 (BWR/6) TS 3.5.2 currently contains the following SRs:

SURVEILLA				FREQUENCY
SR 3.5.2.1	Verify, for each required low pressure ECCS injection/spray subsystem, the suppression pool water level is ≥ [12.67 ft].		[12 hours <u>OR</u>	
				In accordance with the Surveillance Frequency Control Program]
SR 3.5.2.2	Verify, for the (HPCS) subs		ressure Core Spray	[12 hours
	a. Suppre	ession pool water l	evel is ≥ [12.67 ft] or	OR
	b. Conder	nsate storage tank	x water level is ≥ [18 ft]	In accordance with the Surveillance Frequency Control Program]
SR 3.5.2.3	subsystem, t	e required ECCS i he piping is filled irge valve to the in	with water from the	[31 days <u>OR</u>
				In accordance with the Surveillance Frequency Control Program]
SR 3.5.2.4	manual, pow flow path, tha	ver operated, and	ection/spray subsystem automatic valve in the aled, or otherwise prrect position.	[31 days <u>OR</u>
				In accordance with th Surveillance Frequency Control Program]
SR 3.5.2.5	specified flow	required ECCS pu w rate [against a s ng to the specified	ystem head	[In accordance with the Inservice Testing Program
			[System Head Corresponding to	OR
	<u>System</u>	Flow Rate	A Reactor <u>Pressure of</u>	[92 days]
	LPCS LPCI	≥[7115]gpm ≥[7450]gpm	≥[290]psig ≥[125]psig	OR
	HPCS	≥[7115]gpm	≥[445]psig	In accordance with th Surveillance

		Frequency Control
		Program]
SR 3.5.2.6	NOTE	[18months
	Vessel injection/spray may be excluded.	-
		OR
	Verify each required ECCS injection /spray	In accordance with the
	subsystem actuates on an actual or simulated	Surveillance
	automatic initiation signal.	Frequency Control
	-	Program]

The revised SRs for NUREG-1434 (BWR/6) would be:

SURVEILLANCE		FREQUENCY
SR 3.5.2.1	Verify DRAIN TIME ≥ 36 hours.	[12 hours
		0.0
		<u>OR</u>
		In accordance with the
		Surveillance
		Frequency Control
	5000	Program]
SR 3.5.2.2	Verify, for a required low pressure ECCS injection/spray subsystem, the suppression pool	[12 hours
	water level is \geq [12.67 ft].	<u>OR</u>
		In accordance with the
		Surveillance
		Frequency Control
		Program]
SR 3.5.2.3 (HPC	Verify, for a required High Pressure Core Spray S) System, the:	[12 hours
		<u>OR</u>
	ae. Suppression pool water level is \geq [12.67 ft] or	
		In accordance with the
	be. Condensate storage tank water level is	Surveillance
	≥ [18 ft].	Frequency Control Program]
SR 3.5.2.4	Verify, for the required ECCS injection/spray	[31 days
	subsystem, the piping is filled with water from the	L- ·-·/-
	pump discharge valve to the injection valve.	<u>OR</u>
		In accordance with the
		Surveillance
		Frequency Control
		Program]

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1 2 3

2.3.3 Changes to STS Section 3.3

4 Both NUREG-1433 (BWR/4) and NUREG-1434 (BWR/6) STS contain two versions of certain 5 specifications in Section 3.3, Instrumentation. One is applicable for licensees that have not 6 adopted a Setpoint Control Program (the "A" version) and the other is applicable for licensees 7 that have adopted a Setpoint Control Program (the "B" version). In the "A" version of the STS, the Allowable Value column is retained in the Instrumentation Table, and the Instrumentation 8 Table contains footnotes that provide details regarding SRs. In the "B" version of the STS, the 9 10 Allowable Value has been relocated to the *licensee-controlled* Setpoint Control Program, and 11 this column does not appear in the Instrumentation Table. Additionally, in the "B" version, the 12 footnotes that provide details regarding SRs are not necessary. This convention is retained in 13 the revised STS LCOs discussed in this section. 14

For simplicity, the description of changes in this section is presented with the A and B versionscombined.

1 2 3 4 5	2.3.3.1	Changes to STS LCOs 3.3.5.1A and 3.3.5.1B, Emergency Core Cooling System (ECCS) Instrumentation (Without and With Setpoint Control Program), respectively	
6 7 9 10 11 12 13	The STS LCOs 3.3.5.1A and 3.3.5.1B state that "the ECCS instrumentation for each Function in Table 3.3.5.1-1, [Emergency Core Cooling System Instrumentation,] shall be OPERABLE" with the applicability as stated in the table. Table 3.3.5.1-1 currently contains requirements for function operability during Modes 4 and 5 when associated ECCS subsystem(s) are required to be operable per LCO 3.5.2, "ECCS – Shutdown." Throughout this table, the applicability in Modes 4 and 5 is being deleted because the instrumentation requirements during shutdown are being consolidated into the new STS 3.3.5.2. Conforming changes are made to the ACTIONS Table of STS LCO 3.3.5.1A and 3.3.5.1B.		
14 15 16 17 18	2.3.3.2	Insertion of new STS 3.3.5.2A and 3.3.5.2B, Reactor Pressure Vessel (RPV) Water Inventory Control Instrumentation (Without and With Setpoint Control Program), respectively	
19 20 21 22 23 24	A new STS 3.3.5.2 is proposed to provide alternative instrumentation requirements manual initiation of the ECCS injection/spray subsystem required in new STS 3.5.2 automatic isolation of penetration flow paths that may be credited in the determination time. The current STSTS contain instrumentation requirements related to OPDRVs These requirements are being consolidated into new STS 3.3.5.2.		
24 25 26 27 28		The existing STS 3.3.5.2, "Reactor Core Isolation Cooling (RCIC) System Instrumentation," is being renumbered to 3.3.5.3 in order to maintain the STS numbering conventions in the NUREGs.	
20 29 30	2.3.3.2.1	New TS 3.3.5.2A and B LCO and Applicability	
31 32 33	The proposed LCO 3.3.5.2 states:		
34 35 36 37		The RPV Water Inventory Control instrumentation for each Function in Table 3.3.5.2-1 shall be OPERABLE.	
38 39	The applicability states, "According to Table 3.3.5.2-1."		
39 40 41 42	The following sections describe the instrumentation functions contained in the new Table 3.3.5.2-1.		

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1 2 3	2.3.3.2.2 NUREG-1433 BWR/4 New Table 3.3.5.2-1, RPV Water Inventory Control Instrumentation			
4 5	2.3.3.2.2.1	Function 1.a, Core Spray System, Reactor Steam Dome Pressure - Low (Injection Permissive), and		
6 7		Function 2.a, Low Pressure Coolant Injection (LPCI) System, Reactor Steam Dome Pressure - Low (Injection Permissive)		
8 9 10	These functions were moved from current STS 3.3.5.1, Function 1.c and Function 2.c. The following changes are made:			
11 12 13 14 15 16 17	Modes 4 ECCS su revised a	icability is changed. The existing STS 3.3.5.1 applicability for these functions in and 5 is modified by a note that limits the applicability to when the associated ubsystem(s) are required to be operable per LCO 3.5.2, "ECCS - Shutdown." The upplicability is Modes 4 and 5 without exception, to be consistent with the lity of new LCO 3.5.2, "RPV Water Inventory Control."		
18 19	• The num	ber of required channels per function is unchanged.		
20 21 22 23 24 25 26 27 28 29 30 31	 In the new table, a Channel Check and Channel Functional Test are required at the existing frequency. Calibration of the trip units, Channel Calibration, Logic System Functional Test, and ECCS Response Time tests are no longer required in Modes 4 and 5. 			
	to retain	CO 3.3.5.2A, the Allowable Value is revised to eliminate the low pressure limit and the high pressure limit. The RPV <i>pressure</i> is well below the lower limit in Modes 4 to the low pressure limit is not needed.		
	2.3.3.2.2.2	Function 1.b, Core Spray Pump Discharge Flow - Low (Bypass) and Function 2.b, Low Pressure Coolant Injection Pump Discharge Flow - Low (Bypass)		
32 33 34	These functions were moved from current STS 3.3.5.1, Function 1.d and Function 2.g, respectively. The following changes are made:			
35 36 37 38 39 40 41 42 43 44 45 46	Modes 4 a ECCS sul Shutdown	cability is changed. The current STS 3.3.5.1 applicability for these functions in and 5 is modified by a note that limits the applicability to when the associated osystem(s) are required to be operable per current LCO 3.5.2, "ECCS - n." The revised applicability is Modes 4 and 5 without exception, to be consistent pplicability of new LCO 3.5.2, "RPV Water Inventory Control."		
	pump], to changed f "Associate	ion 1.b, the number of required channels per function is changed from [2] or [1 per [1 per pump]. For Function 2.b, the number of required channels per function is from [4] or [1 per pump], to [1 per pump]. Both are modified by a note stating ed with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, 'Reactor Vessel Water Inventory Control.'"		

1 In the new table, a Channel Check and Channel Functional Test are required at the existing • 2 frequency. A Channel Calibration and Logic System Functional Test are no longer required 3 in Modes 4 and 5. 4 5 • In new LCO 3.3.5.2A, the allowable value is unchanged. 6 7 2.3.3.2.2.3 Function 1.c, Core Spray System, Manual Initiation, and 8 Function 2.c, Low Pressure Coolant Injection (LPCI) System, Manual Initiation 9 10 These functions were moved from current STS 3.3.5.1, Function 1.e and Function 2.h. The 11 following changes are made: 12 13 The applicability is changed. The current STS 3.3.5.1 applicability for these functions in • 14 Modes 4 and 5 is modified by a note that limits the applicability to when the associated 15 ECCS subsystem(s) are required to be operable per current LCO 3.5.2, "ECCS -Shutdown." The revised applicability is Modes 4 and 5 without exception, to be consistent 16 17 with the applicability of new LCO 3.5.2, "RPV Water Inventory Control." 18 19 The number of required channels per function is changed from [2] or [1 per subsystem], to 20 [1 per subsystem] and is modified by a note stating "Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory 21 22 Control." New LCO 3.5.2 only requires a single ECCS subsystem and the change in 23 required channels reflects that requirement. 24 25 Both the existing STS 3.3.5.1 and the revised STS 3.3.5.2 require a Logic System Functional Test on this function at the same frequency. 26 27 • There is no allowable value for this function. 28 29 30 2.3.3.2.2.4 Function 3.a, RHR System Isolation, Reactor Vessel Water Level - Low, Level 3 31 32 This function was moved from current STS 3.3.6.1, Function 6.b. The following changes are 33 made: 34 35 The function name is changed from "Shutdown Cooling System Isolation Reactor Vessel • 36 Water Level - Low, Level 3" to "Residual Heat Removal [RHR] System Isolation Reactor 37 Vessel Water Level - Low, Level 3." The current title is a misnomer in the STSs as the Level 3 instruments isolate more than shutdown cooling isolation valves. 38 39 40 The applicability is changed. The existing STS 3.3.6.1 applicability for this function in • 41 Modes 4 and 5 is being deleted. The revised applicability is "when automatic isolation of the 42 associated penetration flow path is credited in calculating Drain Time." 43 44 The number of required channels is changed from [2], with a column header that states 45 "Required Channels per Trip System," to [2 in one trip system]. This retains the requirement that the two channels must be associated with the same trip system. 46 47

2 frequency. A calibration of the trip unit, Channel Calibration, and Logic System Functional 3 Test are no longer required in Modes 4 and 5. 4 5 • The allowable value is unchanged. 6 7 Function 4.a, Reactor Water Cleanup (RWCU) System Isolation, Reactor Vessel 2.3.3.2.2.5 8 Water Level - Low Low, Level 2 9 10 This function was relocated from current STS 3.3.6.1, Function 5.e. The following changes are 11 made: 12 13 The applicability is changed. The current STS 3.3.6.1 applicability for this function is • 14 Modes 1, 2, and 3. The revised applicability is "when automatic isolation of the associated 15 penetration flow path is credited in calculating Drain Time." In other words, if the drain time 16 calculation assumes the RWCU system will be automatically isolated, this function must be 17 operable to perform that function. This is consistent with the definition of drain time and the 18 TS 3.5.2 requirements. 19 20 The number of required channels is changed from [2], with a column header that states • "Required Channels per Trip System," to [2 in one trip system]. This retains the requirement 21 22 that the two channels must be associated with the same trip system. Only one trip system is required to ensure that automatic isolation of one of the two isolation valves will occur on 23 24 low reactor vessel water level. 25 26 • A Channel Check and Channel Functional Test are required at the existing frequency. A 27 calibration of the trip unit, Channel Calibration, Logic System Functional Test, and Isolation 28 System Response Time tests are no longer required in Modes 4 and 5. 29 30 • The allowable value is unchanged. 31 32 2.2.3.2.3 NUREG-1434 BWR/6-New Table 3.3.5.2-1, RPV Water Inventory Control Instrumentation 33 34 35 2.3.3.2.3.1 Function 1.a, Low Pressure Coolant Injection-A (LPCI) and Low Pressure Core 36 Spray (LPCS) Subsystems, Reactor Steam Dome Pressure - Low (Injection 37 Permissive) and 38 Function 2.a, LPCI B and LPCI C Subsystems, Reactor Steam Dome Pressure -39 Low (Injection Permissive) 40 41 These functions were moved from current STS 3.3.5.1, Function 1.d and Function 2.d. The 42 following changes are made: 43 44 The applicability is changed. The current STS 3.3.5.1 applicability for these functions in 45 Modes 4 and 5 is modified by a note that limits the applicability to when the associated 46 ECCS subsystem(s) are required to be operable per current LCO 3.5.2, "ECCS -47 Shutdown." The revised applicability is Modes 4 and 5 without exception, to be consistent

In the new table, a Channel Check and Channel Functional Test are required at the existing

1

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1 2 3 4		1434 the	applicability of new LCO 3.5.2, "RPV Water Inventory Control." Note that <i>NUREG</i> - BWR/6 STS does not include the Mode 4 and 5 applicability of this function. This by was an oversight in development of the NUREG.			
5 6 7 8 9	•	In the new table, the number of required channels per function remains [3] and is modified by a note stating "Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory Control.'" New STS 3.5.2 only requires a single ECCS subsystem to be operable and the change reflects that requirement.				
10 11 12 13 14 15 16 17 18 19 20 21	•	Calibration of the trip units, Channel Calibration, Logic System Functional Test, and ECCS Response Time tests are no longer required in Modes 4 and 5.				
	2.3	3.3.2.3.2	Functions 1.b and 1.c, Low Pressure Coolant Injection-A (LPCI) and Low Pressure Core Spray (LPCS) Subsystems, LPCS Pump Discharge Flow - Low (Bypass) and LPCI Pump A Discharge Flow – Low (Bypass), and Function 2.b, LPCI B and LPCI C Subsystems, LPCI Pump B and LPCI Pump C Discharge Flow – Low (Bypass)			
22 23 24	These functions were moved from current STS 3.3.5.1, Function 1.e, 1.f, and 2.e. The following changes are made:					
24 25 26 27 28 29 30 31 32 33 34 35 36	•	 The applicability is changed. The current STS 3.3.5.1 applicability for these functions is Modes 4 and 5 when the associated ECCS subsystem(s) are required to be operable per LCO 3.5.2, "ECCS - Shutdown." The revised Applicability is Modes 4 and 5 without exception, to be consistent with the Applicability of new LCO 3.5.2, "RPV Water Inventory Control." 				
	•	• The number of required channels per function is changed from [1] to [1 per pump] and is modified by a note stating "Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory Control'." New STS 3.5.2 only requires a single ECCS subsystem and the change in required channels reflects that requirement.				
37 38 39 40	•	• A Channel Check and Channel Functional Test are required at the existing frequency. Calibrating the trip unit, Channel Calibration and Logic System Functional Test are no longer required in Modes 4 and 5.				
41 42	•	 In new LCO 3.3.5.2A, the allowable value is unchanged. 				
42 43 44 45 46	2.3	3.3.2.3.3	Function 1.d, Low Pressure Coolant Injection-A (LPCI) and Low Pressure Core Spray (LPCS) Subsystems, Manual Initiation, and Function 2.c, LPCI B and LPCI C Subsystems, Manual Initiation			

1 2 3	These functions were moved from current STS 3.3.5.1, Function 1.g and Function 2.f. The following changes are made:			
4 5 6 7 8 9	 The applicability is changed. The current STS 3.3.5.1 Applicability for these Functions in Modes 4 and 5 is modified by a note that limits the applicability to when the associated ECCS subsystem(s) are required to be operable per current LCO 3.5.2, "ECCS - Shutdown." The revised applicability is Modes 4 and 5 without exception, to be consistent with the applicability of new LCO 3.5.2, "RPV Water Inventory Control." 			
10 11 12 13 14 15	• The number of required channels per function is changed from [1] to [1 per subsystem] and is modified by a note stating "Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory Control'." New STS 3.5.2 only requires a single ECCS subsystem and the change in required channels reflects that requirement.			
16 17 18	 Both the existing STS 3.3.5.1 and the revised STS 3.3.5.2 require a Logic System Functional Test on this function at the same frequency. 			
19	There is no allowable value for this function.			
20 21 22	2.3.3.2.3.4 Function 3.a, High Pressure Core Spray (HPCS) System, Reactor Vessel Water Level - High, Level 8			
23 24 25	This function was moved from current STS 3.3.5.1, Function 3.c. The following changes are made:			
26 27 28 29 30 31	 The applicability is changed. The current STS 3.3.5.1 applicability for this function is Modes 4 and 5 when the associated ECCS subsystem(s) are required to be operable per existing LCO 3.5.2, "ECCS - Shutdown." The revised applicability is Modes 4 and 5 without exception, to be consistent with the applicability of new LCO 3.5.2, "RPV Water Inventory Control." 			
32 33 34 35 36 37	 The number of required channels per function is changed from [2] to [1] and is modified by note stating "Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.1 'Reactor Pressure Vessel Water Inventory Control'." New STS 3.5.2 only requires a single ECCS subsystem and the change in required channels reflects that requirement. 			
38 39 40 41	• A Channel Check and Channel Functional Test are required at the existing frequency. Calibration of the trip units, Channel Calibration, and Logic System Functional Test tests are no longer required in Modes 4 and 5.			
42 43	• The allowable value in new LCO 3.3.5.2A is unchanged.			
43 44 45 46	2.3.3.2.3.5 Function 3.b, High Pressure Core Spray (HPCS) System, Condensate Storage Tank Level – Low			

- 22 -

1 This function was moved from current STS 3.3.5.1, Function 3.d. The following changes are 2 made:

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19

The applicability is changed. The current STS 3.3.5.1 applicability for this function is
Modes 4 and 5 when the associated ECCS subsystem(s) are required to be operable per
current LCO 3.5.2, "ECCS - Shutdown." The revised applicability is Modes 4 and 5 when
HPCS is operable for compliance with new LCO 3.5.2 and aligned to the Condensate
Storage Tank. If HPCS is not being credited for meeting the new LCO 3.5.2 requirement for
an operable ECCS subsystem, or if HPCS is being credited but is aligned to the suppression
pool, this function is unneeded.

- The number of required channels per function is changed from [2] to [1]. New STS 3.5.2
 only requires a single ECCS subsystem to be operable, and the change in required
 channels reflects that requirement.
- A Channel Check and Channel Functional Test are required at the existing frequency.
 Calibration of the trip units, Channel Calibration, and Logic System Functional Test are no
 longer required in Modes 4 and 5.
- The allowable value in new LCO 3.3.5.2A is unchanged.

22 2.3.3.2.3.6
 23 Functions 3.c and 3.d, High Pressure Core Spray (HPCS) System, HPCS Pump
 24 Discharge Pressure - High (Bypass) and HPCS System Flow Rate - Low
 25

These functions were moved from current STS 3.3.5.1, Function 3.f and 3.g. The following
changes are made:

- The applicability is changed. The current STS 3.3.5.1 applicability for this function is
 Modes 4 and 5 when the associated ECCS subsystem(s) are required to be operable per
 current LCO 3.5.2, "ECCS Shutdown." The revised applicability is Modes 4 and 5 without
 exception, to be consistent with the applicability of new LCO 3.5.2, "RPV Water Inventory
 Control."
- The number of required channels per function is changed from [1] to [1 per pump] and is modified by a note stating "Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory Control'." New STS 3.5.2 only requires a single ECCS subsystem and the change in required channels reflects that requirement.
- A Channel Check and Channel Functional Test are required at the existing frequency.
 Calibration of the trip units, Channel Calibration, and Logic System Functional Test are no
 longer required in Modes 4 and 5.
- 44

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• The allowable value is unchanged.

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2.3.3.2.3.7 Function 3.e, High Pressure Core Spray (HPCS) System, Manual Initiation

- 23 -

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- This function is moved from current STS 3.3.5.1, Function 3.h. The following changes are made:
 The applicability is changed. The current STS 3.3.5.1 applicability for these functions in
- Modes 4 and 5 is modified by a note that limits the applicability to when the associated
 ECCS subsystem(s) are required to be operable per existing LCO 3.5.2, "ECCS Shutdown." The revised applicability is Modes 4 and 5 without exception, to be consistent
 with the applicability of new LCO 3.5.2, "RPV Water Inventory Control."
- 10
- The number of required channels per function is changed from [1] to [1 per subsystem] and is modified by a note stating "Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory Control'." New STS 3.5.2 only requires a single ECCS subsystem and the change in required channels reflects that requirement.
- Both the existing STS 3.3.5.1 and the revised STS 3.3.5.2 require a Logic System
 Functional Test on this function at the same frequency.
- 18There is no allowable value for this function.
- 20
 21 2.3.3.2.3.8 Function 4.a, RHR System Isolation Reactor Vessel Water Level Low, Level 3
 22
- This function was moved from current STS 3.3.6.1, Function 5.c. The following changes are
 made:
- The function name is changed from "Shutdown Cooling System Isolation Reactor Vessel Water Level - Low, Level 3" to "Residual Heat Removal System Isolation Reactor Vessel Water Level - Low, Level 3." This is a misnomer in the STSs as the Level 3 instruments isolate more than shutdown cooling isolation valves.
- The applicability is changed. The current STS 3.3.6.1 applicability for this function is Modes 4 and 5. The revised applicability is "when automatic isolation of the associated penetration flow path is credited in calculating drain time.
- The number of required channels is changed from [2], with a column header that states
 "Required Channels per Trip System," to [2 in one trip system]. This retains the requirement
 that the two channels must be associated with the same trip system. Only one trip system is
 required to ensure automatic isolation of one of the two isolation valves will occur on low
 reactor vessel water level.
- 40
- A Channel Check and Channel Functional Test are required at the existing frequency. A
 calibration of the trip unit, Channel Calibration, Logic System Functional Test, and Isolation
 System Response Time tests are no longer required in Modes 4 and 5.
- 44
- The existing allowable value is retained in new STS 3.3.5.2.
- 46

2.3.3.2.3.9 Function 5.a, Reactor Water Cleanup (RWCU) System Isolation, Reactor Vessel
 Water Level - Low Low, Level 2
 3

This function was relocated from current STS 3.3.6.1, Function 4.k. The following changes are
made:

- The applicability is changed. The current STS 3.3.6.1 applicability is Modes 1, 2, and 3.
 The applicability is "when automatic isolation of the associated penetration flow path is credited in calculating Drain Time." In other words, if the drain time calculation assumes the RWCU system would be automatically isolated, this function must be operable to perform that function. This is consistent with the definition of drain time and the new STS 3.5.2 requirements.
- The number of required channels is changed from [2], with a column header that states
 "Required Channels per Trip System," to [2 in one trip system]. This retains the requirement
 that the two channels must be associated with the same trip system. Only one trip system is
 required to ensure that automatic isolation of one of the two isolation valves will occur on
 low reactor vessel water level.
- A Channel Check and Channel Functional Test are required at the existing frequency. A
 calibration of the trip unit, Channel Calibration, Logic System Functional Test, and Isolation
 System Response Time tests are no longer required in Modes 4 and 5.
- The existing allowable value is retained in LCO 3.3.5.2A.
- 26 2.3.3.2.4 New TS 3.3.5.2A and B ACTIONS Table
- Condition A is applicable when one or more instrument channels are inoperable from
 Table 3.3.5.2-1. Required Action A.1 directs immediate entry into the condition referenced in
 Table 3.3.5.2-1 for that channel.
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Condition B is entered when the RHR system isolation and RWCU system isolation functions
 operability requirements are not met when automatic isolation of the associated penetration flow
 path is credited in calculating drain time. If the instrumentation is inoperable, Required
 Action B.1 directs an immediate declaration that the associated penetration flow path(s) are
 incapable of automatic isolation. Required Action B.2 requires an immediate calculation of drain
 time.

- 38
- 39 Condition C is entered when the Low Reactor Steam Dome Pressure Injection Permissive
- 40 Functions necessary for ECCS subsystem manual initiation operability requirements are not
- 41 met. The channel must be placed in the trip condition within one hour.42
- ForIn NUREG-1433-BWR/4s, Condition D is entered when the operability requirements for the
 Core Spray Pump Discharge Flow Low Bypass, Low Pressure Coolant Injection Pump
 Discharge Flow Low Bypass, or manual initiation of these functions operability requirements
 are not met. The Required Action is to restore the channel to operable status within 24 hours.
- 47

1 In For-NUREG-1434BWR/6s, Condition D is entered when the Condensate Storage Tank Level 2 -Low operability requirements are not met. Required Action D requires declaring the HPCS 3 inoperable and aligning the HPCS pump suction to the suppression pool within one hour. 4 5 In For-NUREG-1433BWR/4s, Condition E is entered if the required Required Action and 6 associated Completion Time of Condition C or D, are not met. Required Action E.1 requires the 7 associated low pressure ECCS injection/spray subsystem to be declared inoperable 8 immediately. 9 10 In For-NUREG-1434BWR/6s, Condition E is entered if the Reactor Vessel Water Level – High 11 Level 8 instrumentation operability requirements are not met. Required Action E.1 requires 12 declaring the HCPS system inoperable in 1 hour and restoring the channel to Operable status 13 within 24 hours. 14 15 In For-NUREG-1434BWR/6s, Condition F is entered if the LPCS Pump Discharge Flow Low 16 (Bypass), LPCI Pump A Discharge Flow Low (Bypass), LPCI Pump B and LPCI Pump C 17 Discharge Flow – Low (Bypass), HPCS Pump Discharge Pressure – High (Bypass) HPCS 18 System Flow Rate – Low – (Bypass) or Manual Initiation associated with these Functions 19 operability requirements are not met. The required action is to restore the channel to OPERABLE status within 24 hours. 20 21 22 In For-NUREG-1434BWR/6s, Condition G is entered if the required Required action Actions and associated completion Completion Ttimestime of Condition C, D, E, or F is not met. Required 23 24 Action G.1 requires the associated ECCS injection/spray subsystem to be declared inoperable 25 immediately. 26 27 2.3.3.2.5 New Surveillance Requirements SR 3.3.5.2.1, 3.3.5.2.2 and 3.3.5.3 28 29 New Table 3.3.5.2-1 specifies which SRs apply for each ECCS function. 30 31 SR 3.3.5.2.1 requires the performance of a Channel Check at a Frequency of [12 hours or in 32 accordance with the Surveillance Frequency Control Program.] 33 34 SR 3.3.5.2.2 requires the performance of a Channel Functional Test at a Frequency of [[92] 35 days or in accordance with the Surveillance Frequency Control Program.] 36 37 SR 3.3.5.2.3 requires the performance of a Logic System Functional Test at a Frequency of 38 [[18] months or in accordance with the Surveillance Frequency Control Program.] 39 40 2.3.3.3 Changes to Containment, Containment Isolation Valve and Standby Gas **Treatment System Requirements** 41 42 43 The following TS are applicable during OPDRVs and/or contain Actions to suspend OPDRVS 44 when the LCO is not met: 45 46 NUREG-1433 (BWR/4 plants) 47 3.6.1.3, Primary Containment Isolation Valves (PCIVs)

48 3.6.4.1, [Secondary] Containment

3.6.4.2, Secondary Containment Isolation Valves (SCIVs) 1 2 3.6.4.3, Standby Gas Treatment System 3 NUREG-1434 (BWR/6 plants) 4 5 3.6.1.3, Primary Containment Isolation Valves (PCIVs) 6 3.6.4.1, [Secondary] Containment 7 3.6.4.2, Secondary Containment Isolation Valves (SCIVs) 8 3.6.4.3, Standby Gas Treatment System 9 10 For each of these TS, the applicability and required action sections are being revised to delete 11 references to OPDRVs. 12 13 2.3.3.4 Changes to Control Room Habitability and Temperature Control Requirements 14 15 NUREG-1433 (BWR/4 plants) 3.7.4, [Main Control Room Environmental Control (MCREC)] System 16 17 3.7.5, [Control Room Air Conditioning (AC)] System 18 19 NUREG-1434 (BWR/6 plants) 20 3.7.3, [Control Room Fresh Air (CRFA)] System 21 3.7.4, [Control Room AC] System 22 23 These LCO's are currently applicable during OPDRVs and contain required actions to 24 immediately initiate action to suspend OPDRVs when certain conditions of the LCO are not met. 25 26 The references to OPDRVs are being deleted from the applicability and required actions of 27 these TS. 28 29 2.3.3.5 Changes to Electrical Sources Requirements 30 31 NUREG-1433 (BWR/4 plants) and NUREG-1434 (BWR/6 plants) 32 3.8.2, AC Sources - Shutdown 33 3.8.5, DC Sources - Shutdown 34 3.8.8, Inverters - Shutdown 35 3.8.10, Distribution Systems - Shutdown 36 37 These TS are applicable in Modes 4 and 5 and currently contain a required action to initiate 38 action to suspend operations with a potential for draining the reactor vessel immediately if 39 certain conditions are not met. 40 41 TS 3.8.2 currently requires, in part, with one required offsite circuit inoperable or one required 42 diesel generator inoperable, to initiate action to suspend operations with a potential for draining 43 the reactor vessel immediately. 44 45 TS 3.8.5 currently requires, in part, with one [or more] required DC electrical power 46 subsystem[s] inoperable for reasons other than an inoperable battery charger, to initiate action 47 to suspend operations with a potential for draining the reactor vessel immediately 48

TS 3.8.3 currently requires, in part, with one [or more] [required] inverter[s] inoperable, to initiate
 action to suspend operations with a potential for draining the reactor vessel immediately.

TS 3.8.10 currently requires, in part, with one or more required AC, DC, [or AC vital bus]
electrical power distribution subsystems inoperable, to initiate action to suspend operations with
a potential for draining the reactor vessel immediately.

8 These required actions are being deleted.9

10 2.4 <u>APPLICABLE REGULATORY REQUIREMENTS</u>

11 12 The regulation at 10 CFR Section 50.36(a)(1) requires an applicant for an operating license to 13 include in the application proposed technical specifications in accordance with the requirements 14 of 10 CFR 50.36. The applicant must include in the application, a "summary statement of the 15 bases or reasons for such specifications, other than those covering administrative controls." 16 However, per 10 CFR 50.36(a)(1), these technical specification bases "shall not become part of 17 the technical specifications." Per 10 CFR 50.90, whenever a holder of a license desires to 18 amend the license, application for an amendment must be filed with the Commission, fully 19 describing the changes desired, and following as far as applicable, the form prescribed for 20 original applications. 21

Additionally, 10 CFR 50.36(b) requires:

Each license authorizing operation of a ... utilization facility ... will include technical specifications. The technical specifications will be derived from the analyses and evaluation included in the safety analysis report, and amendments thereto, submitted pursuant to 10 CFR 50.34 ["Contents of applications; technical information"]. The Commission may include such additional technical specifications as the Commission finds appropriate.

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Per 10 CFR 50.92(a), in determining whether an amendment to a license will be issued to the applicant, the Commission will be guided by the considerations which govern the issuance of initial licenses to the extent applicable and appropriate.

The categories of items required to be in the TSs are provided in 10 CFR 50.36(c). As required
by 10 CFR 50.36(c)(2)(i), the TSs will include LCOs, which are the lowest functional capability
or performance levels of equipment required for safe operation of the facility. Per 10 CFR
50.36(c)(2)(i), when an LCO of a nuclear reactor is not met, the licensee shall shut down the
reactor or follow any remedial action permitted by the TSs until the condition can be met.

43

The regulations at 10 CFR 50.36(c)(2)(ii) state that LCO's must be established for each item meeting one of four criteria:

46 47

1 2 3 4	<i>Criterion 1.</i> Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary.
5 6 7 8 9	<i>Criterion 2.</i> A process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to fission product barrier integrity.
10 11 12 13 14	<i>Criterion 3.</i> A structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.
15 16 17 18 19	<i>Criterion 4.</i> A structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety.
20 21 22 23 24 25 26 27	The regulation at 10 CFR 50.36(c)(3) requires TSs to include items in the category of SRs, which are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the LCOs will be met. Also, the regulation at 10 CFR 50.36(a)(1) states that a summary statement of the bases or reasons for such specifications, other than those covering administrative controls, shall also be included in the application, but shall not become part of the TSs.
28 29 30 31 32 33 34 35	As described in the Commission's Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors, recommendations were made by NRC and industry task groups for new STS that include greater emphasis on human factors principles in order to add clarity and understanding to the text of the STS, and provide improvements to the Bases Section of Technical Specifications, which provides the purpose for each requirement in the specification. Subsequently, improved vendor-specific STS were developed and issued by the NRC in September 1992. The improved STS were published as the following NRC Reports:
36 37	- NUREG-1430, "Standard Technical Specifications, Babcock and Wilcox Plants"
38 39	- NUREG-1431, "Standard Technical Specifications, Westinghouse Plants"
40 41	- NUREG-1432, "Standard Technical Specifications, Combustion Engineering Plants"
42 43	- NUREG-1433, "Standard Technical Specifications, General Electric Plants, BWR/4"
44 45	- NUREG-1434, "Standard Technical Specifications, General Electric Plants, BWR/6"
46 47 48	These improved STS were the result of extensive technical meetings and discussions among the NRC staff, industry owners' groups, vendors, and NUMARC. The Commission recognizes the advantages of improved technical specifications. Clarification of the scope and purpose of

1 technical specifications has provided useful guidance to both the NRC and industry and has 2 served as an important incentive for industry participation in a voluntary program to improve 3 technical specifications. It has resulted in improved STS that are intended to focus licensee and 4 plant operator attention on those plant conditions most important to safety. This should also 5 result in more efficient use of agency and industry resources.

6

7 The NRC staff's guidance for review of TSs is in Chapter 16. Technical Specifications, of 8 NUREG-0800, Revision 3, "Standard Review Plan for the Review of Safety Analysis Reports for 9 Nuclear Power Plants" (SRP), dated March 2010, (ADAMS Accession No. ML100351425). As 10 described therein, as part of the regulatory standardization effort, the NRC staff has prepared 11 STS for each of the light-water reactor nuclear designs. NUREG-1433, Revision 4, contains the 12 STS for BWR/4 plants, and is also applicable to BWR/2, BWR/3, and in some cases, BWR/5 13 plants, and NUREG 1434, Revision 4, contains the STS for BWR/6 plants, and is also 14 applicable in some cases to BWR/5 plants. 15

16 3.0 **TECHNICAL EVALUATION** 17

18 3.1 DRAIN TIME DEFINITION

19 20 The proposed drain time is the time it would take the RPV water inventory to drain from the 21 current level to the TAF assuming the most limiting of the RPV penetrations flow paths with the 22 largest flow rate, or a combination of penetration flow paths that could open due to a common 23 mode failure, were to open and the licensee took no mitigating action.

24 25 The NRC staff reviewed the proposed drain time definition from the traveler. For the purpose of NRC staff considerations, the term "break" describes a pathway for water to drain from the RPV 26 27 that has not been prescribed in the "DRAIN TIME" definition proposed in TSTF-542. All RPV penetrations below the TAF are included in the determination of drain time as potential 28 29 pathways. Attachment 2 to the RAI responses dated March 14, 2016 (ADAMS Accession 30 No. ML16074A448), provided an example bounding drain time calculations for three examples: (1) water level at or below the reactor flange; (2) water level above RPV flange with fuel pool 31 32 gates installed, and; (3) water level above reactor flange with fuel pool gates removed. The 33 drain time is calculated by taking the water inventory above the break and dividing by the 34 limiting drain rate until the TAF is reached. The limiting drain rate is a variable parameter 35 depending on the break size and the reduction of elevation head above break location during 36 the drain down event. The discharge point will depend on the lowest potential drain point for 37 each RPV penetration flow path on a plant-specific basis. This calculation provides a 38 conservative approach to determining the drain time of the RPV.

39

40 Additionally, Attachment 2 to the RAI responses, provides a proposed example table to pair with 41 the drain time calculation. This table correlates the drain time (hours) to the penetration flow 42 path diameter (inches) and the reactor vessel water level (inches above the TAF). The 43 proposed example table is color coded to visually show if LCO 3.5.2 is met, or which LCO 44 condition the licensee would be in. This proposed example table provides operators with a 45 correlation to relate the calculated drain time to the RPV water level and where in the LCO the 46 operators should be. Based on these considerations, the NRC staff finds the proposed drain 47 time definition with supporting calculation and table to be acceptable.

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1 3.2 WATER SOURCES

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The proposed LCO 3.5.2 *in NUREG-1433* states that for BWR/4 TSs, one low pressure Emergency Core Cooling System (ECCS) injection/spray subsystem shall be OPERABLE. *The NUREG-1434 LCO 3.5.2 LCO states* For BWR/6 TSs, one ECCS injection/spray subsystem shall be OPERABLE. It should be noted that the term "low pressure" does not appear in the BWR/6NUREG-1434 LCO because the BWR/5 and BWR/6 High Pressure Core Spray (HPCS) System may *also* be used to satisfy the LCO.

9

10 The NRC staff reviewed the water sources that would be applicable to the proposed TS 3.5.2. 11 The ECCS pumps are high-capacity pumps, with flow rates of thousands of gallons per minute (gpm). Most RPV penetration flow paths would have a drain rate on the order of tens or 12 13 hundreds of gpm. The automatic initiation of an ECCS pump would provide the necessary 14 water source to counter these expected drain rates. The LPCI subsystem is to be considered 15 operable during alignment and operation for decay heat removal if capable of being manually 16 realigned and not otherwise inoperable. Decay heat removal in MODEs 4 and 5 is not affected 17 by the proposed change in TSTF-542 as these requirements on the number of RHR shutdown 18 cooling subsystems that must be operable and in operation to ensure adequate decay heat 19 removal from the core are unchanged. These requirements can be found in the BWR/4 20 STSNUREG-1433 TS 3.4.9, "Residual Heat Removal (RHR) Shutdown Cooling System - Cold 21 Shutdown," TS 3.9.8, "Residual Heat Removal (RHR) – High Water Level, " and TS 3.9.10, "Residual Heat Removal (RHR) - Low Water Level." For the BWR/6 STSNUREG-1434, the 22 23 applicable TS are TS 3.4.10. "Residual Heat Removal (RHR) Shutdown Cooling System - Cold 24 Shutdown," TS 3.9.8, "Residual Heat Removal (RHR) – High Water Level, and TS 3.9.10, "Residual Heat Removal (RHR) - Low Water Level." Based on these considerations, the NRC 25 26 staff finds the water sources provide assurances that the lowest functional capability required for 27 safe operation is maintained and protecting the safety limit.

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3.3 TS 3.5.2 – REACTOR PRESSURE VESSEL (RPV) WATER INVENTORY CONTROL

The proposed TS 3.5.2, "Reactor Pressure Vessel (RPV) Water Inventory Control," LCO
contains two parts. The first part states that DRAIN TIME of RPV water inventory to the top of
active (TAF) shall be ≥ 36 hours, and the second part states *in NUREG-1433*that for BWR/4,
one low pressure ECCS injection/spray subsystem shall be OPERABLE, and *in NURG-1434*for
BWR/6, one ECCS injection/spray subsystem shall be OPERABLE. The proposed applicability
for TS 3.5.2 is MODEs 4 and 5.

- 37
 38 The NRC staff reviewed the proposed STS 3.5.2, focusing on ensuring the fuel remains covered
 39 with water and the changes made compared to the current STS. The proposed STS 3.5.2
- 40 contains Conditions A through E based on either required ECCS injection/spray subsystem
- 41 operability or drain time.
- 42
- 43 The current STS LCO for BWR/in NUREG-1433 and NUREG-1434 4 and BWR/6 plants
- 44 *statesstate* that two ECCS injection/spray subsystems shall be operable, whereas the proposed
- LCO 3.5.2 states that only one ECCS injection/spray subsystem shall be operable. This change
- 46 is reflected in Condition A. The change from two ECCS injection/spray subsystem to one ECCS
- 47 injection/spray subsystem is because this redundancy is not required. With one ECCS
- 48 injection/spray subsystem and non-safety related injection sources, defense-in-depth will be

1 maintained. The defense-in-depth measure is consistent with other events considered during 2 shutdown with no additional single failure assumed. The drain time controls, in addition to the 3 required ECCS injection/spray subsystem, provide reasonable assurance that an unexpected 4 draining event can be prevented or mitigated before the RPV water level would be lowered to 5 the TAF.

5 6

The proposed Condition A states that if the required ECCS injection/spray subsystem is inoperable, it is to be restored to operable status within 4 hours. Proposed Condition B states that if Condition A is not met, a method of water injection capable of operating without offsite electrical power should be established immediately. The proposed Condition B for TS 3.5.2 is different from the STS, which states to initiate action to suspend OPDRVs. The proposed Condition B provides adequate assurance of an available water source should Condition A not be met within the 4-hour completion time.

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15 The proposed Condition C states that for a drain time < 36 hours and \geq 8 hours, to (1) verify 16 [secondary containment] boundary is capable of being established in less than 4 hours, and 17 (2) verify each [secondary containment] penetration flow path is capable of being isolated in less 18 than 4 hours, and (3) verify one standby gas treatment subsystem is capable of being placed in 19 operation in less than 4 hours. The current STS Condition C states if two ECCS injection/spray 20 subsystem are inoperable then restore one to operable status within 4 hours. The proposed 21 Condition C provides adequate protection should the DRAIN TIME be < 36 hours and \geq 8 hours 22 because of the ability to establish secondary containment, isolate additional flow paths, and 23 have the standby gas treatment subsystem operable.

24

25 The proposed Condition D states that when drain time < 8 hours to (1) immediately initiate 26 action to establish an additional method of water injection with water sources capable of 27 maintaining RPV water level > TAF for \geq 36 hours, (2) immediately initiate action to establish 28 [secondary] containment boundary, (3) immediately initiate action to isolate each [secondary] 29 containment penetration flow path or verify it can be manually isolated from the control room, 30 and (4) immediately initiate action to verify one standby gas treatment subsystem is capable of 31 being placed in operation. Additionally, there is a note stating that required ECCS 32 injection/spray subsystem or additional method of water injection shall be capable of operating 33 without offsite electrical power, which is similar to proposed Condition B. The current STS for 34 Condition D are similar to the proposed for when Required Action C.2 is not met. The proposed 35 Condition D provides adequate protection should the DRAIN TIME be < 8 hours because of the 36 ability to establish secondary containment, isolate additional flow paths, and have the standby 37 gas treatment subsystem operable.

38

The proposed Condition E states that when the required action and associated completion time of Condition C or D is not met, or the drain time is < 1 hour, then initiate action to restore drain time to \ge 36 hours immediately. The proposed Condition E is new, as it is not present in the current BWR/4 or BWR/6 STS. The proposed Condition E is acceptable as it provides the necessary step to restore the drain time to \ge 36 hours should the other conditions not be met, or if the drain time is < 1 hour.

45

46 Based on the NRC staff's review, the proposed changes to TS 3.5.2 are acceptable based on

the actions taken to mitigate the water level reaching the TAF with the water sources available and maintaining drain time \geq 36 hours. The LCO correctly specifies the lowest functional capability or performance levels of equipment required for safe operation of the facility. There is

- reasonable assurance that the required actions to be taken when the LCO is not met can be
 conducted without endangering the health and safety of the public.
- 4

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5 The existing TS 3.3.5.2, "RCIC System Instrumentation," is renumbered as TS 3.3.5.3. This 6 increases consistency within the BWR TS as the Reactor Core Isolation Cooling (RCIC) System 7 is discussed in the section on TS 3.5.3.

8 9

3.4 <u>STS 3.3.5.2, REACTOR PRESSURE VESSEL WATER INVENTORY CONTROL</u> INSTRUMENTATION

10 11

12 The proposed TS and associated LCO in TS Section 3.3, "Instrumentation," contains A and B 13 versions of TS 3.3.5.2. The A version is for TS without a Setpoint Control Program and 14 Table 3.3.5.2-1 has a column for listing Allowable Value. The B version is for TS with a Setpoint 15 Control Program and Table 3.3.5.2-1 has no allowable value column, because the Setpoint 16 Control Program dictates the setpoint value. In a like manner the associated LCO 3.3.5.2 has A 17 and B versions. The actions and SRs for both versions A and B are the same in NUREG-1433 18 and NUREG-1434for BWR/4 and BWR/6. 19 20 The purpose of the RPV Water Inventory Control Instrumentation is to support the requirements

of new STS LCO 3.5.2, and the definition of drain time. There are instrumentation and controls and their signal functions that are required for manual initiation or required as a permissive or operational controls on the equipment of the systems that provide water injection capability, certain start commands, and isolation functions. These instruments are required to be operable if the systems that provide water injection and isolation functions are to be considered operable as described in the safety evaluation of new STS 3.5.2. In some cases the reactor operators have alternate, often more complex means, of starting and injecting water than the preferred

- simple push button start.
- 29

30 Specifically, the NUREG-1433 BWR/4 the RPV Water Inventory Control Instrumentation

supports operation of the Core Spray and LPCI including manual initiation when needed as well
as the system isolation of the RHR system and the RWCU system. The equipment involved
with each of these systems is described in the safety evaluation of TS 3.5.2 and the Bases for
LCO 3.5.2.

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36 Specifically, the *NUREG-1434* <u>BWR/6 the</u> RPV Water Inventory Control Instrumentation 37 supports operation of the LPCI with subsystems LPCI A, LPCI B, and LPCI C, LPCS, and 38 HDCS including manual initiation when needed as well as the system inclution of the DHP

HPCS, including manual initiation when needed as well as the system isolation of the RHR
 system and the RWCU system. The equipment involved with each of these systems is

- 40 described in the safety evaluation of TS 3.5.2 and the Bases for LCO 3.5.2.
- 41

42 TSTF-542, Section 3.3, "Proposed TS 3.3.5.2, Reactor Pressure Vessel Water Inventory

43 Control Instrumentation," describes and justifies the instrumentation requirements associated

44 with and needed to support TS 3.5.2 and LCO 3.5.2, "Reactor Pressure Vessel Water Inventory

45 Control." Section 3.3.1 addresses the proposed TS 3.3.5.2 LCO and applicability. Section 3.3.2

46 addresses, presents, discusses, and justifies the proposed actions of TS 3.3.5.2. Section 3.3.3,

- 47 addresses the proposed TS 3.3.5.2 surveillances. Section 3.3.4 addresses, presents,
- discusses, and justifies the proposed Table 3.3.5.2-1. The NRC staff finds the instrumentation

1 2 and actions required to support TS 3.3.2, as presented in Section 3.3, sufficient and necessary as discussed below.

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3.4.1 Proposed TS 3.3.5.2 LCO and Applicability

7 The proposed LCO 3.3.5.2 states, "The RPV Water Inventory Control instrumentation for each
8 Function in Table 3.3.5.2-1 shall be OPERABLE."
9

10 The applicability states, "According to Table 3.3.5.2-1."

12 Section 3.3.1 of TSTF-542, states:

- 14 15 Table 3.3.5.2-1 contains those instrumentation Functions needed 16 to support manual initiation of the ECCS injection/spray 17 subsystem required by LCO 3.5.2, and automatic isolation of 18 penetration flow paths that may be credited in a calculation of 19 Drain Time. The Functions in Table 3.3.5.2-1 are moved from 20 existing TS 3.3.5.1, "ECCS Instrumentation," and TS 3.3.6.1, 21 "Primary Containment Isolation Instrumentation" Functions that 22 are required in Modes 4 or 5 or during OPDRVs. Creation of TS 3.3.5.2 places these Functions in a single location with 23 24 requirements appropriate to support the safety function for 25 TS 3.5.2. 26
 - If plant-specific design and TS require different functions to support manual initiation of an ECCS subsystem, those functions should be included in TS 3.3.5.2.
- 32 3.4.2 Proposed TS 3.3.5.2 Actions for BWR/4 and BWR/6
- 34 TS 3.3.5.2 contains actions to be followed when the LCO is not met.

36 Section 3.3.2, "Proposed TS 3.3.5.2 Actions," of TSTF-542, presents, discusses, and justifies 37 the actions of TS 3.3.5.2 and LCO 3.3.5.2. The NRC staff finds these actions are sufficient and 38 necessary, because when one or more instrument channels are inoperable the equipment and 39 function controlled by these instruments cannot complete the required function in the normal 40 manner and these actions direct the licensee to take appropriate actions as necessary and 41 enter immediately into the Conditions referenced in Table 3.3.5.2-1. These actions satisfy the 42 requirements of 10 CFR 50.36(c)(2)(i) by providing a remedial action permitted by the TS until the LCO can be met. The remedial actions provide reasonable assurance that an unexpected 43 44 draining event can be prevented or mitigated before the RPV water level would be lowered to 45 the TAF. 46

- 47 3.4.3 Proposed TS 3.3.5.2 Actions for BWR/4NUREG-1433
- 48

- 34 -

The following summarizes the proposed actions of Section 3.3.2 for BWR/4NUREG-1433.

1 2

Section 3.3.2, "Proposed TS 3.3.5.2 Actions," of TSTF-542, Revision 2, presents, discusses,
and justifies the actions of TS 3.3.5.2 and LCO 3.3.5.2. The NRC staff finds these actions are
sufficient and necessary, because when one or more instrument channels are inoperable the
equipment and function controlled by these instruments cannot complete the required function
in the normal way, and these actions direct the licensee to take appropriate actions as required.
The actions provide reasonable assurance that an unexpected draining event can be prevented
or mitigated before the RPV water level would be lowered to the TAF.

10

11 Action A is applicable when one or more instrument channels are inoperable from

12 Table 3.3.5.2-1 and directs the licensee to immediately enter the Condition referenced in 13 Table 3.3.5.2-1 for that channel.

14

Action B (concerning the RHR system Isolation and RWCU system Isolation functions) areis applicable when automatic isolation of the associated penetration flow path is credited as not having to be considered. as a path for potential drainage in calculating drain time. If the instrumentation is inoperable, Required Action B.1 directs an immediate declaration that the associated penetration flow path(s) are incapable of automatic isolation. Required Action B.2 requires a re-calculation of drain time, but automatic isolation of the affected penetration flow paths cannot be credited.

22

23 Action C (concerning low reactor steam dome pressure permissive Functions necessary for 24 ECCS subsystem manual initiation) addresses an event in which the permissive is inoperable 25 and manual initiation of ECCS using the control board pushbuttons is prevented. The function 26 must be placed in the trip condition within one hour. With the permissive function instrument in 27 the trip condition, manual initiation may now be performed using the preferred control board 28 pushbuttons. This one-hour completion time is acceptable, because despite the preferred start 29 method being prevented, the reactor operator can take manual control of the pump and the 30 injection valve to inject water into the RPV and achieve the safety function. The time of one hour also provides reasonable time for evaluation and placing the channel in trip. 31 32

33 Action D (concerning pump discharge flow bypass Functions and the manual initiation 34 Functions) addresses actions when the bypass is inoperable and then there is a risk that the 35 associated ECCS pump could overheat when the pump is operating and the associated 36 injection valve is not fully open. In this condition, the operator can take manual control of the 37 pump and the injection. Similar to justification in Action C, while this is not the preferred 38 method, if a manual initiation function is inoperable, the ECCS subsystem pumps can be started 39 manually and the valves can be opened manually. The 24-hour completion time is acceptable, 40 because the functions can be performed manually and it allows time for the operator to evaluate 41 and have necessary repairs completed. Unlike the failure of a pushbutton that may concern 42 electronic component repairs, mechanical components may be involved in repairs, testing, and 43 return to service of pumps and valves. This further justifies a 24-hour completion time as 44 appropriate.

45

46 Action E is needed and becomes necessary, if the required *Required Aactionsaction* and

47 associated Ceompletion Ttimescompletion time of Condition C or D, are not met. If they are not

48 met, then the associated low pressure ECCS injection/spray subsystem may be incapable of

performing the intended function, and the ECCS subsystem must be declared inoperable
 immediately.

4 3.4.4 Proposed TS 3.3.5.2 Actions for BWR/6NUREG-1434

TS 3.3.5.2 contains proposed actions to be followed when the LCO is not met for ain
 BWR/6NUREG-1434.

8

3

5

Section 3.3.2, "Proposed TS 3.3.5.2 Actions," of TSTF-542, Revision 2, presents, discusses,
and justifies the Actions of TS 3.3.5.2 and LCO 3.3.5.2. The NRC staff finds these actions are
sufficient and necessary, because when one or more instrument channels are inoperable the
equipment and function controlled by these instruments cannot complete the required function
in the normal way and these actions direct the licensee to take appropriate actions as required.
The actions provide reasonable assurance that an unexpected draining event can be prevented
or mitigated before the RPV water level would be lowered to the TAF.

- Action A is applicable when one or more instrument channels are inoperable from Table 3.3.5.2-*1* and directs the licensee to immediately enter the condition referenced in Table 3.3.5.2-1 for
 that channel.
- 20

Action B (concerning the RHR system isolation and RWCU system isolation functions) areis applicable when automatic isolation of the associated penetration flow path is credited as not having to be considered as a path for potential drainage in calculating drain time. If the instrumentation is inoperable, Required Action B.1 directs an immediate declaration that the associated penetration flow path(s) are incapable of automatic isolation. Required Action B.2 requires a re-calculation of drain time, but automatic isolation of the affected penetration flow paths cannot be credited.

28

Action C (concerning low reactor steam dome pressure permissive Functions necessary for

ECCS subsystem manual initiation) addresses an event in which the permissive is inoperable
 and manual initiation of ECCS using the control board pushbuttons is prevented. The function
 must be placed in the trip condition within one hour. With the permissive function instrument in

33 the trip condition, manual initiation may now be performed using the preferred control board

- 34 pushbuttons. This one hour completion time is acceptable, because despite the preferred start
- 35 method being prevented, the reactor operator can take manual control of the pump and the
- 36 injection valve to inject water into the RPV and achieve the safety function. The time of one
- 37 hour also provides reasonable time for evaluation and placing the channel in trip.
- 38

39 Action D (concerning loss of adequate water supply for the HPCS System), addresses an event in which there is an inadequate water supply. The instrumentation functions have the ability to 40 41 detect low-water setpoint in the Condensate Storage Tank and actuate valves to realign HPCS 42 suction water source to the Suppression Pool. The Condensate Storage Tank Level - Low 43 Function indicates multiple, inoperable channels within the same Function resulting in a loss of 44 the automatic ability to swap suction to the Suppression Pool. The HPCS system must be 45 declared inoperable within one hour or the HPCS pump suction must be realigned to the Suppression Pool, since, if realigned, the Function is already performed. This one hour is 46

47 acceptable, because it provides sufficient time to take the action in order to minimize the risk of

1 HPCS being needed without an adequate water source by allowing time for restoration or 2 alignment of the HPCS pump suction to the suppression pool.

3

4 Action E (concerning HPCS high water level Function in the RPV) addresses actions when this 5 instrument function is inoperable. HPCS Reactor Vessel Water Level - High, Level 8 function 6 ensures that appropriate actions are taken if the HPCS Reactor Vessel Water Level - High, 7 Level 8 Function is inoperable. If the inoperability results in the channel being tripped, the 8 HPCS pump discharge valve will not open and HPCS injection is prevented. In that case the 9 HPCS System must be declared inoperable within one hour, and the function must be restored 10 to operable status within 24 hours. The one hour completion time is acceptable, because of the 11 ability to manually start the HPCS pumps and open the discharge valve. The 24-hour 12 completion time is acceptable, because it allows time for the operator to evaluate and arrange 13 for repairs. 14 15 Action F (concerning pump discharge flow bypass Functions and the manual initiation

- 16 Functions) addresses an event in which the bypass is inoperable and there is a risk that the 17 associated ECCS pump could overheat when the pump is operating and the associated
- 18 injection valve is not fully open. In this condition, the operator can take manual control of the 19 pump and the injection. Similar to justification in Action C, while this is not the preferred 20 method, if a manual initiation function is inoperable, the ECCS subsystem pumps can be started 21 manually and the valves can be opened manually. The 24-hour completion time is acceptable, 22 because the functions can be performed manually and it allows time for the operator to evaluate and have necessary repairs completed. Unlike the failure of a pushbutton that may concern 23 24 electronic component repairs, mechanical components may be involved in repairs, testing, and
- 25 return to service of pumps and valves further justifying a 24-hour completion time as 26 appropriate.
- 27

28 Action G is needed and becomes necessary, if the required Required action Actions and 29 associated Ceompletion Ttimescompletion time of Condition C, D, E, or F are not met. If they 30 are not met, then the associated low pressure ECCS injection/spray subsystem may be

incapable of performing the intended function, and the ECCS subsystem must be declared 31 32 inoperable immediately. 33

34 3.4.5 Proposed TS 3.3.5.2 Surveillances for BWR/4NUREG-1433 and BWR/6NUREG-35 1434

36

37 Section 3.3.3, "Proposed TS 3.3.5.2 Surveillances," of TSTF-542, presents, discusses, and 38 justifies the SR of TS 3.3.5.2. The TS 3.3.5.2 SR include Channel Checks, Channel Functional 39 Tests, and Logic System Functional Tests. There are three SRs numbered SR 3.3.5.2.1, SR 40 3.3.5.2.2, and SR 3.3.5.2.3. The NRC staff finds these tests are sufficient and adequate, 41 because they are essential to ensure the Functions of TS 3.3.5.2 are operable (i.e., capable of 42 performing the specified safety function in support of TS 3.5.2, Drain Time, and the protection from a potential drain down of the RPV in Modes 4 and 5). The NRC staff finds the proposed 43 44 TS 3.3.5.2 surveillances of LCO 3.5.2 as described in Section 3.3.3 satisfies 10 CFR 50.36(c)(3) 45 by providing the specific SRs relating to test, calibration, or inspection to assure that the 46 necessary quality of systems and components is maintained. 47

1 The following summarizes the notable characteristics of the surveillances described in 2 Section 3.3.3 of TSTF-542, which were reviewed by the NRC staff.

3

4 SR 3.3.5.2.1 requires a Channel Check and is applied to all functions except manual initiation. 5 Performance of the Channel Check ensures that a gross failure of instrumentation has not 6 occurred. A Channel Check is normally a comparison of the parameter indicated on one 7 channel to a similar parameter on other related channels. A Channel Check is significant in 8 assuring that there is a low probability of an undetected complete channel failure and is a key 9 safety practice to verifying the instrumentation continues to operate properly between each 10 Channel Functional Test. The frequency of 12 hours, or in accordance with the Surveillance 11 Frequency Control Program, is consistent with the existing requirements and supports operating 12 shift situational awareness. 13 14 SR 3.3.5.2.2 requires a Channel Functional Test and is applied to all functions except manual 15 initiation. A Channel Functional Test is the injection of a simulated or actual signal into the

16 channel as close to the sensor as practicable to verify operability of all devices in the channel 17 required for channel operability. It is performed on each required channel to ensure that the 18 entire channel will perform the intended function. The frequency is in accordance with the 19 Surveillance Frequency Control Program or 92 days. The applicant states, "This is acceptable 20 because it is consistent with the existing requirements for these Functions and is based upon 21 operating experience that demonstrates channel failure is rare." Since periods in MODEs 4 and 22 5 as refueling outages are often in the order of 30 days or less, licensees could include this SR. 23 if desired, as part of a refueling activity. 24

SR 3.3.5.2.3 requires a Logic System Functional Test and is only applied to the manual initiation
functions. The Logic System Functional Test is a test of all logic components required for
operability of a logic circuit, from as close to the sensor as practicable up to, but not including,
the actuated device, and demonstrates the operability of the required manual initiation logic for
a specific channel. The ECCS subsystem functional testing performed in proposed SR 3.5.2.7
overlaps this surveillance to complete testing of the assumed safety function. The traveler
states:

- 32 33
- 34The Frequency of [18] months, or in accordance with the35Surveillance Frequency Control Program, is consistent with the36existing requirements, and is based upon operating experience37that that has shown that these components usually pass the38Surveillance when performed at this Frequency.
- There are no SRs included to verify or adjust the instrument setpoint derived from the allowable
 value using a Channel Calibration or a surveillance to calibrate the trip unit. The traveler states,
- 43 44
- 45 A draining event in Mode 4 or 5 is not an analyzed accident and, 46 therefore, there is no accident analysis on which to base the
- 47 calculation of a setpoint. The purpose of the Functions is to allow
- 48 ECCS manual initiation or to automatically isolate a penetration

1 2 3 4 5 6 7	flow path, but no specific RPV water level is assumed for those actions. Therefore, the Mode 3 Allowable Value was chosen for use in Modes 4 and 5 as it will perform the desired function. Calibrating the Functions in Modes 4 and 5 is not necessary, as TS 3.3.5.1 and TS 3.3.6.1 continue to require the Functions to be calibrated on an [18] month Frequency.
8 9 10 11 12 13 14	And: A draining event in Mode 4 or 5 is not an analyzed accident and, therefore, there are no accident analysis assumptions on response time.
14 15 16 17 18	This is acceptable, because this is adequate to ensure the channel responds with the required pumping systems to inject water when needed and isolation equipment to perform when commanded.
19 20 21 22 23 24 25 26 27 28 29	ECCS Response Time and Isolation System Response Time testing ensure that the individual channel response times are less than or equal to the maximum values assumed in the accident analysis. TS 3.3.5.2 does not include SRs to participate in any ECCS Response Time testing and Isolation System Response Time testing. This is acceptable because the purpose of these tests are to ensure that the individual channel response times are less than or equal to the maximum values assumed in the accident analysis, but a draining event in Mode 4 or 5 is not an analyzed accident and, therefore, there are no accident analysis assumptions on response time and there are alternate manual methods for achieving the safety function. A potential draining event in MODEs 4 and 5 is a slower event than a LOCA. More significant protective actions are required as the calculated drain time decreases.
30	3.4.6 Conclusion of NRC Staff Review of TS 3.3.5.2
31 32 33 34 35 36 37	The NRC staff finds that proposed TS 3.3.5.2 and LCO 3.3.5.2 satisfies Criterion 4 of 10 CFR 50.36(c)(3), because specific instrumentation is provided that helps prevent or mitigate a potential RPV drain down event. Operating experience highlights that RPV draining events are potentially significant to public health and safety, as established in the following NRC documents:
38 39 40 41 42 43	 Information Notice 84-81 "Inadvertent Reduction in Primary Coolant Inventory in Boiling Water Reactors During Shutdown and Startup," November 1984. Information Notice 86-74, "Reduction of Reactor Coolant Inventory Because of Misalignment of RHR Valves," August 1986. Generic Letter 92-04, "Resolution of the Issues Related to Reactor Vessel Water Level Instrumentation in BWRs Pursuant to 10 CFR 50.54(f)," August 1992.
44 45 46	 NRC Bulletin 93-03, "Resolution of Issues Related to Reactor Vessel Water Level draining event in Mode 4 Instrumentation in BWRs," May 1993.
47 48	The NRC staff finds that proposed LCO 3.3.5.2 correctly specifies the lowest functional capability or performance levels of equipment required for safe operation of the facility. There is

reasonable assurance that the required actions to be taken when the LCO is not met can be
 conducted without endangering the health and safety of the public.

3 4 5

3.5

TABLE 3.3.5.2-1, "RPV WATER INVENTORY CONTROL INSTRUMENTATION"

In order to support the requirements of TS 3.5.2, and LCO 3.5.2, "Reactor Pressure Vessel
(RPV) Water Inventory Control," and the definition of "DRAIN TIME"; the instrumentation
requirements are designated in Table 3.3.5.2-1. These instruments are required to be operable
if the systems that provide water injection and isolation functions are to be considered operable
as described in the NRC staff's safety evaluation of TS 3.5.2.

11

Table 3.3.5.2-1 specifies the instrumentation that shall be operable for each function in the table for Modes 4 and 5 (or other specified conditions), the required number of channels per function, conditions referenced from required Required action Action A.1, SR for the functions, the allowable value (if version A), and footnotes concerning items of the table.

16

Table 3.3.5.2-1 for in BWR/4NUREG-1433 and BWR/6NUREG-1434 differ only in that version A
has a column for the allowable value and B does not. Version A has a potential or generic
allowable value in brackets. The brackets indicate that a plant-specific value should be used in
the LAR to adopt TSTF-542.

21

22 Section 3.3.4, "Proposed Table TS 3.3.5.2-1, 'RPV Water Inventory Control Instrumentation" of 23 TSTF-542, presents details on the functions required to support the equipment and functions of 24 TS 3.5.2 for in BWR/4NUREG-1433 and BWR/6NUREG-1434. The NRC staff finds the 25 presentation in this table acceptable, because this section sufficiently discusses the purpose of 26 the functions, the applicability, the number of required channels, the references to the Condition 27 to be entered by letter (e.g., A, B, C) if the function is inoperable, the applicable SRs, the 28 selection of the allowable value, if applicable, and justification of differences between the 29 existing and proposed TS functions. This RPV Water Inventory Control Instrumentation set is 30 acceptable, because it is adequate to ensure the instruments of the channels responds with the 31 required accuracy permitting pumps systems to operate to inject water when needed and 32 isolation of equipment when commanded to support the prevention of or mitigate a potential 33 RPV draining event.

34

35 Each of the ECCS subsystems in the BWR/4NUREG-1433 and BWR/6NUREG-1434 in MODEs 4 and 5 are initiated by manual pushbutton. The traveler states, "... automatic initiation of an 36 37 ECCS injection/spray subsystem, with injection rates of thousands of gpm, may be undesirable 38 as it can lead to overflowing the RPV cavity." Thus, there is adequate time to take manual 39 actions (e.g., hours versus minutes). Considering the action statements as the drain time decreases (the proposed TS 3.5.2, Action E, prohibits plant conditions that could result in drain 40 41 times less than one hour), therefore, there is sufficient time for the reactor operators to take 42 manual action to stop the draining event, and to manually start an ECCS injection/spray subsystem or the additional method of water injection as needed. Consequently, there is no 43 44 need for automatic initiation of ECCS to respond to an unexpected draining event. This is 45 acceptable, because a draining event is a slow evolution when compared to a design basis 46 LOCA assumed to occur at a significant power level.

47

48 3.5.1 Proposed Table 3.3.5.2-1 Functions for BWR/4NUREG-1433

1

2 The following summarizes notable characteristics of the RPV Water Inventory Control 3 Instrumentation as discussed in Section 3.3.4 of TSTF-542, Revision 2.

4

5 For the NUREG-1433 Table 3.3.5.2-1 Functions 1.a and 2.a, BWR/4-CS and LPCI Systems, 6 Reactor Steam Dome Pressure - Low (Injection Permissive), these signals are used as 7 permissives and protection for these low pressure ECCS injection/spray subsystem manual initiation functions. This function ensures that the reactor pressure has fallen to a value below 8 9 these subsystems' maximum design pressure before permitting the operator to open the 10 injection valves of the low pressure ECCS subsystems. Even though during MODEs 4 and 5 11 the reactor steam dome pressure is expected to virtually always be below the ECCS maximum 12 design pumping pressure, the Reactor Steam Dome Pressure - Low signals are required to be 13 operable and capable of permitting initiation of the ECCS.

14

15 For the NUREG-1433 Table 3.3.5.2-1 Functions 1.b and 2.b, BWR/4-CS and LPCI Systems, 16 Pump Discharge Flow - Low (Bypass), these minimum flow instruments are provided to protect 17 the associated low pressure ECCS pumps from overheating when the pump is operating and 18 the associated injection value is not fully open. The minimum flow line value is opened when 19 low flow is sensed, and the valve is automatically closed when the flow rate is adequate to 20 protect the pump. Where applicable, allowable values (version A) specified are high enough to 21 ensure that the pump flow rate is sufficient to protect the pump, yet low enough to ensure that 22 the closure of the minimum flow valve is initiated to allow full flow into the core. Brackets 23 around allowable value indicate the actual value is to be plant-specific and dependent on actual 24 equipment. The LPCI minimum flow valves are time delayed such that the valves will not open 25 for 10 seconds after the switches detect low flow. This time delay is acceptable, because it is 26 provided to limit reactor vessel inventory loss during the startup of the RHR shutdown cooling 27 mode.

28

29 For the NUREG-1433 Table 3.3.5.2-1 Functions 1.c and 2.c, BWR/4-CS System Manual 30 Initiation and LPCI, System Manual Initiation, the manual initiation pushbutton channels introduce signals into the appropriate ECCS logic to provide manual initiation capability. There 31 32 is one push button for each of the CS and LPCI subsystems (i.e., two for CS and two for LPCI). 33 There is no allowable value for this Function since the channels are mechanically actuated 34 based solely on the position of the push buttons. An instrument channel of the Manual Initiation 35 Function (one channel per subsystem) is required to be Operable in MODEs 4 and 5 when the 36 associated ECCS subsystems are required to be Operable per LCO 3.5.2.

37

38 For the NUREG-1433 Table 3.3.5.2-1 Function 3.a, BWR/4 RHR System Isolation, Reactor 39 Vessel Water Level - Low, Level 3, the function is only required to be operable when automatic 40 isolation of the associated penetration flow path is credited in the drain time calculation. The 41 number of required instrument channels is [2 in one trip system], which retains the requirement 42 that the two instrument channels must be associated with the same trip system. Each trip system isolates one of two redundant isolation valves, and only one trip system is required to be 43 44 operable to ensure that automatic isolation of one of the two isolation valves will occur on low 45 reactor vessel water level indication. The allowable value (version A) was chosen to be the 46 same as the Primary Containment Isolation Instrumentation Reactor Vessel Water Level - Low, 47 Level 3 Allowable Value from LCO 3.3.6.1.

48

1 For the NUREG-1433 Table 3.3.5.2-1 Function 4.a, BWR/4 RWCU, System Isolation, Reactor 2 Vessel Water Level - Low Low, Level 2, the function is only required to be operable when 3 automatic isolation of the associated penetration flow path is credited in the drain time calculation. The number of required channels is [2 in one trip system], which retains the 4 5 requirement that the two instrument channels must be associated with the same trip system. 6 Only one trip system is required to be operable to ensure that automatic isolation of one of the 7 two isolation valves will occur on low reactor vessel water level. Allowable value (version A) 8 was chosen to be the same as the ECCS Reactor Vessel Water Level - Low Low, Level 2 9 Allowable Value from LCO 3.3.5.1.

- 10
- 11 12

3.5.2 Proposed Table 3.3.5.2.-1 Functions for BWR/6NUREG-1434

The following summarizes notable characteristics of the RPV Water Inventory Control
 Instrumentation as discussed in Section 3.3.4 of TSTF-542, Revision 2.

15

For the *NUREG-1434* Table 3.3.5.2-1 Functions 1.a and 2.a, BWR/6-LPCS and LPCI Systems,
 Reactor Steam Dome Pressure - Low (Injection Permissive), these signals are used as

permissives and protection for these low pressure ECCS injection/spray subsystem manual initiation functions. This function ensures that the reactor pressure has fallen to a value below these subsystems' maximum design pressure before permitting the operator from opening the injection valves of the low pressure ECCS subsystems. Even though during MODEs 4 and 5 the reactor steam dome pressure is expected to virtually always be below the ECCS maximum design pumping pressure, the Reactor Steam Dome Pressure - Low signals are required to be operable and capable of permitting initiation of the ECCS.

25

26 For the NUREG-1434 Table 3.3.5.2-1 Functions 1.b, 1.c, and 2.b, BWR/6 LPCS and LPCI 27 Systems Low Pressure Coolant Injection and Low Pressure Core Spray Pump Discharge Flow -28 Low (Bypass), these instruments are provided to protect the associated low pressure ECCS 29 pump from overheating when the pump is operating and the associated injection valve is not 30 fully open. The minimum flow line valve is opened when low flow is sensed, and the valve is 31 automatically closed when the flow rate is adequate to protect the pump. Where applicable 32 allowable values (version A) specified are high enough to ensure that the pump flow rate is 33 sufficient to protect the pump, yet low enough to ensure that the closure of the minimum flow 34 valve is initiated to allow full flow into the core. Brackets around allowable value indicate the 35 actual value is to be plant-specific and dependent on actual equipment.

36

For the *NUREG-1434* Table 3.3.5.2-1 Functions 1.d and 2.c, BWR/6 LPCS and LPCI Systems,
Manual Initiation, the manual initiation pushbutton channels introduce signals into the
appropriate ECCS logic to provide manual initiation capability. There is one pushbutton for
each subsystem in the two divisions of low pressure ECCS (i.e., Division 1 ECCS, LPCS and
LPCI A: Division 2 ECCS, LPCI B and LPCI C). There are four subsystems, thus four

LPCI A; Division 2 ECCS, LPCI B and LPCI C). There are four subsystems, thus four
 pushbuttons for the low pressure ECCS. The only manual initiation function required to be

43 operable is that associated with the ECCS subsystem that is required to be operable by LCO

44 3.5.2. Since the channels are mechanically actuated based solely on the position of the

45 pushbuttons, there is no allowable value (version A) for this function. When this instrument

46 function is inoperable, manual initiation with the control board push buttons is inoperable.

47 However, the ECCS pumps can be started manually and valves can be opened manually by the

48 reactor operator. This is not the preferred condition.

1

2 For the NUREG-1434 Table 3.3.5.2-1 Functions 3.a, BWR/6 HPCS System Reactor Vessel 3 Water Level - High, Level 8, the High RPV water level, Level 8 signal, is used to close the HPCS injection value to prevent overflow into the main steam lines (MSLs). One instrument 4 5 channel associated with the HPCS system is required to be operable to support LCO 3.5.2. The 6 LCO 3.3.5.2 allowable value (version A) is chosen to isolate flow from the HPCS system prior to 7 water overflowing into the MSLs.

8

9 For the NUREG-1434 Table 3.3.5.2-1 Functions 3.b, BWR/6 HPCS System, Condensate 10 Storage Tank Level – Low, the low level signal in the Condensate Storage Tank (CST) indicates 11 the lack of an adequate supply of makeup water from this primary source for HPCS. Normally, 12 the water source for the suction for HPCS is the CST. If the water level in the CST falls below a 13 preselected level, instrumentation logic controls valves so suction is then pulled from the 14 Suppression Pool. First the Suppression Pool suction valve is automatically opened and then 15 the CST suction valve is automatically closed in a manner to ensure that an adequate supply of 16 makeup water is available to the HPCS pump. The Condensate Storage Tank Level - Low signals are initiated from two level transmitters. The Condensate Storage Tank Level - Low 17 18 Function Allowable Value is high enough to ensure adequate pump suction head while water is being taken from the CST.

19 20

21 For the NUREG-1434 Table 3.3.5.2-1 Functions 3.c and 3.d, BWR/6 HPCS System, HPCS 22 Pump Discharge Pressure - High (Bypass) and HPCS System Flow Rate - Low (Bypass), the 23 minimum flow instruments are provided to protect the HPCS pump from overheating when the 24 pump is operating and the associated injection valve is not fully open. The minimum flow line 25 valve is opened when low flow and high pump discharge pressure are sensed, and the valve is automatically closed when the flow rate is adequate to protect the pump or the discharge 26 27 pressure is low (indicating the HPCS pump is not operating).

28

29 For the NUREG-1434 Table 3.3.5.2-1 Function 3.e, BWR/6 HPCS System, Manual Initiation, 30 the Manual Initiation push button channel introduces a signal into the HPCS logic to provide

31 manual initiation capability. There is one pushbutton for the HPCS system.

32

33 For the NUREG-1434 Table 3.3.5.2-1 Function 4.a, BWR/6 RHR System Isolation, Reactor 34 Vessel Water Level - Low, Level 3, the Function is only required to be operable when automatic 35 isolation of the associated RHR system penetration flow path is credited in calculating drain 36 time. The definition of drain time allows crediting the closing of penetration flow paths that are capable of being automatically isolated by RPV water level isolation instrumentation prior to the 37 38 RPV water level dropping below the TAF, but if the instrument function is inoperable, a closed 39 path cannot be credited and a drain time calculation must be re-performed.

40

41 For the NUREG-1434 Table 3.3.5.2-1 Function 5.a, BWR/6 RWCU System Isolation, Reactor

Vessel Water Level - Low Low, Level 2, the Function is only required to be Operable when 42

43 automatic isolation of the associated RWCU system penetration flow path is credited in

44 calculating drain time. The definition of drain time allows crediting the closing of penetration 45

flow paths that are capable of being automatically isolated by RPV water level isolation

46 instrumentation prior to the RPV water level dropping below the TAF, but if the instrument 47

function is inoperable, a closed path cannot be credited and a drain time calculation must be re-

48 performed. This function is not applicable in MODEs 4 or 5 in TS 3.3.6.1, but is being added to 1 TS 3.3.5.2 to support crediting the automatic isolation of the RWCU system in calculating drain 2 time.

3 4

3.6 OTHER DIFFERENCES BETWEEN THE CURRENT AND PROPOSED TS

Section 3.4., "Evaluation of other Differences between the Current and Proposed TS," of TSTF542, presents and discusses other differences between the current TS requirements related to
OPDRVs and the proposed TS requirements for RPV WIC. The current STS contain
requirements related to instrumentation that are applicable during OPDRVs and are applicable
when the existing LCO 3.5.2 is applicable. They do not specifically impact the focus on TS
3.3.5.2 and the associated LCO 3.5.2 and Table 3.3.5.2-1.

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3.7 STS 3.5.2 – REACTOR PRESSURE VESSEL (RPV) WATER INVENTORY CONTROL

The categories of items required to be in the TSs are provided in 10 CFR 50.36(c). As required by 10 CFR 50.36(c)(2)(i), the TSs will include LCOs, which are the lowest functional capability or performance levels of equipment required for safe operation of the facility. Per 10 CFR 50.36(c)(2)(i), when an LCO of a nuclear reactor is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the TSs until the condition can be met.

The regulations at 10 CFR 50.36(c)(2)(ii) state that LCOs must be established for each item
 meeting one of four criteria:

Criterion 1. Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary.

Criterion 2. A process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to fission product barrier integrity.

Criterion 3. A structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.

Criterion 4. A structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety.

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45 Technical Specification Safety Limit 2.1.1.3 requires that reactor vessel water level shall be

46 greater than the top of active irradiated fuel. Maintaining water level above the TAF ensures

47 that the fuel cladding fission product barrier is protected during shutdown conditions. The

48 changes to the STS described in traveler TSTF-542 establish specifications for equipment and

associated instrumentation that ensure the reactor vessel water level is maintained above the
 TAF during MODE 4 and 5 operations.

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4 NUREG-0800, Revision 3, Standard Review Plan (March 2010) (ADAMS Accession 5 No. ML100351425), describes LOCAs as postulated accidents that would result from the loss of 6 reactor coolant, at a rate in excess of the capability of the normal reactor coolant makeup 7 system, from piping breaks in the reactor coolant pressure boundary. During operation in 8 MODEs 4 and 5, the reactor coolant system is at a low operating temperature (</200] ° 9 Fahrenheit) and is depressurized. An event involving a loss of inventory while in the shutdown 10 condition is judged to not exceed the capacity of one ECCS subsystem. The accidents that are 11 postulated to occur during shutdown conditions, the Fuel Handling Accident and the Waste Gas 12 Decay Tank Rupture, do not involve a loss of inventory. The equipment and instrumentation 13 associated with the Reactor Vessel Water Inventory Control TS do not provide detection or 14 mitigation related to these design basis accidents. 15 16 The revised STS LCO 3.5.2 contains requirements for operability of one ECCS subsystem 17 along with requirements to maintain a sufficiently long drain time that plant operators would 18 have time to diagnose and mitigate an unplanned draining event. The NRC staff has 19 determined that the LCO 3.5.2 and 3.3.5.2 provide alternatives for the lowest functional 20 capability or performance levels of equipment required for safe operation of the facility. On this 21 basis, the NRC staff concludes that the requirements of 10 CFR 50.36(c)(2)(i) are met. 22 23 Additionally, the revised STS LCOs 3.5.2 and 3.3.5.2 provide remedial actions to be taken in the 24 event the LCO is not satisfied, therefore meeting the requirements of 10 CFR 50.36(c)(2)(i). 25 The NRC staff has found that the remedial actions provide reasonable assurance that an 26 unexpected draining event can be prevented or mitigated before the RPV water level would be 27 lowered to the TAF. 28 29 The regulation at 10 CFR 50.36(c)(3) requires TSs to include items in the category of SRs. 30 which are requirements relating to test, calibration, or inspection to assure that the necessary 31 quality of systems and components is maintained, that facility operation will be within safety limits, and that the LCOs will be met. The NRC staff reviewed the SRs associated with the 32 33 revised LCOs 3.5.2 and 3.3.5.2. The NRC staff reviewed the new SRs and determined that 34 they are appropriate for ensuring the operability of the equipment and instrumentation specified 35 in LCOs 3.5.2. Therefore, the NRC staff concludes that 10 CFR 50.36(c)(3) is met. 36 37 The regulation at 10 CFR 50.36(a)(1) states that a summary statement of the bases or reasons 38 for such specifications, other than those covering administrative controls, shall also be included 39 in the application, but shall not become part of the TSs. Traveler TSTF-542 contains TS Bases 40 changes that describe the basis for the affected TS. A summary of the NRC staff's evaluation of 41 the TS Bases changes is provided in an attachment to this SE. 42 43 The NRC staff's guidance for review of TSs is in Chapter 16. *Technical Specifications*, of 44 NUREG-0800, Revision 3, Standard Review Plan (March 2010) (ADAMS Accession

45 No. ML100351425). As described therein, as part of the regulatory standardization effort, the

46 NRC staff has prepared STS for each of the light–water reactor nuclear designs. NUREG-1433,

47 Revision 4, contains the STS for BWR/4 plants, and is applicable to BWR/2, BWR/3, and, in

48 some cases, BWR/5 plants, and NUREG 1434, Revision 4, contains the STS for BWR/6 plants

and is applicable, in some cases, to BWR/5 plants.. The changes to the STS were reviewed for
 technical clarity and consistency with customary terminology and format with the existing
 requirements. The NRC staff found that the proposed changes were consistent with the existing
 framework.

6 4.0 CONCLUSION

7 8 The NRC staff reviewed traveler TSTF-542, Revision 2, "Reactor Pressure Vessel Water 9 Inventory Control," which proposed changes to NUREG-1433, Volumes 1 (STS for BWR/4) and 10 2 (Bases) and NUREG-1434 Volumes 1 (STS for BWR/6) and 2 (Bases). The NRC staff 11 determined that the proposed changes to the STS for BWR/4 and the STS for BWR/6 met the 12 standards for TS in 10 CFR 50.36(b). The proposed LCOs appropriately specify the lowest 13 functional capability or performance levels of equipment required for safe operation of the 14 facility, as required by 10 CFR 50.36(c)(2)(i). The remedial actions to be taken when an LCO is 15 not met action statements provide adequate protection to the health and safety of the public, 16 thereby satisfy the Act and 10 CFR 50.36(c)(2)(i). The proposed surveillance requirements 17 assure that the necessary quality of systems and components is maintained, that facility 18 operation will be within safety limits, and that the LCOs will be met, and satisfy 10 CFR 19 50.36(c)(3). 20 21 The proposed bases, which will be added to future revisions to NUREG-1433, Volume 2, and 22 NUREG-1434, Volume 2, satisfy the Commission's Policy Statement by addressing the 23 questions specified in the policy statement, and cite references to appropriate licensing 24 documentation to support the Bases. 25 26 Technical contacts: Matt Hardgrove, NRR/DSS/SRXB 27 Eugene Eagle, NRR/DE/EICB 28 29 Attachment: Basis for Accepting the Proposed Changes to the Standard Technical 30 Specification Bases, Volume 2 of NUREGs 1433 and 1434 31

32 Date:

	ATTACHMENT
BAS	SIS FOR ACCEPTING THE PROPOSED CHANGES TO THE STANDARD TECHNICAL SPECIFICATION BASES, VOLUME 2 OF NUREGS 1433 AND 1434
1.0	INTRODUCTION
BWR/ Acces Plants No. M Volum	ler TSTF-542 proposes changes to "Standard Technical Specifications, General Electric 4 Plants, BWR/4" NUREG-1433, Volume 2, "Bases," Revision 4.0, April 2012, ADAMS asion No. ML12104A193 and "Standard Technical Specifications, General Electric BWR/6 6, BWR/6" NUREG-1434, Volume 2, "Bases," Revision 4.0, April 2012, ADAMS Accession IL12104A196. The changes would be incorporated into future revisions of NUREG-1433, ne 2, and NUREG-1434, Volume 2. A summary of the changes and the staff's evaluation se changes are presented in this Attachment.
2.0	REGULATORY EVALUATION
2.1	Applicable Regulations and Guidance
opera specif bases also b In its I React Speci impler nuclea	egulation at 10 CFR 50.36(a)(1) states that each applicant for a license authorizing tion of a production or utilization facility shall include in his application proposed technical fications in accordance with the requirements of this section. A summary statement of the or reasons for such specifications, other than those covering administrative controls, shall be included in the application, but shall not become part of the technical specifications. Final Policy Statement on Technical Specifications Improvements for Nuclear Power ors, the Commission presented its policy on the scope and purpose of the Technical fications. The Commission explained how implementation of the policy statement through mentation of the improved STS is expected to produce an improvement in the safety of ar power plants through the use of more operator-oriented TS, improved TS Bases, ed action-statement-induced plant transients, and more efficient use of NRC and industry rces.
	inal Policy Statement provides the following description of the scope and the purpose of echnical Specification Bases: Appropriate Surveillance Requirements and Actions should be retained for each LCO which remains or is included in the Technical Specifications. Each LCO, Action, and Surveillance Requirement should have supporting Bases. The Bases should at a minimum address the following questions and cite references to appropriate licensing documentation (e.g., FSAR, Topical Report) to support the Bases.
	 1.0 Trave BWR/Access Plants No. M Volum of tho 2.0 2.1 The respective bases also be in its I React Specific implementation of the specific intervence of the specific

1 2 3 4	1.	What is the justification for the Technical Specification, i.e., which Policy Statement criterion requires it to be in the Technical Specifications?
5 6 7 8 9 10	2.	What are the Bases for each LCO, i.e., why was it determined to be the lowest functional capability or performance level for the system or component in question necessary for safe operation of the facility and, what are the reasons for the Applicability of the LCO?
10 11 12 13 14 15 16 17	3.	What are the Bases for each Action, i.e., why should this remedial action be taken if the associated LCO cannot be met; how does this Action relate to other Actions associated with the LCO; and what justifies continued operation of the system or component at the reduced state from the state specified in the LCO for the allowed time period?
18 19	4.	What are the Bases for each Safety Limit?
20 21 22 23 24 25 26 27 28	5.	What are the Bases for each Surveillance Requirement and Surveillance Frequency; i.e., what specific functional requirement is the surveillance designed to verify? Why is this surveillance necessary at the specified frequency to assure that the system or component function is maintained, that facility operation will be within the Safety Limits, and that the LCO will be met? Note: In answering these questions the Bases for each number (e.g., Allowable Value, Response Time, Completion Time,
29 30 31 32		Surveillance Frequency), state, condition, and definition (e.g., operability) should be clearly specified. As an example, a number might be based on engineering judgment, past experience, or PSA insights; but this should be clearly stated.
33 34 35 36		ff used the guidance contained in the Final Policy Statement during its review of changes to the Bases.
37	2.2 <u>Descr</u>	iption of Changes
38 39 40 41		REGs-1433 and -1434 contain the Bases for each Safety Limit and each LCO Volume 1. The Bases for each LCO is organized into sections:
42		Background

1	Applicable Safety Analyses, LCO, and Applicability
2	Actions
3	Surveillance Requirements
4	References
5	
6	The Bases for LCOs 3.3.5.2 (A and B) and 3.5.2 were rewritten in their entirety to reflect the
7	changes in the associated LCOs. The Bases for the remainder of the affected LCOs were
8	modified to reflect the deletion of OPDRVs.
9	
10	In the following sections, the discussion is applicable to both NUREG 1433 (for BWR/4 plants)
11	and NUREG 1434 (for BWR/6 plants) unless otherwise noted. The discussion provides a
12	summary of the revised Bases, followed by the NRC staff's evaluation of the revised Bases.
13	
14	3.0 TECHNICAL EVALUATION
15	
16	3.1 Evaluation of B 3.3.5.2 (A) and B 3.3.5.2 (B)
17	P = 2 = 2 = 2(A) is applicable in the charges of a Saturaint Control Program, and $P = 2 = 2 = 2(P)$ is
18 10	B 3.3.5.2(A) is applicable in the absence of a Setpoint Control Program, and B 3.3.5.2(B) is
19 20	applicable if a Setpoint Control Program is used. For simplicity in presentation, the description
20 21	provided below applies to both the (A) and (B) versions, unless otherwise stated.
22	The Background section provides
23	
24	• a description of the reactor pressure vessel (RPV) design, which includes penetrations
25	below the top of active fuel
26	 a description of Safety Limit 2.1.1.3, which requires the RPV water level to be above the
27	top of active fuel
28	 an explanation of the purpose of the RPV water Inventory Control Instrumentation, which
29	is to support the requirements of LCO 3.5.2, Reactor Pressure Vessel Water Inventory
30	Control by ensuring that the functions required for manual initiation of required
31	Emergency Core Cooling System (ECCS) injection/spray subsystem are available and
32	that other functions supporting isolation of flowpath(s) on low RPV level are available.
33	
34	The Applicable Safety Analyses, LCO and Applicability section provides:
35	
36	• a statement that indicates that water inventory control is required in Modes 4 and 5 to
37	protect Safety Limit 2.1.1.3 and a discussion that due to the reduced Reactor Coolant
38	System (RCS) pressure in the shutdown condition, a very large break in the RCS is not
39	postulated in the shutdown condition.
40	• an explanation that this LCO is applicable in Modes 4 and 5 to support operability of
41	subsystems required to be operable in Modes 4 and 5 as specified in LCO 3.5.2.

1	٠		ription of why the LCO meets Criterion 4 specified in 10 CFR 50.36(c)(2(ii) as a				
2			structure, system or component which operating experience has shown to be significant				
3		to pub	lic health and safety.				
4	٠	a deta	iled discussion regarding each function contained in LCO 3.3.5.2				
5							
6		0	Core Spray and Low Pressure Coolant Injection Systems:				
7		0	Function 1.a, 2.a Reactor Steam Dome Pressure – Low (Injection Permissive) is				
8			required to be Operable to ensure the capability of initiating ECCS when				
9			pressure is below the injection subsystems design pressure. The actuation logic				
10			is one out of two taken twice, four channels are required to be operable.				
11		0	For NUREG-1433BWR/4s:				
12			 Function 1.b, 2.b Core Spray and Low Pressure Coolant Injection Pump 				
13			Discharge Flow – Low (Bypass) is required to be operable to ensure				
14			minimum flow line is available to protect the associated low pressure				
15			ECCS pump from overheating on low discharge and to ensure closure of				
16			the minimum flow valve is initiated at the proper point to ensure full				
17			injectionpoint when the flow when required rate is adequate to protect the				
18			<i>pump.</i> One channel per required pump is required to be operable.				
19		0	For BWR/6 NUREG-1434's6's:				
20			 Function 1.b, 1.c, 2.b Low Pressure Coolant Injection and Low Pressure 				
21			Core Spray pumpPump Discharge Flow - Low (Bypass) is required to be				
22			operable to ensure minimum flow line is available to protect the				
23			associated low pressure ECCS pump from overheating on low discharge				
24			and to ensure closure of the minimum flow valve is initiated at the proper				
25			point to ensure full injection flow when required. when the flow rate is				
26			adequate to protect the pump. One channel per required pump is				
27			required to be operable.				
28		0	Function 1.c (for BWR/4<i>NUREG-1433</i>) , 1.d (for BWR/6<i>NUREG-1434</i>) 2.c,				
29			Manual Initiation, is required to be operable to provide manual initiation				
30			capability. One channel (pushbutton) per required subsystem is required to be				
31			operable per ECCS subsystem required to be operable.				
32		0	For BWR/6NUREG-1434, High Pressure Core Spray System				
33			 Function 3.a, Reactor Vessel Water Level – High, Level 8 is used to close 				
34			the HPCS injection valve to prevent overflow into the main steam lines.				
35			One channel associated with the HPCS system required by LCO 3.5.2 is				
36			required to be operable. The allowable value is chosen to ensure no				
37			overflow into the main steam lines.				
38			 Function 3.b, Condensate Storage Tank (CST) Level, Low indicates low 				
39			supply of makeup water from this source. HPCS is normally aligned to				
40			take suction on the CST. On low CST level, the HPCS pump suction				
41			valvesvalve from the suppression pool open and then the suction				

1			valvesvalve from the CST close. One channel is required to be operable
2			when HPCS is required per LCO 3.5.2 and the HPCS is aligned to the
3			CST. The allowable value is selected to ensure adequate pump suction
4			head.
5			 Function 3.c., 3.d HPCS Pump Discharge Pressure – High (Bypass) and
6			HPCS System Flow Rate – Low (Bypass) is required to be operable to
7			ensure minimum flow line is available to protect the associated HPCS
8			pump from overheating on low discharge. The allowable value is set to
9			ensure the flow is sufficient to protect the pump, but closes when a
10			minimum flow is reached to ensure full injection flow into the core.
11			adequate to protect the pump or the discharge pressure is low (indicating
12			<i>the HPCS pump is not operating</i>). One channel is required when HPCS
13			is required to be operable per LOC 3.5.2.
14			 Function 3.e, Manual Initiation, is required to be operable to ensure
15			manual initiation capability. One channel is required when the associated
16			ECCS subsystem is required by LCO 3.5.2.
17			
18		0	RHR System Isolation:
19		0	Function 3.a (for BWR/4NUREG-1433) and 4.a (for BWR/6NUREG-1434),
20		Ŭ	Reactor Vessel Water Level – Low Level 3 may be credited for automatic
21			isolation of penetration flow paths associated with the RHR system. The function
22			is required to be operable when automatic isolation of the associated penetration
23			flow path is assumed in the calculated Drain Time. Two channels in the same
24			trip system are required to be operable.
25			
26		0	Reactor Water Cleanup (RWCU) System Isolation
27		0	Function 4.a (for NUREG-1433BWR/4) and 5.a (for BWR/6NUREG-1434),
28		0	Reactor Vessel Water level – Low Low, Level 2 may be credited for automatic
29			isolation of penetration flow paths associated with the RWCU System. This
30			function is required to be operable when automatic isolation of the associated
31			penetration flow path is assumed in the calculated Drain Time. Two channels in
32			the same trip system are required to be operable.
33		0	The Allowable Value selected is the same as the Allowable Value during Power
34		0	Operation.
34 35			
36	• •		Nanation of each Required Action and Completion Time contained in the Actions
		n exp able	planation of each Required Action and Completion Time contained in the Actions
37	I		
38		0	For NUREG-1433BWR/4s:
39 40			 Condition A is entered when a channel is declared inoperable and Beguired Action A 1 directs entry into the Appropriate Condition
40 41			Required Action A.1 directs entry into the Appropriate Condition.
41 42			 Condition B is entered when Functions 13.a, 2 and 4.a, 1.b or 2.b are incorrected. The Required Action is to dealers the acception of the properties.
42			inoperable. The Required Action is to declare the associated penetration

4	flere weth in some ble strende verdig is sleting on slate we selected to the Durin
1	flow path incapable of automatic isolation and to recalculate the Drain
2	Time without taking credit for the automatic isolation of the affect
3	pathway.
4	 Condition C is entered when the steam dome pressure signal permissive
5	is inoperable. Inoperability of the permissive means that the injection
6	function cannot be manually initiated. The Required Action is to place the
7	permissive in the tripped condition within one hour. This enables manual
8	initiation of the injection function. The one hour allowance provides
9	sufficient time for the operator to place the channel in trip.
10	 Condition D is entered when the Core Spray or Low Pressure Coolant
11	Injection Pump Discharge Flow – Low bypass functions are unavailable.
12	In this condition, the Required Action is to restore the channel to operable
13	status within 24 hours. The 24 hour is judged to be appropriate because
14	manual operation of the pumps and the minimum flow valvesinjection
15	valves is still available, but this is not the preferred condition.
16	 Condition E is entered when the Required Action and associated
17	Completion Time for Condition C or D is not met. In this case, the
18	associated ECCS subsystem may not be capable of performing its
19	intended function, and is declared inoperable immediately.
20	
21	o For BWR/6NUREG-1434s6s :
22	 Condition A is entered when a channel is declared inoperable and
23	Required Action A.1 directs entry into the Appropriate Condition.
24	 Condition B is entered when the RHR System Isolation or RWCU System
25	Isolation functions are inoperable. The Required Action is to declare the
26	associated penetration flow path incapable of automatic isolation and to
27	recalculate the Drain Time without taking credit for the automatic isolation
28	of the affect pathway.
29	 Condition C is entered when the Steam Dome Low Pressure Signal
30	(Injection Permissive) is inoperable. Inoperability of the permissive
31	means that the injection function cannot be manually initiated. The
32	Required Action is to place the permissive in the tripped condition within
33	one hour. This enables manual initiation of the injection function. The
34	one hour allowance provides sufficient time for the operator to place the
35	channel in trip.
36	 Condition D is entered when the CST Level – Low function is inoperable.
37	The Required Action is to declare HPCS system inoperable and to align
38	the HPCS pump suction to the suppression pool within 1 hour.
39	 Condition E is entered when the Reactor Vessel Water Level – High –
40	Level 8 function is inoperable. The Required Action is to declare HPCS
41	system inoperable within 1 hour and to restore the channel to operable
42	status within 24 hours.

1	 Condition F is entered when the LPCS Pump Discharge Flow – Low (Durage) L DCL Durage A Discharge Flow – Low (Durage) L DCL Durage R
2	(Bypass), LPCI Pump A Discharge Flow – Low (Bypass), LPCI Pump B
3	and LPCI pump C Discharge Flow – Low (Bypass), HPCS Pump
4	Discharge Pressure – High (Bypass), HPCS System Flow rate – Low
5	(Bypass) or any of the required Manual functions are inoperable. The
6	Required Action is to restore the channel to operable status within 24
7	hours. The 24 hour is judged to be appropriate because manual
8	operation of the pumps and the minimum flow-injection valves is still
9	available, but this is not the preferred condition.
10	 Condition G is entered when the Required Action and associated Completing Times for Condition C. D. F. or F is not motion this encode the
11	Completion Time for Condition C, D, E or F is not met. In this case, the
12	associated ECCS subsystem may not be capable of performing its
13 14	intended function, and is declared inoperable immediately.
14 15	The Surveillence Requirements section provides:
15 16	The Surveillance Requirements section provides:
17	a description of the nurness of each Surveillence Dequirement and the basis for the
17	a description of the purpose of each Surveillance Requirement and the basis for the autricillance frequency calculated. For each function
10 19	surveillance frequency selected. For each function, o A channel check is performed to verify that a gross failure of an instrument
19 20	
20 21	channel has not occurred. Agreement criteria is established based on channel
21	instrument uncertainties and readability. The surveillance is performed once per 12 hours or in accordance with the Surveillance Frequency Control Program.
22 23	
23 24	The frequency was selected based on operating experience that indicates channel failure is rare.
24 25	
25 26	 A channel functional test is performed to verify the channel is capable of performing its intended function. The surveillance is performed once per 92 days
20 27	or in accordance with the Surveillance Frequency Control Program. The
28	frequency was selected based on operating experience that indicates channel
20 29	failure is rare.
29 30	 A logic system functional test is performed to verify proper functioning of the
31	required initiation logic for a channel. The surveillance is performed once per 18
32	months or in accordance with the Surveillance Frequency Control Program. The
33	frequency was selected because of the preference to perform the surveillance
34	under shutdown conditions.
35	
36	The References section provides lists Regulatory Guide 1.105, "Setpoints for Safety-Related
30 37	Instrumentation," and NEDE-770-06-2, "Addendum to Bases for Changes to Surveillance Test
38	Intervals and Allowed Out-of-Service Times for Selected instrumentation Instrumentation
39	Technical Specifications."
40	
41	The staff reviewed the revised bases to ensure the applicable criteria from 10 CFR 50.36 is
42	identified and justified. The revised bases state that the proposed LCO meets the Criterion 4

42 identified and justified. The revised bases state that the proposed LCO meets the Criterion 4

1	•	ed in 10 CFR 50.36(c)(2(ii) and provides a discussion of why this Criterion applies. The				
2	reasons for the selection of each instrument function and required number of channels in the					
3	LCO is described and the reason for the applicable modes is stated. Each instrument function					
4 5	is necessary to support operability of the equipment required by LCO 3.5.2, and the applicable					
6	modes are consistent with those in LCO 3.5.2. The purpose of each required action is described. The purpose of each instrument surveillance and the basis for the performance					
7		ncy is addressed, and appropriate references are cited. The staff concluded that each of				
8		ements of the Final Policy Statement were satisfactorily addressed. Therefore, the staff				
9	determ	nined that the revised Bases adheres to the guidance provided in the Final Policy				
10	Staten					
11						
12	3.2	EVALUATION OF B 3.5.2				
13						
14	The Ba	ackground section provides				
15						
16	•	a description of the reactor pressure vessel (RPV) design, which includes penetrations				
17		below the top of active fuel				
18	•	a description of Safety Limit 2.1.1.3, which requires the RPV water level to be above the				
19		top of active fuel				
20						
21 22	The A	oplicable Safety Analyses, LCO and Applicability section provides:				
22	•	a statement that indicates that water inventory control is required in Modes 4 and 5 to				
23 24	•	protect Safety Limit 2.1.1.3 and a discussion that due to the reduced RCS pressure in				
24 25		the shutdown condition, a very large break in the RCS is not postulated in the shutdown				
26		condition.				
27	•	an explanation that one low pressure ECCS injection/spray subsystem can maintain				
28	•	adequate RPV level (explanation retained from previous Bases for LCO 3.5.2)				
29	•	a description of why the LCO meets Criterion 4 specified in 10 CFR 50.36(c)(2(ii) as a				
30	-	structure, system or component which operating experience has shown to be significant				
31		to public health and safety.				
32	•	an explanation that a Drain Time of 36 hours was selected for the LCO because this				
33		time period is reasonable for the operator to identify and initiate remedial measures.				
34	•	an explanation that the LCO also requires one low pressure ECCS injection/spray				
35		subsystem to be operable and capable of being manually started so that it is available				
36		should an unexpected drain event occur. The ECCS injection/spray subsystem may be				
37		considered operable during alignment for decay heat removal because the restriction on				
38		drain time ensures sufficient time is available to initiate LPCI operation to maintain				
39		inventory if required.				
10	•	an explanation of each Required Action and Completion Time contained in the Actions				

40 • an explanation of each Required Action and Completion Time contained in the Actions
 41 Table

1 2 3 4 5 6 7 8 9	0	Condition A is entered if the required ECCS injection/spray subsystem is inoperable. The Required Action is to restore it to Operable status within 4 hours. The 4 hour Completion Time is judged to be appropriate because of the controls on Drain Time and the low probably of a drain event occurring. Condition B is entered if the Required Action and Completion Time of Condition A is not met. Condition B requires establishing an alternate method of water injection capable of injecting without the use of offsite power, with attendant necessary support equipment, and access to water inventory capable of maintaining the RPV water level above TAF for 36 hours. The Completion Time
10		is immediately.
11	0	Condition C is entered if the drain time is less than 36 hours but greater than or
12		equal to 8 hours. The Required Actions associated with this Condition ensure
13 14		the availability of compensatory actions should an unexpected drain event occur. The Required Actions include actions to ensure the secondary containment
15		boundary can be restored in less than the Drain Time to provide a volume to
16		contain, dilute and process radioactive materials if an unexpected drain event
17		were to occur. The Actions also include verification of the ability to place the
18		Standby Gas Treatment System in service within the Drain Time to provide a
19		means to maintain the secondary containment volume at a negative pressure
20		and to filter the contents prior to release. A Completion Time of 4 hours was
21		selected for these verifications because this ensures that the actions are
22		completed well within the minimum Drain Time of 8 hours.
23 24	0	Condition D is entered if the drain time is less than 8 hours. When the Drain Time is this short, mitigating actions as well as compensatory actions are
24 25		needed. The Required Actions include an immediate action to establish an
26		additional method of water injection. This method is in addition to the injection
27		method required by the LCO. The Required Actions include the compensatory
28		actions of immediately establishing the secondary containment boundary,
29		verifying secondary containment penetrations can be isolated, and verifying that
30		at least one Standby Gas Treatment subsystem can be placed into operation.
31		These Actions are performed immediately because of the short Drain Time.
32	0	Condition E is also applicable when Drain Time is less than 1 hour. The
33 34		Required Action is to immediately restore the Drain Time to greater than 36
34 35		hours. Restoration of the Drain Time to 36 hours is necessary to ensure there is adequate time to perform mitigating actions should an unexpected drain event
36		occur.
37		
38	The Surveillar	nce Requirements section provides:
39		
40		ription of the purpose of each Surveillance Requirement and the basis for the
41	survei	llance frequency selected

1	0	The Drain Time is required to be verified to be \geq 36 hours once per 12 hours or in
2		accordance with the Surveillance Frequency Control Program. This Surveillance
3		verifies the LCO for Drain Time is met. The frequency is selected based on the
4		fact that numerous indications of changes in RPV level are available to the
5		operator. Changes in RPV level would necessitate recalculation of the Drain
6		Time.
7	0	The suppression pool water level for a required LPCI subsystem, or suppression
8		pool water level or Condensate Storage Tank level for a required core spray
9		subsystem is required to be verified to ensure net positive suction head is
10		available for the ECCS injection/spray subsystem required to be operable by the
11		LCO. This Surveillance is required to be performed once per 12 hours or in
12		accordance with the Surveillance Frequency Control Program. The frequency
13		was chosen based on the availability of other indications available in the Control
14		Room regarding suppression pool water level and Condensate Storage Tank
15		level.
16	0	The surveillance requirements to verify the piping is full of water and to verify
17		correct valve alignment was retained from the existing TS 3.5.2.
18	0	The required ECCS injection/spray subsystem is required to be operated through
19		its recirculation line for \ge 10 minutes every 92 days or in accordance with the
20		Surveillance Frequency Control Program. This demonstrates that the subsystem
21		is capable for operation. The time limit is based on engineering judgement. The
22		frequency is consistent with other at-power testing.
23	0	, , , , , , , , , , , , , , , , , , , ,
24		actuate to a simulated actuation signal is required every 18 months or
25		accordance with the Surveillance Frequency Control Program. The frequency
26		was selected because it is desirable to perform the surveillance during shutdown
27		conditions to avoid operational transients.
28	0	
29		manual actuation signal is required every 18 months or accordance with the
30		Surveillance Frequency Control Program. The frequency was selected because
31		it is desirable to perform the surveillance during shutdown conditions to avoid
32		operational transients.
33		
34 25		ces section cites the applicable operating generic correspondence describing
35 36		perience related to inventory control during shutdown conditions. It lists Information "Inadvertent Reduction in Primary Coolant Inventory in Boiling Water Reactors
37		
38		lant Inventory Because of Misalignment of RHR Valves," August 1986; Generic
39	Letter 92-04,	"Resolution of the Issues Related to Reactor Vessel Water Level Instrumentation
38	Reactor Coo Letter 92-04,	

40 41

in BWRs Pursuant to 10 CFR 50.54(*Ff*), "August 1992; NRC Bulletin 93-03, "Resolution of Issues Related to Reactor Vessel Water Level Instrumentation in BWRs," May 1993; Information Notice 94-52, "Inadvertent Containment Spray and Reactor Vessel Draindown at 42

Millstone 1," July 1994; and General Electric Service Information Letter No. 388, "RHR Valve
 Misalignment During Shutdown Cooling Operation for BWR 3/4/5/6," February 1983.

3

4 The revised Bases (Volume 2 of NUREG 1434) for TS 3.5.2 for the BWR/6 differ from the

5 revised Bases for *NUREG-1433the BWR/4 TS*. The major difference is that the LCO requires

6 one ECCS injection/spray subsystem to be operable. The ECCS injection/spray subsystem is

7 either one of the three Low Pressure Coolant Injection subsystems, one Low Pressure Core

8 Spray System, or one High Pressure Core Spray System. This difference is reflected

9 throughout the Bases for TS 3.5.2 in NUREG 1434, Volume 2.

10

11 The staff reviewed the revised bases to ensure the applicable criterion from 10 CFR 50.36 is 12 identified and justified, the reasons for the selection of each instrument function and required 13 number of channels in the LCO is described, the reason for the applicable modes is stated, the 14 purpose of each required action is described, and the purpose of each surveillance and the 15 basis for the performance frequency is addressed, and appropriate references are cited. The 16 staff concluded that each of the elements of the Final Policy Statement were satisfactorily 17 addressed. Therefore, the staff determined that the revised Bases adheres to the guidance 18 provided in the Final Policy Statement.

19

23

20 21

3.3 EVALUATION OF ADDITIONAL BASES CHANGES

22 3.3.1 <u>B 3.3.5.1, ECCS Instrumentation</u>

24 The Bases for several instrument functions related to automatic ECCS initiation were revised to 25 reflect a revised Applicability. The functions would no longer be required during Modes 4 and 5 26 because of the relatively slow transient of unexpected drain events. It is judged that sufficient 27 time is permitted for operators to mitigate such a transient. The instrumentation affected in 28 NUREG-1433 is for BWR/4s are Reactor Vessel Water Level – Low Low Low, Level 1; Low 29 Pressure Coolant Injection (LPCI) System Reactor Vessel Water Level – Low Low, Level 1; 30 and Low Pressure Coolant Injection Pump Start – Time Delay Relay. For BWR/6NUREG-31 1434's6's, the affected functions are LPCI A and LPCS: Reactor Vessel Water Level – Low Low 32 Low, Level 1; LPCI Pump A Start – Time Delay Relay; LPCI B and C: Reactor Vessel Water 33 Level – Low Low, Level 1; LPCI Pump B Start – Time Delay Relay; and HPCS Reactor 34 Vessel Water Level Lo Low, Level 2. 35 The remainder of the changes to the Bases for this LCO reflect the relocation of instrumentation 36 37 function requirements to the LCO 3.3.5.2.

- 38
- 39 3.3.2 <u>B 3.3.6.1, Primary Containment Isolation Instrumentation</u>
- 40

The Bases for the Shutdown Cooling Isolation, Reactor Vessel Water Level – Low, Level 3 was
revised to reflect the relocation of this requirement to LCO 3.3.5.2.

1 2 3 4	For BWR/6NUREG-1434's6's, the function for Primary Containment Isolation, Containment and Drywell Ventilation Exhaust Radiation – High is revised to reflect the deletion of this requirement.
5 6 7	3.3.2 <u>B 3.5.3, RCIC System</u>
7 8 9 10	The Applicability was revised to state that in Modes 4 and 5, RPV water inventory control is provided by LCO 3.5.2.
10 11 12	3.3.3 <u>B 3.6.1.3, PCIVs [Primary Containment Isolation Valves]</u>
13 14 15 16	The Applicability was changed to replace the statement that certain valves are required to be operable to prevent inadvertent drain down to state that certain valves are required to be operable when the associated instrumentation is required to be operable.
10 17 18	The description of the Applicability and Actions is revised to delete the discussion of OPDRVs.
19 20	3.3.4 Other Affected Bases
21 22 23 24	The description of the Applicability, Actions and Applicable Safety Analyses Sections are revised to delete the discussion of OPDRVs-or, inadvertent drain down of the vessel, or other related administrative changes for the following LCOs:
25 26 27 28 29 30 31 32 33 34 35 36 37	 3.3.6.1 Primary2 Secondary Containment Isolation Instrumentation 3.3.7.1 MCREC System Instrumentation 3.5.1 ECCS - Operating 3.6.2.2 Suppression Pool Water Level 3.6.4.1 [Secondary] Containment 3.6.4.2 SCIVs [Secondary Containment Isolation Valves] 3.6.4.3 SGT System [Standby Gas Treatment] 3.7.4 MCREC [Main Control Room Environmental Control] System 3.7.5 Control Room AC [Air Conditioning] System 3.8.2 AC Sources – Shutdown 3.8.8 Inverters – Shutdown 3.8.10 Distribution Systems – Shutdown
38 39	3.10.1 Inservice Leak and Hydrostatic Testing Operation
40	The NRC staff reviewed the revised Bases sections and concluded that the revisions accurately

41 reflect the changes contained in the associated LCO's. The Applicability, Actions and

Applicable Safety Analyses sections continue to contain information regarding the reasons for 1 2 each of the LCO requirements. The staff determined that the Bases for the LCO's continue to 3 satisfy the guidance in the Final Policy Statement.

5 4.0 **CONCLUSION**

6

4

7 The NRC staff determined that TS Bases changes are consistent with the proposed TS changes 8 and provide an explanation and supporting information for each requirement in the specification. 9 Therefore, the staff determined that the revised Bases are consistent with the Commission's 10 Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors,

11 dated July 2, 1993 (58 FR 39132).

12

Attachment 2

TSTF Comments on the TSTF-542 Draft Model Safety Evaluation for Plant-Specific Adoption

Summary Table of TSTF Comments

Section and Location	Comment
	Generic Comment 1: The model application applies to changes to plant-specific TS. While NUREG-1433 and NUREG-1434 are titled as the BWR/4 and BWR/6 STS, respectively, they are also representative of BWR/2, BWR/3, and BWR/5 designs. Therefore, plant-type references in the model SE reviewer guidance are revised in multiple locations to include all applicable BWR designs in order to assist the NRC reviewer in selecting the correct description for the plant-specific SE.
	General Comment 2: The NUREG-1433 and NUREG-1434 TS contain "A" and "B" version of the instrumentation TS. The "B" versions are representative of plants with a Setpoint Control Program and the "A" versions are representative of plants that do not have a Setpoint Control Program. In both cases, the plant- specific TS would not include the designation "A" or "B" to the reference to the Setpoint Control Program in the TS titles (for example, the plant TS would include TS 3.3.5.1, not TS 3.3.5.1A or TS 3.3.5.1B.) Therefore, the model SE is revised in many locations to not discuss "A" and "B" versions of instrumentation TS.
1.0, page 1, line 23	Revised the OPDRV definition to be consistent with the STS.
1.0, page 1, line 41	Editorial correction to eliminate the word "do"
2.2, page 2, line 31	Editorial correction from "have" to "has"
2.2, page 2, lines 33-34	Revised the sentence to not imply that a potential to drain will result in loss of core cooling.
2.2, page 2, lines 37-41, page 3, lines 1-5	The Modes for BWRs are combinations of reactor mode switch position and reactor coolant temperature. Changes are made to be consistent with Table 1-1 of the STS.
2.2, page 3, line 10	Added BWR design option.
2.3.1, page 4, lines 20 and 22	Revised the text to be consistent with TSTF-542.

Section and	Comment
Location	
2.3.2.3, page	Added brackets to the discussion of the LCO 3.5.2 Note, as many plants have
5, lines 32-	removed this note from their TS.
40, 44, and	
45	
2.3.2.3, page	Changed "is" to "may be" as other ECCS subsystems besides high pressure core
5, line 45	spray may be used to satisfy the LCO.
2.3.2.5,	Since this section describes the changes for all BWR plant types, "secondary
Page 6, lines	containment" should be bracketed and the reference to standby gas should also
19-20	be bracketed.
2.3.2.6, page 6, lines 33- 35	The model SE is a written for a specific plant, so only one option applies.
2.3.3, page 6, lines 39- 47	Deleted note per General Comment 2.
2.3.3, page	Added a Reviewers Note pointing out that some BWRs do not have the
7, lines 1-2	capability to perform channel checks. If the existing TS do not have channel checks, the channel checks are not added to TS 3.3.5.1.
2.3.3.2, page 7, line 22	Changed "four" to "two." The TSTF refers to four TS (3.3.6.2A and B, and 3.3.7.1A and B). However, for an applicant using either the A or B version, there
	are only two.
2.3.3.2, page	Add a reviewer's note stating that an acceptable variation in the model
7, lines 25- 26	application is not renumber 3.3.5.2 and to make the new TS 3.3.5.3.
2.3.3.2.2.1	The introductory note at the top of page 1 states that plant-specific information is
through	in brackets and in bold text. The plant-specific current function number are in
2.3.3.2.2.9,	bold, but are not in brackets. Brackets are added.
pages 8	
through 14	
2.3.3.2.2.9,	Corrected section number.
page 14, line	
35	
2.3.3.2.2.9,	Added the acceptable deviations regarding plant-specific instruments from the
page 15,	model application.
lines 13-25	
2.3.3.3, page	Brackets are added to indicate plant-specific information. BWR design
17, lines 3-4	differences should be reflected in SE by using the terms "[secondary]
	containment" and "[secondary containment]." Also, some BWR/6 plants do not
	have a Standby Gas Treatment System.
2.3.3.3, page	Added the acceptable deviations discussion from the model application.
17, lines 10-	
13	

Section and Location	Comment
2.3.3.5, page 18, lines 10- 18	Added the acceptable deviations discussion from the model application.
3.2, page 20, lines 36-42	Simplified the presentation.
3.3, page 21, lines 26-27	There is no discussion of the acceptability of the Applicability. A discussion is added.
3.3, page 21, line 49	Added brackets around discussion of the Standby Gas Treatment System and secondary containment to reflect BWR/6 plant design differences.
3.3, page 22, lines 1-8	Added brackets around discussion of the Standby Gas Treatment System and secondary containment to reflect BWR/6 plant design differences.
3.3, page 22, line 23	There is no discussion of the TS 3.5.2 Surveillance Requirement changes.
3.3, page 22, lines 24-27	This paragraph is misplaced and is moved to Section 3.4.
3.4.2, page 23, line 48	Changed reference from 10 CFR 50.36(c)(3) to 10 CFR 50.36(c)(2)(i). Paragraph (c)(3) discusses SRs, not actions.
3.4.3, page 25, lines 1 and 4	Editorial corrections. Deleted unnecessary comma and bracket.
3.4.3, page 25, line 21	Editorial correction.
3.4.3, page 25, lines 22- 23	Note (b) under Applicability for these functions specifically says they are required when credited in calculating Drain Time. The discussion is corrected.
3.4.4, page 26, line 49, and page 27, lines 1, 8, 9	Added missing brackets around plant-specific information.
3.4.4, page 28, line 10	Editorial correction. Changed "ensure" to "ensures"
3.7, page 33, line 7	TS Table 1.1-1 indicates the Mode change temperature is plant-specific. The value 200 is placed in brackets.

Draft Model Safety Evaluation Mark-Up

1 2 3 4 5 6	General Directions: This model SE provides the format and content to be used when preparing the plant-specific SE of an LAR to adopt TSTF-542. The bolded bracketed information shows text that should be filled in for the specific amendment; individual licensees would furnish site-specific nomenclature or values for these bracketed items. The italicized wording provides guidance on what should be included in each section and should not be included in the SE.
7	DRAFT MODEL SAFETY EVALUATION
8	BY THE OFFICE OF NUCLEAR REACTOR REGULATION
9	TECHNICAL SPECIFICATIONS TASK FORCE TRAVELER
10	TSTF-542, REVISION 2
11	"REACTOR PRESSURE VESSEL WATER INVENTORY CONTROL"
12 13 14	1.0 INTRODUCTION
15 16 17 18 19 20 21	By application dated [enter date] (Agencywide Documents Access and Management System (ADAMS) Accession No. [MLXXXXXXX]), [name of licensee] (the licensee) requested to adopt Technical Specifications Task Force (TSTF) Traveler TSTF-542, "Reactor Pressure Vessel Water Inventory Control," Revision 2, which changes to the technical specifications (TS) for [PLANT] . Traveler TSTF-542, Revision 2, was approved by the NRC on [enter date] (ADAMS Accession No. [MLXXXXXXX]).
22 23 24 25 26	The proposed changes would replace the existing requirements in the TS related to "operations <i>with a which have the</i> potential for draining the reactor vessel" (OPDRVs) with revised TS providing an alternative for Reactor Pressure Vessel Water Inventory Control (RPV WIC). These alternative requirements would protect Safety Limit 2.1.1.3, which requires reactor pressure vessel (RPV) water level to be greater than the top of the active fuel (TAF).
27 28 29 30	Choose applicable paragraphs based on information provided in the LAR: [The licensee is not proposing any variations from the TS changes described in the TSTF-542 or the applicable parts of the NRC staff's safety evaluation of TSTF-542.]
31 32 33 34	[The licensee is proposing the following variations from the TS changes described in the TSTF-542 or the applicable parts of TSTF-542 or the NRC staff's safety evaluation.]
35 36 37 38 39 40	[The [PLANT] TS utilize different [numbering][and][titles] than the Standard Technical Specifications on which TSTF-542 was based. Specifically, [describe differences between the plant-specific TS numbering and/or titles and the TSTF-542 numbering and titles.] These differences are administrative and do not affect the applicability of TSTF-542 to the [PLANT] TS.]
41 42 43 44	[The [PLANT] TS limiting condition for operation (LCO) 3.5.2 does do-not contain a Note regarding realignment to the Low Pressure Coolant Injection mode. This has no effect on the adoption of the TSTF-542 and is an acceptable variation.]

Enclosure 2

1 2.0 REGULATORY EVALUATION

3 2.1 <u>TECHNICAL SPECIFICATIONS</u>

Section IV, "The Commission Policy," of the Final Policy Statement on Technical Specifications
Improvements for Nuclear Power Reactors (58 *Federal Register* 39132), dated July 22, 1993,
states in part:

- 9 The purpose of Technical Specifications is to impose those 10 conditions or limitations upon reactor operation necessary to 11 obviate the possibility of an abnormal situation or event giving rise 12 to an immediate threat to the public health and safety by 13 identifying those features that are of controlling importance to 14 safety and establishing on them certain conditions of operation 15 which cannot be changed without prior Commission approval.
- 16[T]he Commission will also entertain requests to adopt portions of17the improved STS [(e.g., TSTF-542)], even if the licensee does18not adopt all STS improvements...
- 19The Commission encourages all licensees who submit Technical20Specification related submittals based on this Policy Statement to21emphasize human factors principles...
- In accordance with this Policy Statement, improved STS have
 been developed and will be maintained for [the BWR/4 and
 BWR/6 designs]. The Commission encourages licensees to use
 the STS as the basis for plant-specific Technical Specifications...
- 26[I]t is the Commission intent that the wording and Bases of the27improved STS be used [] to the extent practicable.
- 28

29 2.2 SYSTEM DESCRIPTION

The boiling water reactor (BWR) RPV hasve a number of penetrations located below the TAF.
These penetrations provide entry for control blades, recirculation flow, and shutdown cooling.
Since these penetrations are below the TAF, this gives creates a potential to drain the reactor
vessel water inventory and thus lose effective core cooling. The loss of water inventory and
effective core cooling can potentially lead to fuel cladding failure and radioactive release.

37 During operation in Modes 1 (Power Operation - *Reactor Mode Switch in Run*)-with reactor

38 mode switch position in run), 2 (Startup - with *R*reactor *M*mode Sswitch position in *R*refuel (with

39 *all reactor vessel head closure bolts fully tensioned*) or Sstartup/Hhot Sstandby), and 3 (Hot

40 Standby - with Rreactor mode Mode Sswitch position in Run and average reactor coolant

41 *temperature* > [200] ${}^{\circ}F$), shutdown), the TS for instrumentation and emergency core cooling

systems (ECCS) require operability of sufficient equipment to ensure large quantities of water
 can be injected into the vessel should level decrease below the preselected value. These

44 requirements are designed to mitigate the effects of a loss-of-coolant accident (LOCA), but also

44 provide protection for other accidents and transients that involve a water inventory loss.

46

During BWR operation in Mode 4 (Cold Shutdown with - Reactor Mode Switch in Shutdown with
 all reactor vessel head closure bolts fully tensioned and average reactor coolant temperature

3 ≤ [200] °F), and Mode 5 (Refueling with - One or more reactor vessel head closure bolts less

4 than fully tensioned and Reactor Mode Switch in Shutdown or Refuelone or more reactor vessel

5 head closure bolts less than fully tensioned), the pressures and temperatures that could cause a

6 LOCA are not present. During certain phases of refueling (Mode 5) a large volume of water is

7 available above the RPV (i.e., the RPV head is removed, the water level is \geq [23 feet] over the

8 top of the RPV flange, and [for BWR/2, /3, /4, and /5 plants enter "the spent fuel storage pool

9 gates are removed" or for BWR/6 plants enter "the upper containment pool is connected to

10 the RPV" or upper containment cavity to dryer pool gate removed"].

11

The large volume of water available in and above the RPV (during much of the time when in Mode 5) provides time for operator detection and manual operator action to stop and mitigate an RPV draining event. However, typically at other times during a refueling outage, during cold shutdown (Mode 4) or refueling (Mode 5), there may be a potential for significant drainage paths from certain outage activities, human error, and other events when it is more likely to have some normally available equipment, instrumentation, and systems inoperable due to maintenance and outage activities. There may not be as much time for operator action as compared to times

19 when there are large volumes of water above the RPV.

20

In comparison to Modes 1, 2, and 3, with typical high temperatures and pressures (especially in
Modes 1 and 2), Modes 4 and 5 generally do not have the high pressure and temperature
considered necessary for a LOCA envisioned from a high energy pipe failure. Thus, while the
potential sudden loss of large volumes of water from a LOCA are not expected, operators
monitor for BWR RPV water level decrease from potential significant or even unexpected
drainage paths. These potential drainage paths in Modes 4 and 5 generally would require less
water replacement capability to maintain water above TAF.

28

To address the drain down potential during Modes 4 and 5, the current TS contain specifications that are applicable during an OPDRV, or require suspension of OPDRVs if certain equipment is inoperable. The term OPDRV is not specifically defined in the TS and historically has been subject to inconsistent application by licensees. The changes discussed in this safety evaluation (SE) are intended to resolve any ambiguity by creating a new RPV WIC TS with attendant equipment operability requirements, required actions and surveillance requirements, and deleting references to OPDRVs throughout the TS.

37 2.3 CHANGES TO THE TS

The proposed changes would (1) provide a definition of a new term, DRAIN TIME; (2) revise
and rename TS 3.5.2 as "Reactor Pressure Vessel Water Inventory Control;" (3) provide a new
TS 3.3.5.2, "Reactor Pressure Vessel Water Inventory Control Instrumentation;" and (4) delete
existing references to "operations with the potential to drain the reactor pressure vessel"
throughout the TS. The descriptions of the proposed changes are provided in this section.

44

A summary statement of the bases or reasons for such specifications, other than those covering
administrative controls, were also included in the application, but these bases shall not become
part of the technical specifications.

48

49 2.3.1 Insertion of New Definition of DRAIN TIME

1 2 3 4	The following definition of "DRAIN TIME" would be added to the TS Section 1.1, "Definitions":
5 6 7 8	The DRAIN TIME is the time it would take for the water inventory in and above the Reactor Pressure Vessel (RPV) to drain to the top of the active fuel (TAF) seated in the RPV assuming:
9 10 11	 a) The water inventory above the TAF is divided by the limiting drain rate;
12 13 14 15 16 17	b) The limiting drain rate is the larger of the drain rate through a single penetration flow path with the highest flow rate, or the sum of the drain rates through multiple penetration flow paths susceptible to a common mode failure (e.g., seismic event, loss of normal power, single human error), for all penetration flow paths below the TAF except:
18 19 20 21 22 23 24	 Penetration flow paths connected to an intact closed system, or isolated by manual or automatic valves <i>that</i> are locked, sealed, or otherwise secured in the closed position, blank flanges, or other devices that prevent flow <u>or-of</u> reactor coolant through the penetration flow paths;
25 26 27 28 29	 Penetration flow paths capable of being isolated by valves that will close automatically without offsite power prior to the RPV water level being equal to the TAF when actuated by RPV water level isolation instrumentation; or
30 31 32 33 34 35 36	 Penetration flow paths with isolation devices that can be closed prior to the RPV water level being equal to the TAF by a dedicated operator trained in the task, who is in continuous communication with the control room, is stationed at the controls, and is capable of closing the penetration flow path isolation device without offsite power.
30 37 38 39 40 41	 c) The penetration flow paths required to be evaluated per paragraph b) are assumed to open instantaneously and are not subsequently isolated, and no water is assumed to be subsequently added to the RPV water inventory;
42 43	d) No additional draining events occur; and
44 45	e) Realistic cross-sectional areas and drain rates are used.
46 47 48 49	A bounding DRAIN TIME may be used in lieu of a calculated value.

- 1 2.3.2 <u>Changes to TS Section 3.5</u>: 2
- 3 2.3.2.1 Title of TS 3.5

The title of Section 3.5 is being revised from "Emergency Core Cooling System (ECCS) and
Reactor Core Isolation Cooling System (RCIC)" to "Emergency Core Cooling Systems (ECCS),
RPV Water Inventory Control, and Reactor Core Isolation Cooling (RCIC) System."

9 2.3.2.2 Title of TS 3.5.2

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The title of TS 3.5.2 is being revised from "ECCS – Shutdown" to "Reactor Pressure Vessel
 (RPV) Water Inventory Control."

14 2.3.2.3 LCO 3.5.2

15
16 TS limiting condition for operation (LCO) 3.5.2 currently states "Two low pressure ECCS
17 injection/spray subsystems shall be OPERABLE." [The LCO note currently states: "One LPCI
18 subsystem may be considered OPERABLE during alignment and operation for decay heat
19 removal if capable of being manually realigned and not otherwise inoperable."]

- 20 21 For BWR/2, /3, /4 plants choose:
- 22 [LCO 3.5.2 would be revised to state:
 - DRAIN TIME of RPV water inventory to the top of active fuel (TAF) shall be \geq 36 hours.
 - AND
 - One low pressure ECCS injection/spray subsystem shall be OPERABLE.
- 34 **[The note for LCO 3.5.2 would be revised to state:**
 - A Low Pressure Coolant Injection (LPCI) subsystem may be considered OPERABLE during alignment and operation for decay heat removal if capable of being manually realigned and not otherwise inoperable.]]
- 4243 For BWR/5 and /6 plants choose:
- 44 [The phrase "low pressure" in LCO 3.5.2 is omitted because the high pressure core spray
- 45 system may be is used to satisfy this requirement.]
- 46

1 2

2.3.2.4 Applicability of TS LCO 3.5.2

For BWR/2, /3, /4 and /5 plants choose: [LCO 3.5.2 is currently applicable in MODE 4 and in MODE 5, except with the spent fuel storage pool gates removed and water level \ge [23 ft] over the top of the reactor pressure vessel flange.]

For BWR/6 plants choose: [LCO 3.5.2 is currently applicable in Mode 4 and Mode 5 except with
the upper containment [cavity to dryer] pool [gate] removed and water level ≥ [22 ft 8 inches]
over the top of the reactor pressure vessel flange.]

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11 The applicability would be revised to be Modes 4 and 5, with no exceptions.

13 2.3.2.5 Actions Table of TS 3.5.2

15 The existing Actions Table of TS 3.5.2 contains requirements to restore at least one train of 16 ECCS injection/spray systems to operable status if the LCO is not met.

The revised TS 3.5.2 Actions Table would provide increasingly stringent requirements on [secondary] containment], [secondary] containment] isolation valves, [the standby gas treatment system] and methods for water injection as the Drain Time decreases. If the Drain Time is one hour or less, immediate action must be taken to increase the Drain Time.

23 2.3.2.6 TS 3.5.2 Surveillance Requirements

TS 3.5.2 currently contains Surveillance Requirements (SRs) to verify the availability of a
 suction source, the availability of an appropriate flow path, and proper functioning of the ECCS
 injection/spray system pump(s).

28

The revised SRs would verify the Drain Time is greater than or equal to 36 hours and verify the
 availability of a suction source, appropriate flow path and proper functioning of the required
 ECCS injection/spray system pump.

The existing and proposed TS 3.5.2 Surveillances *Frequencies [are described below][provide* the option to perform the Surveillances at a fixed interval or are in accordance with the
 Surveillance Frequency Control Program (SFCP), for those plants that have adopted an SFCP].

3637 2.3.3 Changes to TS Section 3.3:

38
 39 NOTE: The STS contain two versions of certain specifications in Section 3.3, Instrumentation.

40 One is applicable for licensees that have not adopted a Setpoint Control Program (the "A"

41 version) and the other is applicable for licensees that have adopted a Setpoint Control Program

42 (the "B" version). In the "A" version of the STS, the Allowable Value column is retained in the

- 43 Instrumentation Table, and the Instrumentation Table contains footnotes that provide details
- 44 regarding SRs. In the "B" version of the STS, the Allowable Value has been relocated to the
- 45 Setpoint Control Program, and this column does not appear in the Instrumentation Table.
- Additionally, in the "B" version, the footnotes that provide details regarding SRs are not
 necessary. Choose the A or B version below to correspond with the plant-specific TS.
- 48

[Reviewer's Note: Some BWRs do not have the capability to perform Channel Checks. If the 1 2 existing TS do not include Channel Checks, TS 3.3.5.1 will not add Channel Checks.] 3 4 2.3.3.1 Changes to TS LCOs [3.3.5.1A or 3.3.5.1B], Emergency Core Cooling System 5 (ECCS) Instrumentation ([Without or With Setpoint Control Program]) 6 7 The TS LCO [3.3.5.1A or 3.3.5.1B] states that "the ECCS instrumentation for each Function in 8 Table 3.3.5.1-1, shall be OPERABLE" with the applicability as stated in the table. 9 Table 3.3.5.1-1 currently contains requirements for function operability during Modes 4 and 5 10 when associated ECCS subsystem(s) are required to be operable per LCO 3.5.2, "ECCS -11 Shutdown." Throughout this table, the applicability in Modes 4 and 5 is being deleted because 12 the instrumentation requirements during shutdown are being consolidated into the new TS 13 3.3.5.2. Conforming changes are made to the Actions Table of TS LCO [3.3.5.1 A or 3.3.5.1 B]. 14 15 2.3.3.2 Insertion of new TS [3.3.5.2A or 3.3.5.2B], Reactor Pressure Vessel (RPV) 16 Water Inventory Control Instrumentation ([Without or With Setpoint Control 17 Program]) 18 19 A new TS 3.3.5.2 is proposed to provide alternative instrumentation requirements to support 20 manual initiation of the ECCS injection/spray subsystem required in new TS 3.5.2 and automatic 21 isolation of penetration flow paths that may be credited in the determination of drain time. The 22 current TS contain instrumentation requirements related to OPDRVs in four-two TS. These 23 requirements are being consolidated into new TS 3.3.5.2. 24 25 [Reviewer's Note: An acceptable variation in the model application is to not renumber 3.3.5.2 26 and to number the new TS 3.3.5.3.] [The existing TS 3.3.5.2, "Reactor Core Isolation Cooling 27 (RCIC) System Instrumentation," is being renumbered to 3.3.5.3 in order to maintain the TS 28 numbering conventions.] 29 30 2.3.3.2.1 New TS 3.3.5.2[A or B] LCO and Applicability 31 32 The proposed LCO 3.3.5.2 states: 33 34 The RPV Water Inventory Control instrumentation for each 35 Function in Table 3.3.5.2-1 shall be OPERABLE. 36 37 The applicability states, "According to Table 3.3.5.2-1." 38 39 The following sections describe the instrumentation functions contained in the new 40 Table 3.3.5.2-1. 41 42 2.3.3.2.2 New Table 3.3.5.2-1, RPV Water Inventory Control Instrumentation 43

44 For BWR/2, /3, or /4 choose 2.3.3.2.2.1 through 2.3.3.2.2.5:

1 2 3 4 5 6	2.3.3.2.2.1	Function 1.a, Core Spray System, Reactor Steam Dome Pressure - Low (Injection Permissive) Function 2.a, Low Pressure Coolant Injection (LPCI) System, Reactor Steam Dome Pressure - Low (Injection Permissive)			
6 7 8 9		ons were moved from current TS 3.3.5.1, Function [1.c] and Function [2.c] . The anges are made:			
9 10 11 12 13 14 15	Modes 4 ECCS s revised	blicability is changed. The existing TS 3.3.5.1 applicability for these functions in 4 and 5 is modified by a note that limits the applicability to when the associated ubsystem(s) are required to be operable per LCO 3.5.2, "ECCS - Shutdown." The applicability is Modes 4 and 5 without exception, to be consistent with the bility of new LCO 3.5.2, "RPV Water Inventory Control."			
16 17	• The nur	nber of required channels per function is unchanged.			
18 19 20 21	frequen	ew table, a Channel Check and Channel Functional Test are required at the existing cy. Calibration of the trip units, Channel Calibration, Logic System Functional Test, CS Response Time tests are no longer required in Modes 4 and 5.			
22 23 24 25	to retain	CO 3.3.5.2A, the Allowable Value is revised to eliminate the low pressure limit and the high pressure limit. The RPV pressure is well below the lower limit in Modes 4 to the low pressure limit is not needed.			
26 27 28	2.3.3.2.2.2	Functions 1.b and 2.b, Core Spray and Low Pressure Coolant Injection (LPCI) Systems, Core Spray and Low Pressure Coolant Injection Pumps Discharge Flow - Low (Bypass)			
29 30 31		ons were moved from current TS 3.3.5.1, Function [1.d] and Function [2.g] . The anges are made:			
32 33 34 35 36 37 38 39 40 41 42	 The applicability is changed. The current TS 3.3.5.1 applicability for these functions in Modes 4 and 5 is modified by a note that limits the applicability to when the associated ECCS subsystem(s) are required to be operable per current LCO 3.5.2, "ECCS - Shutdown." The revised applicability is Modes 4 and 5 without exception, to be consistent with the applicability of new LCO 3.5.2, "RPV Water Inventory Control." 				
	 The number of required channels per function is changed from [2] or [4] or [1 per pump [1 per pump] and is modified by a note stating "Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory Control.'" 				
43 44 45 46 47	 In the new table, a Channel Check and Channel Functional Test are required at the existin frequency. A Channel Calibration and Logic System Functional Test are no longer require in Modes 4 and 5. 				
47 48	• In new L	CO 3.3.5.2A, the allowable value is unchanged.			

1 2 3	2.3.3.2.2.3	Function 1.c, Core Spray System, Manual Initiation, and Function 2.c, Low Pressure Coolant Injection (LPCI) System, Manual Initiation
4 5 6 7 8 9 10 11 12 13		ons were moved from current TS 3.3.5.1, Function [1.e] and Function [2.h] . The inges are made:
	Modes 4 ECCS su Shutdow	icability is changed. The current TS 3.3.5.1 applicability for these functions in and 5 is modified by a note that limits the applicability to when the associated ibsystem(s) are required to be operable per current LCO 3.5.2, "ECCS - n." The revised applicability is Modes 4 and 5 without exception, to be consistent applicability of new LCO 3.5.2, "RPV Water Inventory Control."
14 15 16 17 18 19	[1 per su required Control.'"	ber of required channels per function is changed from [2 , or 1 per subsystem,] to (bsystem) and is modified by a note stating "Associated with an ECCS subsystem to be OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory New LCO 3.5.2 only requires a single ECCS subsystem and the change in channels reflects that requirement.
20 21 22		existing TS 3.3.5.1 and the revised TS 3.3.5.2 require a Logic System Functional his function at the same frequency.
23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38	• There is	no allowable value for this function.
	2.3.3.2.2.4	Function 3.a, RHR System Isolation, Reactor Vessel Water Level - Low, Level 3
	This function made:	was moved from current TS 3.3.6.1, Function [6.b] . The following changes are
	Water Le Vessel W	tion name is changed from "Shutdown Cooling System Isolation Reactor Vessel vel - Low, Level 3" to "Residual Heat Removal [RHR] System Isolation Reactor /ater Level - Low, Level 3." The current title is a misnomer in the TSs as the nstruments isolate more than shutdown cooling isolation valves.
	Modes 4	icability is changed. The existing TS 3.3.6.1 applicability for this function in and 5 is being deleted. The revised applicability is "when automatic isolation of the ed penetration flow path is credited in calculating Drain Time."
39 40 41 42	"Require	ber of required channels is changed from [2] , with a column header that states d Channels per Trip System," to [2 in one trip system] . This retains the ent that the two channels must be associated with the same trip system.
43 44 45 46	frequenc	w table, a Channel Check and Channel Functional Test are required at the existing y. A calibration of the trip unit, Channel Calibration, and Logic System Functional no longer required in Modes 4 and 5.
47 48	• The allow	vable value is unchanged.

2.3.3.2.2.5 Function 4.a, Reactor Water Cleanup (RWCU) System Isolation, Reactor Vessel Water Level - Low Low, Level 2

- 4 This function exists in the current TS 3.3.6.1, Function **[5.e]**. The function is inserted into new 5 STS 3.3.5.2 as follows:
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The current TS 3.3.6.1 applicability for this function is Modes 1, 2, and 3. The applicability in STS 3.3.5.2 is "when automatic isolation of the associated penetration flow path is credited in calculating Drain Time." In other words, if the drain time calculation assumes the RWCU system will be automatically isolated, this function must be operable to perform that function. This is consistent with the definition of drain time and the TS 3.5.2 requirements.

- The number of required channels is changed from [2], with a column header that states
 "Required Channels per Trip System," to [2 in one trip system]. This retains the
 requirement that the two channels must be associated with the same trip system. Only one
 trip system is required to ensure that automatic isolation of one of the two isolation valves
 will occur on low reactor vessel water level.
- A Channel Check and Channel Functional Test are required at the existing frequency. A
 calibration of the trip unit, Channel Calibration, Logic System Functional Test, and Isolation
 System Response Time tests are no longer required in Modes 4 and 5.
- The allowable value is unchanged.

For BWR/<mark>5 and /</mark>6 choose 2.3.3.2.2.1 through 2.3.3.2.2.9:

- 27 2.3.3.2.2.1
 28 Function 1.a, Low Pressure Coolant Injection-A (LPCI) and Low Pressure Core
 29 Spray (LPCS) Subsystems, Reactor Steam Dome Pressure Low (Injection
 29 Permissive), and
 30 Function 2.a, LPCI B and LPCI C Subsystems, Reactor Steam Dome Pressure 31 Low (Injection Permissive)
- These functions were moved from current TS 3.3.5.1, Function **[1.d]** and Function **[2.d]**. The following changes are made:
- The applicability is changed. The current TS 3.3.5.1 applicability for these functions in
 Modes 4 and 5 is modified by a note that limits the applicability to when the associated
 ECCS subsystem(s) are required to be operable per current LCO 3.5.2, "ECCS Shutdown." The revised applicability is Modes 4 and 5 without exception, to be consistent
 with the applicability of new LCO 3.5.2, "RPV Water Inventory Control."
- In the new table, the number of required channels per function remains [3] and is modified by a note stating "Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory Control.'" New TS 3.5.2 only requires a single ECCS subsystem to be operable and the change reflects that requirement.

1 2 3 4	 A Channel Check and Channel Functional Test are required at the existing frequency. Calibration of the trip units, Channel Calibration, Logic System Functional Test, and ECCS Response Time tests are no longer required in Modes 4 and 5. 						
$\begin{array}{c} 5 \\ 6 \\ 7 \\ 8 \\ 9 \\ 10 \\ 11 \\ 12 \\ 13 \\ 14 \\ 15 \\ 16 \\ 17 \\ 18 \\ 19 \\ 20 \\ 21 \\ 22 \\ 23 \\ 24 \\ 25 \\ 26 \\ 27 \\ 28 \\ 29 \\ 30 \\ 31 \\ 32 \\ 33 \\ 4 \\ 35 \\ 36 \\ 37 \\ 38 \end{array}$		CO 3.3.5.2A, the allowable value is revised to eliminate the low pressure limit and the high pressure limit.					
	2.3.3.2.2.2	Functions 1.b and 1.c, Low Pressure Coolant Injection-A (LPCI) and Low Pressure Core Spray (LPCS) Subsystems, LPCS Pump Discharge Flow - Low (Bypass) and LPCI Pump A Discharge Flow – Low (Bypass), and Function 2.b, LPCI B and LPCI C Subsystems, LPCI Pump B and LPCI Pump C Discharge Flow – Low (Bypass)					
		ons were moved from current TS 3.3.5.1, Function [1.e] , [1.f] , and [2.e] . The inges are made:					
	 The applicability is changed. The current TS 3.3.5.1 applicability for these functions is Modes 4 and 5 when the associated ECCS subsystem(s) are required to be operable per LCO 3.5.2, "ECCS - Shutdown." The revised Applicability is Modes 4 and 5 without exception, to be consistent with the Applicability of new LCO 3.5.2, "RPV Water Inventory Control." 						
	modified by LCO 3 requires a	• The number of required channels per function is changed from [1] to [1 per pump] and is modified by a note stating "Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory Control'." New TS 3.5.2 only requires a single ECCS subsystem and the change in required channels reflects that requirement.					
	Calibratir	el Check and Channel Functional Test are required at the existing frequency. ng the trip unit, Channel Calibration and Logic System Functional Test are no longer in Modes 4 and 5.					
	• In new LO	CO 3.3.5.2 <mark>A</mark> , the allowable value is unchanged.					
	2.3.3.2.2.3	Function 1.d, Low Pressure Coolant Injection-A (LPCI) and Low Pressure Core Spray (LPCS) Subsystems, Manual Initiation, and Function 2.c, LPCI B and LPCI C Subsystems, Manual Initiation					
39 40	These functions were moved from current TS 3.3.5.1, Function [1.g] and Function [2.f] . The following changes are made:						
41 42 43 44 45 46 47	 The applicability is changed. The current TS 3.3.5.1 Applicability for these Functions in Modes 4 and 5 is modified by a note that limits the applicability to when the associated ECCS subsystem(s) are required to be operable per current LCO 3.5.2, "ECCS - Shutdown." The revised applicability is Modes 4 and 5 without exception, to be consistent with the applicability of new LCO 3.5.2, "RPV Water Inventory Control." 						

1 2 3 4 5 6 7 8 9 10 11	•	and is mo OPERAB TS 3.5.2 c	per of required channels per function is changed from [1] to [1 per subsystem] dified by a note stating "Associated with an ECCS subsystem required to be LE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory Control'." New only requires a single ECCS subsystem and the change in required channels at requirement.			
	•		existing TS 3.3.5.1 and the revised TS 3.3.5.2 require a Logic System Functional is function at the same frequency.			
	•	There is r	no allowable value for this function.			
12 13	2.3	3.3.2.2.4	Function 3.a, High Pressure Core Spray (HPCS) System, Reactor Vessel Water Level - High, Level 8			
14 15 16 17 18 19 20 21 22 23 24 25 26 27 28		is function ade:	was moved from current TS 3.3.5.1, Function [3.c] . The following changes are			
	 The applicability is changed. The current TS 3.3.5.1 applicability for this function is Mo and 5 when the associated ECCS subsystem(s) are required to be operable per existin LCO 3.5.2, "ECCS - Shutdown." The revised applicability is Modes 4 and 5 without exception, to be consistent with the applicability of new LCO 3.5.2, "RPV Water Invento Control." 					
	•	a note sta 3.5.2, 'Re	per of required channels per function is changed from [2] to [1] and is modified by ating "Associated with an ECCS subsystem required to be OPERABLE by LCO actor Pressure Vessel Water Inventory Control'." New TS 3.5.2 only requires a CS subsystem and the change in required channels reflects that requirement.			
29 30 31 32	•	Calibratio	el Check and Channel Functional Test are required at the existing frequency. n of the trip units, Channel Calibration, and Logic System Functional Test tests are required in Modes 4 and 5.			
33	•	The allow	able value in new LCO 3.3.5.2 <mark>A</mark> is unchanged.			
34 35 36 37 38 39 40 41 42 43 44 45 46 47 48	2.3	3.3.2.2.5	Function 3.b, High Pressure Core Spray (HPCS) System, Condensate Storage Tank Level – Low			
		is function ade:	was moved from current TS 3.3.5.1, Function [3.d] . The following changes are			
	• The applicability is changed. The current TS 3.3.5.1 applicability for this function is Modes 4 and 5 when the associated ECCS subsystem(s) are required to be operable per current LCO 3.5.2, "ECCS - Shutdown." The revised applicability is Modes 4 and 5 when HPCS is operable for compliance with new LCO 3.5.2 and aligned to the Condensate Storage Tank. If HPCS is not being credited for meeting the new LCO 3.5.2 requirement for an operable ECCS subsystem, or if HPCS is being credited but is aligned to the suppression pool, this function is unneeded.					

1 The number of required channels per function is changed from [2] to [1]. New TS 3.5.2 only • 2 requires a single ECCS subsystem to be operable, and the change in required channels 3 reflects that requirement. 4 5 A Channel Check and Channel Functional Test are required at the existing frequency. Calibration of the trip units, Channel Calibration, and Logic System Functional Test are no 6 7 longer required in Modes 4 and 5. 8 9 The allowable value in new LCO 3.3.5.2A is unchanged. • 10 11 2.3.3.2.2.6 Functions 3.c and 3.d, High Pressure Core Spray (HPCS) System, HPCS Pump Discharge Pressure - High (Bypass) and HPCS System Flow Rate - Low 12 13 (Bypass) 14 15 These functions were moved from current TS 3.3.5.1, Function [3.f] and [3.g]. The following 16 changes are made: 17 18 The applicability is changed. The current TS 3.3.5.1 applicability for this function is Modes 4 • 19 and 5 when the associated ECCS subsystem(s) are required to be operable per current 20 LCO 3.5.2, "ECCS - Shutdown." The revised applicability is Modes 4 and 5 without 21 exception, to be consistent with the applicability of new LCO 3.5.2, "RPV Water Inventory 22 Control." 23 24 The number of required channels per function is changed from [1] to [1 per pump] and is 25 modified by a note stating "Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory Control'." New TS 3.5.2 only 26 27 requires a single ECCS subsystem and the change in required channels reflects that 28 requirement. 29 30 A Channel Check and Channel Functional Test are required at the existing frequency. 31 Calibration of the trip units, Channel Calibration, and Logic System Functional Test are no 32 longer required in Modes 4 and 5. 33 34 • The allowable value is unchanged. 35 36 2.3.3.2.2.7 Function 3.e, High Pressure Core Spray (HPCS) System, Manual Initiation 37 38 This function is moved from current TS 3.3.5.1, Function [3.h]. The following changes are 39 made: 40 41 The applicability is changed. The current TS 3.3.5.1 applicability for these functions in 42 Modes 4 and 5 is modified by a note that limits the applicability to when the associated 43 ECCS subsystem(s) are required to be operable per existing LCO 3.5.2, "ECCS -Shutdown." The revised applicability is Modes 4 and 5 without exception, to be consistent 44 45 with the applicability of new LCO 3.5.2, "RPV Water Inventory Control." 46 47 The number of required channels per function is changed from [1] to [1 per subsystem] 48 and is modified by a note stating "Associated with an ECCS subsystem required to be

- 5 Both the existing TS 3.3.5.1 and the revised TS 3.3.5.2 require a Logic System Functional 6 Test on this function at the same frequency. 7 8 There is no allowable value for this function. • 9 10 Function 4.a, RHR System Isolation Reactor Vessel Water Level - Low, Level 3 2.3.3.2.2.8 11 12 This function was moved from current TS 3.3.6.1, Function **[5.c]**. The following changes are 13 made: 14 15 The function name is changed from "Shutdown Cooling System Isolation Reactor Vessel Water Level - Low, Level 3" to "Residual Heat Removal System Isolation Reactor Vessel 16 17 Water Level - Low, Level 3." 18 19 The applicability is changed. The current TS 3.3.6.1 applicability for this function is Modes 4 20 and 5. The revised applicability is "when automatic isolation of the associated penetration 21 flow path is credited in calculating drain time. 22 23 • The number of required channels is changed from [2], with a column header that states 24 "Required Channels per Trip System," to [2 in one trip system]. This retains the 25 requirement that the two channels must be associated with the same trip system. Only one trip system is required to ensure automatic isolation of one of the two isolation valves will 26 27 occur on low reactor vessel water level. 28 29 A Channel Check and Channel Functional Test are required at the existing frequency. A calibration of the trip unit, Channel Calibration, Logic System Functional Test, and Isolation 30 31 System Response Time tests are no longer required in Modes 4 and 5. 32 33 The existing allowable value is retained in new TS 3.3.5.2. • 34 35 2.<mark>32</mark>.3.2.2.9 Function 5.a, Reactor Water Cleanup (RWCU) System Isolation, Reactor Vessel 36 Water Level - Low Low, Level 2 37 38 This function exists in the current STS 3.3.6.1 as Function [4.k]. The function is inserted into 39 new STS 3.3.5.2 as follows: 40 41 The current STS 3.3.6.1 applicability for this function is Modes 1, 2, and 3. The applicability 42 in STS 3.3.5.2 is "when automatic isolation of the associated penetration flow path is 43 credited in calculating Drain Time." In other words, if the drain time calculation assumes the 44 RWCU system would be automatically isolated, this function must be operable to perform 45 that function. This is consistent with the definition of drain time and the new TS 3.5.2 46 requirements.

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reflects that requirement.

OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory Control'." New

TS 3.5.2 only requires a single ECCS subsystem and the change in required channels

1 The number of required channels is changed from [2], with a column header that states 2 "Required Channels per Trip System," to [2 in one trip system]. This retains the 3 requirement that the two channels must be associated with the same trip system. Only one 4 trip system is required to ensure that automatic isolation of one of the two isolation valves 5 will occur on low reactor vessel water level. 6 7 A Channel Check and Channel Functional Test are required at the existing frequency. A 8 calibration of the trip unit, Channel Calibration, Logic System Functional Test, and Isolation 9 System Response Time tests are no longer required in Modes 4 and 5. 10 11 The existing allowable value is retained in LCO 3.3.5.2A. • 12 13 Acceptable variations are the inclusion of any plant-specific instrumentation functions that: 14 Provide automatic initiation of ECCS water injection on low RPV water level. 15 Provide Residual Heat Removal (RHR) System isolation on low water level and/or, for 16 BWR/6 plants, isolate the primary containment and drywell ventilation exhaust. 17 Provide manual and automatic isolation of the [Secondary Containment] on low water 18 level. 19 Provide automatic isolation of the control room on low water level. 20 Provide automatic isolation of penetration flow paths below the TAF on low RPV water 21 level. 22 • Support manual initiation of an ECCS subsystem. 23 24 Changes to these instrumentation functions is justified by the discussion in Sections 3.3 and 25 3.4.1 of the TSTF-542 justification and are described in the licensee's application. 26 27 2.3.3.2.3 New TS 3.3.5.2[A or B] Actions Table 28 29 Condition A is applicable when one or more instrument channels are inoperable from 30 Table 3.3.5.2-1. Required Action A.1 directs immediate entry into the condition referenced in Table 3.3.5.2-1 for that channel. 31 32 33 Condition B is entered when the RHR system isolation and RWCU system isolation functions 34 operability requirements are not met when automatic isolation of the associated penetration flow 35 path is credited in calculating drain time. If the instrumentation is inoperable, Required 36 Action B.1 directs an immediate declaration that the associated penetration flow path(s) are 37 incapable of automatic isolation. Required Action B.2 requires an immediate calculation of drain 38 time. 39 40 Condition C is entered when the Low Reactor Steam Dome Pressure Injection Permissive 41 Functions necessary for ECCS subsystem manual initiation operability requirements are not 42 met. The channel must be placed in the trip condition within one hour. 43 44 For BWR/2, /3, or /4 plants choose: 45 [Condition D is entered when the operability requirements for the Core Spray Pump Discharge Flow – Low Bypass, Low Pressure Coolant Injection Pump Discharge Flow – Low Bypass, or 46 47 manual initiation of these functions operability requirements are not met. The Required Action

48 is to restore the channel to operable status within 24 hours.

4	
1	Condition F is optored if the required Action and approxisted Completion Time of Condition C or
2	Condition E is entered if the required Action and associated Completion Time of Condition C or
3	D, are not met. Required Action E.1 requires the associated low pressure ECCS injection/spray
4	subsystem to be declared inoperable immediately.]
5	
6	For BWR/ <mark>5 or /</mark> 6 plants choose:
7	[Condition D is entered when the Condensate Storage Tank Level –Low operability
8	requirements are not met. Required Action D requires declaring the HPCS inoperable and
9	aligning the HPCS pump suction to the suppression pool within one hour.
10	
11	Condition E is entered if the Reactor Vessel Water Level – High Level 8 instrumentation
12	operability requirements are not met. Action E.1 requires declaring the HCPS system
13	inoperable in 1 hour and restoring the channel to Operable status within 24 hours.
14	
15	Condition F is entered if the LPCS Pump Discharge Flow Low (Bypass), LPCI Pump A
16	Discharge Flow Low (Bypass), LPCI Pump B and LPCI Pump C Discharge Flow – Low
17	(Bypass), HPCS Pump Discharge Pressure – High (Bypass) HPCS System Flow Rate – Low –
18	(Bypass) or Manual Initiation associated with these Functions operability requirements are not
19	met. The required action is to restore the channel to OPERABLE status within 24 hours.
20	
20	Condition G is entered if the required action and associated completion time of Condition C, D,
22	E, or F is not met. Required Action G.1 requires the associated ECCS injection/spray
23	subsystem to be declared inoperable immediately.]
24	
25	2.3.3.2.4 New Surveillance Requirements 3.3.5.2.1, 3.3.5.2.2 and 3.3.5.3
26	
27	New Table 3.3.5.2-1 specifies which SRs apply for each ECCS function.
28	
29	SR 3.3.5.2.1 requires the performance of a Channel Check at a Frequency of [12 hours or in
30	accordance with the Surveillance Frequency Control Program.]
31	
32	SR 3.3.5.2.2 requires the performance of a Channel Functional Test at a Frequency of [[92]
33	days or in accordance with the Surveillance Frequency Control Program.]
34	
35	SR 3.3.5.2.3 requires the performance of a Logic System Functional Test at a Frequency of
36	[[18] months or in accordance with the Surveillance Frequency Control Program.]
37	
38	2.3.3.3 Changes to Containment, Containment Isolation Valve and Standby Gas
39	Treatment System Requirements
40	
41	The following TS are applicable during OPDRVs and/or contain Actions to suspend OPDRVs
42	when the LCO is not met:
43	
44	For BWR/2, /3, /4 or /54 plants choose:
45	[3.6.1.3, Primary Containment Isolation Valves (PCIVs)
46	3.6.4.1, [Secondary] Containment
47	3.6.4.2, Secondary Containment Isolation Valves (SCIVs)
48	3.6.4.3, Standby Gas Treatment System]
49	
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- For BWR/6 plants choose: 1
- 2 [3.6.1.3, Primary Containment Isolation Valves (PCIVs)
- 3.6.4.1, [Secondary] Containment/ 3
- 3.6.4.2, [Secondary Containment] Isolation Valves (SCIVs) 4
- 5 [3.6.4.3, Standby Gas Treatment System]
- 6

7 For each of these TS, the applicability and required action sections are being revised to delete 8 references to OPDRVs.

9

10 Acceptable variations are the inclusion of any plant-specific TS that that provide primary or 11 secondary containment, primary or secondary containment isolation valves, or standby gas 12 treatment functions. Changes to the TS controls on these systems is justified by the discussion

- 13 in Sections 3.4.2 and 3.4.3 of the TSTF-542 justification.
- 14
- 15 2.3.3.4 Changes to Control Room Habitability and Temperature Control Requirements

16 17 The following LCOs are applicable during OPDRVs and contain required actions to immediately 18 initiate action to suspend OPDRVs when certain conditions of the LCO are not met:

19

20 For BWR/4 plants choose:

- 21 [3.7.4, [Main Control Room Environmental Control (MCREC)] System
- 22 3.7.5, [Control Room Air Conditioning (AC)] System]
- 23 For BWR/6 plants choose: 24
- 25 [3.7.3, [Control Room Fresh Air (CRFA)] System
- 26 3.7.4, [Control Room AC] System]
- 27

28 The references to OPDRVs are being deleted from the applicability and required actions of 29 these TS.

- 30
- 31 2.3.3.5 Changes to Electrical Sources Requirements
- 32

33 The following TS are applicable in Modes 4 and 5 and currently contain a required action to 34 initiate action to suspend operations with a potential for draining the reactor vessel immediately

- 35 if certain conditions are not met:
- 36
- 37 3.8.2, AC Sources - Shutdown
- 3.8.5, DC Sources Shutdown 38
- 39 3.8.8, Inverters - Shutdown
- 40 3.8.10, Distribution Systems - Shutdown
- 41
- 42 TS 3.8.2 currently requires, in part, with one required offsite circuit inoperable or one required 43 diesel generator inoperable, to initiate action to suspend operations with a potential for draining 44 the reactor vessel immediately.

45

- 46 TS 3.8.5 currently requires, in part, with one **[or more**] required DC electrical power
- 47 subsystem[s] inoperable for reasons other than an inoperable battery charger, to initiate action
- 48 to suspend operations with a potential for draining the reactor vessel immediately
- 49

1 TS 3.8.3 currently requires, in part, with one **[or more] [required]** inverter**[s]** inoperable, to 2 initiate action to suspend operations with a potential for draining the reactor vessel immediately. 3

TS 3.8.10 currently requires, in part, with one or more required AC, DC, [or AC vital bus]
electrical power distribution subsystems inoperable, to initiate action to suspend operations with
a potential for draining the reactor vessel immediately.

8 These required actions are being deleted.

An acceptable deviation is the inclusion of plant-specific systems that provide the electrical power functions in the TS. Changes to the TS controls on these systems is justified by the discussion in Section 3.4.4 of the TSTF-542 justification.

13

9

Note: Insert description of any licensee specific TS changes. An acceptable variation from
TSTF-542 is the elimination of any plant-specific TS requirements related to OPDRVs, the
related concepts such as "RHR integrity maintained," and Required Actions to "suspend
OPDRVs" that do not appear in the NUREG-1433 and NUREG 1434. Changes to these TS
controls are justified by the discussion in the TSTF-542 justification.

20 2.4 <u>APPLICABLE REGULATORY REQUIREMENTS</u>

Pursuant to 10 CFR 50.90, whenever a holder of an operating license desires to amend the
license, application for an amendment must be filed with the Commission fully describing the
changes desired, and following as far as applicable, the form prescribed for original
applications. The technical information to be included in an application for an operating license
is governed in particular by 10 CFR 50.34(b).

27

21

10 CFR 50.36(a)(1) requires each applicant for a license authorizing operation of a utilization facility to include in the application proposed technical specifications in accordance with the requirements of 10 CFR 50.36. The regulation at 10 CFR Section 50.36(a)(1) requires an applicant to submit, as part of the application, a "summary statement of the bases or reasons for such specifications, other than those covering administrative controls." However, per 10 CFR 50.36(a)(1), these technical specification bases "shall not become part of the technical specifications."

35

36 As described in 10 CFR 50.92(a), in determining whether an amendment to a license will be 37 issued to the applicant, the Commission will be guided by the considerations which govern the 38 issuance of initial licenses applicable and appropriate. The general considerations that guide 39 the Commission include, as stated in 10 CFR 50.40(a), how the technical specifications provide 40 reasonable assurance the health and safety of the public will not be endangered. Also, to issue 41 an operating license, of which technical specifications are a part, the Commission must make 42 the findings of 10 CFR 50.57, including finding the 10 CFR 50.57(a)(3)(i) finding that there is 43 reasonable assurance that the activities authorized by the operating license can be conducted 44 without endangering the health and safety of the public. 45

As required by 10 CFR 50.36(b), the TS "will be derived from the analyses and evaluation
 included in the safety analysis report, and amendments thereto, submitted pursuant to 10 CFR

47 Included in the safety analysis report, and amendments thereto, submitted pursuant to 10 Cr (48 50.34 ["Contents of applications; technical information"]. The Commission may include such

49 additional technical specifications as the Commission finds appropriate."

1 2 3	The categories of items required to be in the TSs are provided in 10 CFR 50.36(c). As required by 10 CFR 50.36(c)(2)(i), the TSs will include LCOs, which are the lowest functional capability				
4 5 6	or performance levels of equipment required for safe operation of the facility. Per 10 CFR $50.36(c)(2)(i)$, when an LCO of a nuclear reactor is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the TSs until the condition can be met.				
7 8 9 10	The regulations at 10 CFR 50.36(c)(2)(ii) state that LCO's must be established for each item meeting one of four criteria:				
11 12 13 14	<i>Criterion 1.</i> Installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary.				
15 16 17 18	<i>Criterion 2.</i> A process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to fission product barrier integrity.				
19 20 21 22 23	<i>Criterion</i> 3. A structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier.				
24 25 26 27 28	<i>Criterion 4.</i> A structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety.				
29 30 31 32 33 34 35 36	The regulation at 10 CFR 50.36(c)(3) requires TSs to include items in the category of SRs, which are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the LCOs will be met. Also, the regulation at 10 CFR 50.36(a)(1) states that a summary statement of the bases or reasons for such specifications, other than those covering administrative controls, shall also be included in the application, but shall not become part of the TSs.				
37 38 39 40 41 42 43 44	The NRC staff's guidance for review of TSs is in Chapter 16, <i>Technical Specifications</i> , of NUREG-0800, Revision 3, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants" (SRP), dated March 2010, (ADAMS Accession No. ML100351425). As described therein, as part of the regulatory standardization effort, the NRC staff has prepared STS for each of the light-water reactor nuclear designs. <i>Choose applicable NUREG:</i> [NUREG-1433, Revision 4, contains the STS for BWR/4 plants, <i>and is also applicable to BWR/2, BWR/3, and in some cases, BWR/5 plants</i>] or [NUREG 1434, Revision 4, contains the STS for BWR/6 plants, <i>and is also applicable in some cases to BWR/5 plants</i>]].				
45 46 47 48 49	3.0 TECHNICAL EVALUATION 3.1 DRAIN TIME DEFINITION				
-					

1 The proposed drain time is the time it would take the RPV water inventory to drain from the

- current level to the TAF assuming the most limiting of the RPV penetrations flow paths with the
 largest flow rate, or a combination of penetration flow paths that could open due to a common
 mode failure, were to open.
- 5 6 The NRC staff reviewed the proposed drain time definition. For the purpose of NRC staff 7 considerations, the term "break" describes a pathway for water to drain from the RPV that has 8 not been prescribed in the proposed "DRAIN TIME" definition. All RPV penetrations below the 9 TAF are included in the determination of drain time as potential pathways. The drain time is 10 calculated by taking the water inventory above the break and dividing by the limiting drain rate 11 until the TAF is reached. The limiting drain rate is a variable parameter depending on the break 12 size and the reduction of elevation head above break location during the drain down event. The 13 discharge point will depend on the lowest potential drain point for each RPV penetration flow 14 path on a plant-specific basis. This calculation provides a conservative approach to determining 15 the drain time of the RPV.
- 17 3.2 WATER SOURCES
- 18 19 For BWR/2, /3 /4, or /5 plants choose:
- [The proposed LCO 3.5.2 states that, one low pressure Emergency Core Cooling System (ECCS) injection/spray subsystem shall be OPERABLE.]
- 22
- 23 For BWR/6 plants choose:
- 24 [The proposed LCO 3.5.2 states that, one ECCS injection/spray subsystem shall be
- 25 OPERABLE.]
- 26

16

27 The NRC staff reviewed the water sources that would be applicable to the proposed TS 3.5.2. The ECCS pumps are high-capacity pumps, with flow rates of thousands of gallons per minute 28 29 (qpm). Most RPV penetration flow paths would have a drain rate on the order of tens or 30 hundreds of gpm. The automatic initiation of an ECCS pump would provide the necessary water source to counter these expected drain rates. The LPCI subsystem is to be considered 31 32 operable during alignment and operation for decay heat removal if capable of being manually 33 realigned and not otherwise inoperable. Decay heat removal in MODEs 4 and 5 is not affected 34 by the proposed change as these requirements on the number of RHR shutdown cooling 35 subsystems that must be operable and in operation to ensure adequate decay heat removal 36 from the core are unchanged. For BWR/4 plants choose: [These requirements can be found in TS [3.4.9], "Residual Heat Removal (RHR) Shutdown Cooling System - Cold Shutdown." 37 38 TS [3.9.8], "Residual Heat Removal (RHR) – High Water Level, " and TS [3.9.10], "Residual 39 Heat Removal (RHR) – Low Water Level."] For the BWR/6 plants choose: [These requirements can be found in TS 3.4.10, "Residual Heat Removal (RHR) Shutdown Cooling 40 41 System - Cold Shutdown," TS 3.9.8, "Residual Heat Removal (RHR) - High Water Level, and TS 3.9.10, "Residual Heat Removal (RHR) – Low Water Level."] Based on these 42 43 considerations, the NRC staff finds the water sources provide assurances that the lowest 44 functional capability required for safe operation is maintained and supports the safety limit. 45

1 3.3 <u>TS 3.5.2 – REACTOR PRESSURE VESSEL (RPV) WATER INVENTORY CONTROL</u> 2

The proposed TS 3.5.2, "Reactor Pressure Vessel (RPV) Water Inventory Control," LCO
contains two parts. The first part states that drain time of RPV water inventory to the TAF shall
be ≥ 36 hours. For BWR/2, /3, of /4 choose: [The second part states, one low pressure
ECCS injection/spray subsystem shall be OPERABLE.] For BWR/5 or /6 plants choose:
[The second part states, one ECCS injection/spray subsystem shall be OPERABLE.] The
proposed applicability for TS 3.5.2 is MODEs 4 and 5.

- The NRC staff reviewed the proposed TS 3.5.2, focusing on ensuring the fuel remains covered with water and the changes made compared to the current TS. The proposed TS 3.5.2 contains Conditions A through E based on either required ECCS injection/spray subsystem operability or drain time.
- 14

15 The current TS LCO states that two ECCS injection/spray subsystems shall be operable,

whereas the proposed LCO 3.5.2 states that only one ECCS injection/spray subsystem shall be operable. This change is reflected in Condition A. The change from two ECCS injection/spray

- subsystem to one ECCS injection/spray subsystem is because this redundancy is not required.
 With one ECCS injection/spray subsystem and non-safety related injection sources, defense-indepth will be maintained. The defense-in-depth measure is consistent with other events
 considered during shutdown with no additional single failure assumed. The drain time controls,
 in addition to the required ECCS injection/spray subsystem, provide reasonable assurance that
 an unexpected draining event can be prevented or mitigated before the RPV water level would
- be lowered to the TAF.
- 25

The proposed Mode 4 and 5 applicability of TS 3.5.2 is appropriate given the unaffected TS requirements on ECCS and RPV water level in Modes 1, 2, and 3.

28

The proposed Condition A states that if the required ECCS injection/spray subsystem is inoperable, it is to be restored to operable status within 4 hours. Proposed Condition B states that if Condition A is not met, a method of water injection capable of operating without offsite electrical power should be established immediately. The proposed Condition B provides adequate assurance of an available water source should Condition A not be met within the 4-hour completion time.

35

36 The proposed Condition C states that for a drain time < 36 hours and \geq 8 hours, to (1) verify 37 [secondary containment] boundary is capable of being established in less than 4 hours, and 38 (2) verify each [secondary containment] penetration flow path is capable of being isolated in 39 less than 4 hours, and f(3) verify one standby gas treatment subsystem is capable of being 40 placed in operation in less than 4 hours]. The proposed Condition C provides adequate 41 protection should the DRAIN TIME be < 36 hours and \geq 8 hours because of the ability to 42 establish/secondary containment, isolate additional flow paths, fand have the standby gas 43 treatment subsystem operable].

44

45 The proposed Condition D states that when drain time < 8 hours to (1) immediately initiate

- 46 action to establish an additional method of water injection with water sources capable of
- 47 maintaining RPV water level > TAF for \geq 36 hours, (2) immediately initiate action to establish
- 48 [secondary] containment] boundary, (3) immediately initiate action to isolate each
- 49 [secondary] containment] penetration flow path or verify it can be manually isolated from the

control room, and (4) *[immediately initiate action to verify one standby gas treatment subsystem]* 1 2 is capable of being placed in operation. Additionally, there is a note stating that required ECCS injection/spray subsystem or additional method of water injection shall be capable of operating 3 without offsite electrical power, which is similar to proposed Condition B. The current TS for 4 5 Condition D are similar to the proposed for when Required Action C.2 is not met. The proposed 6 Condition D provides adequate protection should the DRAIN TIME be < 8 hours because of the 7 ability to establish [secondary containment], isolate additional flow paths, and [have the standby 8 gas treatment subsystem operable.]

9

10 The proposed Condition E states that when the required action and associated completion time 11 of Condition C or D is not met, or the drain time is < 1 hour, then initiate action to restore drain 12 time to \ge 36 hours immediately. The proposed Condition E is new, as it is not present in the 13 current TS. The proposed Condition E is acceptable as it provides the necessary step to 14 restore the drain time to \ge 36 hours should the other conditions not be met, or if the drain time is 15 < 1 hour.

17 Based on the NRC staff's review, the proposed changes to TS 3.5.2 are acceptable based on 18 the actions taken to mitigate the water level reaching the TAF with the water sources available 19 and maintaining drain time \geq 36 hours. The LCO correctly specifies the lowest functional 20 capability or performance levels of equipment required for safe operation of the facility. There is 21 reasonable assurance that the required actions to be taken when the LCO is not met can be 22 conducted without endangering the health and safety of the public

23

16

The existing TS 3.3.5.2, "RCIC System Instrumentation," is renumbered as TS 3.3.5.3. This
increases consistency within the TS as the Reactor Core Isolation Cooling (RCIC) System is
discussed in the section on TS 3.5.3. NOTE: Some licensees may choose to assign a different
number to this new TS. This is an acceptable alternative.

28 29 30

31

3.4 <u>TS 3.3.5.2, REACTOR PRESSURE VESSEL WATER INVENTORY CONTROL</u> INSTRUMENTATION

The existing TS 3.3.5.2, "RCIC System Instrumentation," is renumbered as TS 3.3.5.3. This increases consistency within the TS as the Reactor Core Isolation Cooling (RCIC) System is discussed in the section on TS 3.5.3. NOTE: Some licensees may choose to assign a different number to this new TS. This is an acceptable alternative.

36

37 The purpose of the RPV Water Inventory Control Instrumentation is to support the requirements 38 of new TS LCO 3.5.2, and the definition of drain time. There are instrumentation and controls 39 that are required for manual initiation or required as a permissive or operational controls on the 40 equipment of the systems that provide water injection capability, certain start commands, and 41 isolation functions. These instruments are required to be operable if the systems that provide 42 water injection and isolation functions are to be considered operable as described in the safety 43 evaluation of new TS 3.5.2. In some cases the reactor operators have alternate, often more 44 complex means, of starting and injecting water than the preferred simple push button start. 45

46 For BWR/2, /3, or /4 plants choose:

47 [Specifically, the RPV Water Inventory Control Instrumentation supports operation of the

48 Core Spray and LPCI including manual initiation when needed as well as the system

isolation of the RHR system and the RWCU system. The equipment involved with each 1 2 of these systems is described in the evaluation of TS 3.5.2 and the Bases for LCO 3.5.2.] 3 4 For BWR/5 or /6 plants choose: [Specifically, the RPV Water Inventory Control Instrumentation supports operation of the 5 6 LPCI with subsystems LPCI A, LPCI B, and LPCI C, LPCS, and HPCS, including manual 7 initiation when needed as well as the system isolation of the RHR system and the RWCU 8 system. The equipment involved with each of these systems is described in the 9 evaluation of TS 3.5.2 and the Bases for LCO 3.5.2.1 10 11 3.4.1 Proposed TS 3.3.5.2 LCO and Applicability 12 13 The proposed LCO 3.3.5.2 states, "The RPV Water Inventory Control instrumentation for each Function in Table 3.3.5.2-1 shall be OPERABLE." 14 15 16 The applicability states, "According to Table 3.3.5.2-1." 17 18 Section 3.3.1 of TSTF-542, states: 19 20 21 Table 3.3.5.2-1 contains those instrumentation Functions needed 22 to support manual initiation of the ECCS injection/spray subsystem required by LCO 3.5.2, and automatic isolation of 23 penetration flow paths that may be credited in a calculation of 24 25 Drain Time. The Functions in Table 3.3.5.2-1 are moved from 26 existing TS 3.3.5.1, "ECCS Instrumentation," and TS 3.3.6.1, "Primary Containment Isolation Instrumentation" Functions that 27 are required in Modes 4 or 5 or during OPDRVs. Creation of 28 29 TS 3.3.5.2 places these Functions in a single location with 30 requirements appropriate to support the safety function for 31 TS 3.5.2. 32 33 If plant-specific design and TS require different functions to 34 support manual initiation of an ECCS subsystem, those functions 35 should be included in TS 3.3.5.2. 36 37 38 3.4.2 Proposed TS 3.3.5.2 Actions 39 TS 3.3.5.2 contains actions to be followed when the LCO is not met. 40 41 42 TSTF-542, Section 3.3.2, "Proposed TS 3.3.5.2 Actions," discusses the actions of TS 3.3.5.2 43 and LCO 3.3.5.2. The NRC staff finds these actions are sufficient and necessary, because 44 when one or more instrument channels are inoperable the equipment and function controlled by 45 these instruments cannot complete the required function in the normal manner and these 46 actions direct the licensee to take appropriate actions as necessary and enter immediately into 47 the Conditions referenced in Table 3.3.5.2-1. These actions satisfy the requirements of 10 CFR

 $48 \quad 50.36(c)(\frac{32}{i})$ by providing a remedial action permitted by the TS until the LCO can be met.

- 24 -

1 The remedial actions provide reasonable assurance that an unexpected draining event can be 2 prevented or mitigated before the RPV water level would be lowered to the TAF.

4 For BWR/2, /3, or /4 plants choose the following Section 3.4.3:

- 5 3.4.3 Proposed TS 3.3.5.2 Actions
- 6

3

TSTF-542, Section 3.3.2, "Proposed TS 3.3.5.2 Actions," discusses the actions of TS 3.3.5.2
and LCO 3.3.5.2. The NRC staff finds these actions are sufficient and necessary, because
when one or more instrument channels are inoperable the equipment and function controlled by
these instruments cannot complete the required function in the normal way, and these actions
direct the licensee to take appropriate actions as required. The actions provide reasonable
assurance that an unexpected draining event can be prevented or mitigated before the RPV
water level would be lowered to the TAF.

- 14
- 15 Action A is applicable when one or more instrument channels are inoperable from
- 16 Table 3.3.5.2-1 and directs the licensee to immediately enter the Condition referenced in
- 17 Table 3.3.5.2-1 for that channel.
- 18

Action B (concerning the RHR system Isolation and RWCU system Isolation functions) are applicable when automatic isolation of the associated penetration flow path is credited as not having to be considered as a path for potential drainage in calculating drain time. If the instrumentation is inoperable, Required Action B.1 directs an immediate declaration that the associated penetration flow path(s) are incapable of automatic isolation. Required Action B.2 requires a re-calculation of drain time, but automatic isolation of the affected penetration flow paths cannot be credited.

26

27 Action C (concerning low reactor steam dome pressure permissive Functions necessary for 28 ECCS subsystem manual initiation) addresses an event in which the permissive is inoperable 29 and manual initiation of ECCS using the control board pushbuttons is prevented. The function 30 must be placed in the trip condition within one hour. With the permissive function instrument in 31 the trip condition, manual initiation may now be performed using the preferred control board 32 pushbuttons. This one-hour completion time is acceptable, because despite the preferred start 33 method being prevented, the reactor operator can take manual control of the pump and the 34 injection valve to inject water into the RPV and achieve the safety function. The time of one 35 hour also provides reasonable time for evaluation and placing the channel in trip. 36

37 Action D (concerning pump discharge flow bypass Functions and the manual initiation 38 Functions) addresses actions when the bypass is inoperable and then there is a risk that the 39 associated ECCS pump could overheat when the pump is operating and the associated 40 injection valve is not fully open. In this condition, the operator can take manual control of the 41 pump and the injection. Similar to justification in Action C, while this is not the preferred 42 method, if a manual initiation function is inoperable, the ECCS subsystem pumps can be started manually and the valves can be opened manually. The 24-hour completion time is acceptable, 43 44 because the functions can be performed manually and it allows time for the operator to evaluate 45 and have necessary repairs completed. Unlike the failure of a pushbutton that may concern 46 electronic component repairs, mechanical components may be involved in repairs, testing, and 47 return to service of pumps and valves. This further justifies a 24-hour completion time as 48 appropriate.

49

Action E is needed and becomes necessary, if the required action and associated completion
 time of Condition C or D, are not met. If they are not met, then the associated low pressure
 ECCS injection/spray subsystem may be incapable of performing the intended function, and the
 ECCS subsystem must be declared inoperable immediately.

- 5
- 6 For BWR/5 or /6 plants choose the following Section 3.43:
 7 3.4.3 Proposed TS 3.3.5.2 Actions
 - 7 8

9 TSTF-542, Section 3.3.2, "Proposed TS 3.3.5.2 Actions," discusses the actions of TS 3.3.5.2 and LCO 3.3.5.2. The NRC staff finds these actions are sufficient and necessary, because when one or more instrument channels are inoperable the equipment and function controlled by these instruments cannot complete the required function in the normal way and these actions direct the licensee to take appropriate actions as required. The remedial actions provide reasonable assurance that an unexpected draining event can be prevented or mitigated before the RPV water level would be lowered to the TAF.

16

Action A is applicable when one or more instrument channels are inoperable from Table 3.3.5.2-*1* and directs the licensee to immediately enter the condition referenced in Table 3.3.5.2-1 for
that channel.

20

Action B (concerning the RHR system isolation and RWCU system isolation functions) *is* are applicable when automatic isolation of the associated penetration flow path is credited as not having to be considered as a path for potential drainage in calculating drain time. If the instrumentation is inoperable, Required Action B.1 directs an immediate declaration that the associated penetration flow path(s) are incapable of automatic isolation. Required Action B.2 requires a re-calculation of drain time, but automatic isolation of the affected penetration flow paths cannot be credited.

28

Action C (concerning low reactor steam dome pressure permissive Functions necessary for ECCS subsystem manual initiation) addresses an event in which the permissive is inoperable

ECCS subsystem manual initiation) addresses an event in which the permissive is inoperable
 and manual initiation of ECCS using the control board pushbuttons is prevented. The function
 must be placed in the trip condition within one hour. With the permissive function instrument in

- the trip condition, manual initiation may now be performed using the preferred control board
- pushbuttons. This one hour completion time is acceptable, because despite the preferred start
 method being prevented, the reactor operator can take manual control of the pump and the
- 36 injection valve to inject water into the RPV and achieve the safety function. The time of one
- 37 hour also provides reasonable time for evaluation and placing the channel in trip.
- 38

39 Action D (concerning loss of adequate water supply for the HPCS System), addresses an event 40 in which there is an inadequate water supply. The instrumentation functions have the ability to 41 detect low-water setpoint in the Condensate Storage Tank and actuate valves to realign HPCS 42 suction water source to the Suppression Pool. The Condensate Storage Tank Level - Low 43 Function indicates multiple, inoperable channels within the same Function resulting in a loss of 44 the automatic ability to swap suction to the Suppression Pool. The HPCS system must be 45 declared inoperable within one hour or the HPCS pump suction must be realigned to the 46 Suppression Pool, since, if realigned, the Function is already performed. This one hour is 47 acceptable, because it provides sufficient time to take the action in order to minimize the risk of 48 HPCS being needed without an adequate water source by allowing time for restoration or 49 alignment of the HPCS pump suction to the suppression pool.

1

2 Action E (concerning HPCS high water level Function in the RPV) addresses actions when this 3 instrument function is inoperable. HPCS Reactor Vessel Water Level - High, Level 8 function 4 ensures that appropriate actions are taken if the HPCS Reactor Vessel Water Level - High, 5 Level 8 Function is inoperable. If the inoperability results in the channel being tripped, the 6 HPCS pump discharge valve will not open and HPCS injection is prevented. In that case the 7 HPCS System must be declared inoperable within one hour, and the function must be restored 8 to operable status within 24 hours. The one hour completion time is acceptable, because of the 9 ability to manually start the HPCS pumps and open the discharge valve. The 24-hour 10 completion time is acceptable, because it allows time for the operator to evaluate and arrange 11 for repairs. 12

13 Action F (concerning pump discharge flow bypass Functions and the manual initiation 14 Functions) addresses an event in which the bypass is inoperable and there is a risk that the 15 associated ECCS pump could overheat when the pump is operating and the associated 16 injection value is not fully open. In this condition, the operator can take manual control of the 17 pump and the injection. Similar to justification in Action C, while this is not the preferred 18 method, if a manual initiation function is inoperable, the ECCS subsystem pumps can be started 19 manually and the valves can be opened manually. The 24-hour completion time is acceptable, 20 because the functions can be performed manually and it allows time for the operator to evaluate 21 and have necessary repairs completed. Unlike the failure of a pushbutton that may concern 22 electronic component repairs, mechanical components may be involved in repairs, testing, and 23 return to service of pumps and valves further justifying a 24-hour completion time as 24 appropriate. 25

- 26 Action G is needed and becomes necessary, if the required action and associated completion 27 time of Condition C, D, E, or F are not met. If they are not met, then the associated low 28 pressure ECCS injection/spray subsystem may be incapable of performing the intended 29 function, and the ECCS subsystem must be declared inoperable immediately.
- 30

31 3.4.4 Proposed TS 3.3.5.2 Surveillances

32 33 The TS 3.3.5.2 SR include Channel Checks, Channel Functional Tests, and Logic System 34 Functional Tests. There are three SRs numbered SR 3.3.5.2.1, SR 3.3.5.2.2, and SR 3.3.5.2.3. 35 The NRC staff finds these tests are sufficient and adequate, because they are essential to 36 ensure the Functions of TS 3.3.5.2 are operable (i.e., capable of performing the specified safety function in support of TS 3.5.2, Drain Time, and the protection from a potential drain down of the 37 38 RPV in Modes 4 and 5). The NRC staff finds the proposed TS 3.3.5.2 surveillances of LCO 39 3.5.2 as described in Section 3.3.3 satisfies 10 CFR 50.36(c)(3) by providing the specific SRs 40 relating to test, calibration, or inspection to assure that the necessary quality of systems and 41 components is maintained.

42

43 SR 3.3.5.2.1 requires a Channel Check and is applied to all functions except manual initiation.

44 Performance of the Channel Check ensures that a gross failure of instrumentation has not

45 occurred. A Channel Check is normally a comparison of the parameter indicated on one

channel to a similar parameter on other related channels. A Channel Check is significant in 46

47 assuring that there is a low probability of an undetected complete channel failure and is a key

48 safety practice to verifying the instrumentation continues to operate properly between each Surveillance Frequency Control Program], is consistent with the existing requirements and supports operating shift situational awareness.

3

4 SR 3.3.5.2.2 requires a Channel Functional Test and is applied to all functions except manual 5 initiation. A Channel Functional Test is the injection of a simulated or actual signal into the 6 channel as close to the sensor as practicable to verify operability of all devices in the channel 7 required for channel operability. It is performed on each required channel to ensure that the 8 entire channel will perform the intended function. The frequency is **[in accordance with the** 9 Surveillance Frequency Control Program or 92 days]. The applicant states, "This is 10 acceptable because it is consistent with the existing requirements for these Functions and is 11 based upon operating experience that demonstrates channel failure is rare." Since periods in 12 MODEs 4 and 5 as refueling outages are often in the order of 30 days or less, licensees could 13 include this SR, if desired, as part of a refueling activity. 14 15 SR 3.3.5.2.3 requires a Logic System Functional Test and is only applied to the manual initiation 16 functions. The Logic System Functional Test is a test of all logic components required for 17 operability of a logic circuit, from as close to the sensor as practicable up to, but not including, 18 the actuated device, and demonstrates the operability of the required manual initiation logic for 19 a specific channel. The ECCS subsystem functional testing performed in proposed SR 3.5.2.7 20 overlaps this surveillance to complete testing of the assumed safety function. The TSTF-542, 21 Section 3.2.4.6, states: 22 23 24 The Frequency of [18] months, or in accordance with the 25 Surveillance Frequency Control Program, is consistent with the 26 existing requirements, and is based upon operating experience 27 that that has shown that these components usually pass the 28 Surveillance when performed at this Frequency. 29 30 31 There are no SRs included to verify or adjust the instrument setpoint derived from the allowable 32 value using a Channel Calibration or a surveillance to calibrate the trip unit. TSTF-542, 33 Section 3.3.3, states, 34 35 36 A draining event in Mode 4 or 5 is not an analyzed accident and, 37 therefore, there is no accident analysis on which to base the 38 calculation of a setpoint. The purpose of the Functions is to allow 39 ECCS manual initiation or to automatically isolate a penetration 40 flow path, but no specific RPV water level is assumed for those 41 actions. Therefore, the Mode 3 Allowable Value was chosen for use in Modes 4 and 5 as it will perform the desired function. 42 43 Calibrating the Functions in Modes 4 and 5 is not necessary, as 44 TS 3.3.5.1 and TS 3.3.6.1 continue to require the Functions to be 45 calibrated on an [18] month Frequency. 46 47 And: 48

A draining event in Mode 4 or 5 is not an analyzed accident and, 1 2 therefore, there are no accident analysis assumptions on 3 response time. 4 5 6 This is acceptable, because this is adequate to ensure the channel responds with the required 7 pumping systems to inject water when needed and isolation equipment to perform when 8 commanded. 9 10 ECCS Response Time and Isolation System Response Time testing ensures that the individual 11 channel response times are less than or equal to the maximum values assumed in the accident 12 analysis. TS 3.3.5.2 does not include SRs to participate in any ECCS Response Time testing 13 and Isolation System Response Time testing. This is acceptable because the purpose of these 14 tests are to ensure that the individual channel response times are less than or equal to the 15 maximum values assumed in the accident analysis, but a draining event in Mode 4 or 5 is not an 16 analyzed accident and, therefore, there are no accident analysis assumptions on response time 17 and there are alternate manual methods for achieving the safety function. A potential draining 18 event in MODEs 4 and 5 is a slower event than a LOCA. More significant protective actions are 19 required as the calculated drain time decreases. 20 21 3.4.5 Conclusion of NRC Staff Review of TS 3.3.5.2 22 23 The NRC staff finds that proposed TS 3.3.5.2 and LCO 3.3.5.2 satisfies Criterion 4 of 24 10 CFR 50.36(c)(3), because specific instrumentation is provided that helps prevent or mitigate 25 a potential RPV drain down event. Operating experience highlights that RPV draining events 26 are potentially significant to public health and safety, as established in the following NRC 27 documents: 28 29 1. Information Notice 84-81 "Inadvertent Reduction in Primary Coolant Inventory in 30 Boiling Water Reactors During Shutdown and Startup," November 1984. 31 2. Information Notice 86-74, "Reduction of Reactor Coolant Inventory Because of 32 Misalignment of RHR Valves," August 1986. 33 3. Generic Letter 92-04, "Resolution of the Issues Related to Reactor Vessel Water 34 Level Instrumentation in BWRs Pursuant to 10 CFR 50.54(f)," August 1992. 35 4. NRC Bulletin 93-03, "Resolution of Issues Related to Reactor Vessel Water Level 36 draining event in Mode 4 Instrumentation in BWRs," May 1993. 37 38 The NRC staff finds that proposed LCO 3.3.5.2 correctly specifies the lowest functional 39 capability or performance levels of equipment required for safe operation of the facility. There is 40 reasonable assurance that the required actions to be taken when the LCO is not met can be 41 conducted without endangering the health and safety of the public. 42 43 3.5 TABLE 3.3.5.2-1, "RPV WATER INVENTORY CONTROL INSTRUMENTATION" 44 45 In order to support the requirements of TS 3.5.2, and LCO 3.5.2, "Reactor Pressure Vessel 46 (RPV) Water Inventory Control," and the definition of "DRAIN TIME"; the instrumentation 47 requirements are designated in Table 3.3.5.2-1. These instruments are required to be operable 48 if the systems that provide water injection and isolation functions are to be considered operable 49 as described in the NRC staff's evaluation of TS 3.5.2.

1

Table 3.3.5.2-1 specifies the instrumentation that shall be operable for each function in the table
for Modes 4 and 5 (or other specified conditions), the required number of channels per function,
conditions referenced from required action A.1, SR for the functions, [the allowable value
(removed this if version B)], and footnotes concerning items of the table.

6

Note: Table 3.3.5.2-1 version A has a column for the allowable value. Version A has the
 allowable value in brackets. The brackets indicate that a plant-specific value should be used in

9 the LAR to adopt TSTF-542. Table 3.3.5.2-1 version B does not have a column for the

- 10 allowable value.
- 11

12 Proposed Table TS 3.3.5.2-1, 'RPV Water Inventory Control Instrumentation,' presents details 13 on the functions required to support the equipment and functions of TS 3.5.2. The NRC staff 14 finds the presentation in this table acceptable, because this section sufficiently discusses the 15 purpose of the functions, the applicability, the number of required channels, the references to 16 the Condition to be entered by letter (e.g., A, B, C) if the function is inoperable, the applicable 17 SRs, the selection of the allowable value, and justification of differences between the existing 18 and proposed TS functions. This RPV Water Inventory Control Instrumentation set is 19 acceptable, because it is adequate to ensure the instruments of the channels responds with the 20 required accuracy permitting pumps systems to operate to inject water when needed and 21 isolation of equipment when commanded to support the prevention of or mitigate a potential RPV draining event.

22 23

24 Each of the ECCS subsystems in MODEs 4 and 5 are initiated by manual pushbutton. 25 Automatic initiation of an ECCS injection/spray subsystem may be undesirable because it could 26 lead to overflowing the RPV cavity, due to injection rates of thousands of gallons per minute. 27 Thus, there is adequate time to take manual actions (e.g., hours versus minutes). Considering 28 the action statements as the drain time decreases (the proposed TS 3.5.2, Action E, prohibits 29 plant conditions that could result in drain times less than one hour), therefore, there is sufficient 30 time for the reactor operators to take manual action to stop the draining event, and to manually 31 start an ECCS injection/spray subsystem or the additional method of water injection as needed. 32 Consequently, there is no need for automatic initiation of ECCS to respond to an unexpected 33 draining event. This is acceptable, because a draining event is a slow evolution when 34 compared to a design basis LOCA assumed to occur at a significant power level.

35

36 For BWR/2, /3, or /4 plant, choose the following Section 3.5.1:

- 37 3.5.1 Proposed Table 3.3.5.2-1, Functions
- 38

39 For the Table 3.3.5.2-1 Functions 1.a and 2.a, BWR/4 CS and LPCI Systems, Reactor Steam 40 Dome Pressure - Low (Injection Permissive), these signals are used as permissives and 41 protection for these low pressure ECCS injection/spray subsystem manual initiation functions. 42 This function ensures that the reactor pressure has fallen to a value below these subsystems' 43 maximum design pressure before permitting the operator to open the injection valves of the low 44 pressure ECCS subsystems. Even though during MODEs 4 and 5 the reactor steam dome 45 pressure is expected to virtually always be below the ECCS maximum design pumping 46 pressure, the Reactor Steam Dome Pressure - Low signals are required to be operable and 47 capable of permitting initiation of the ECCS.

48

Low (Bypass), these minimum flow instruments are provided to protect the associated low pressure ECCS pumps from overheating when the pump is operating and the associated injection valve is not fully open. The minimum flow line valve is opened when low flow is sensed, and the valve is automatically closed when the flow rate is adequate to protect the pump. *Use for Version "A" TS:* [Where applicable, allowable values specified are high enough to ensure that the pump flow rate is sufficient to protect the pump, yet low enough to ensure that the closure of the minimum flow valve is initiated to allow full flow into the core.] The LPCI

8 the closure of the minimum flow valve is initiated to allow full flow into the core.] The LPCI
9 minimum flow valves are time delayed such that the valves will not open for 10 seconds after
10 the switches detect low flow. This time delay is acceptable, because it is provided to limit

11 reactor vessel inventory loss during the startup of the RHR shutdown cooling mode.

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For the Table 3.3.5.2-1 Functions 1.c and 2.c, CS System Manual Initiation and LPCI, System
 Manual Initiation, the manual initiation pushbutton channels introduce signals into the

15 appropriate ECCS logic to provide manual initiation capability. There is one push button for

16 each of the CS and LPCI subsystems (i.e., two for CS and two for LPCI). There is no allowable

17 value for this Function since the channels are mechanically actuated based solely on the

18 position of the push buttons. An instrument channel of the Manual Initiation Function (one

19 channel per subsystem) is required to be Operable in MODEs 4 and 5 when the associated

20 ECCS subsystems are required to be Operable per LCO 3.5.2.

21

22 For the Table 3.3.5.2-1 Function 3.a, RHR System Isolation, Reactor Vessel Water Level - Low, 23 Level 3, the function is only required to be operable when automatic isolation of the associated 24 penetration flow path is credited in the drain time calculation. The number of required 25 instrument channels is [2 in one trip system], which retains the requirement that the two 26 instrument channels must be associated with the same trip system. Each trip system isolates 27 one of two redundant isolation valves, and only one trip system is required to be operable to 28 ensure that automatic isolation of one of the two isolation valves will occur on low reactor vessel 29 water level indication. Use for Version "A" TS: [The allowable value was chosen to be the 30 same as the Primary Containment Isolation Instrumentation Reactor Vessel Water Level - Low, 31 Level 3 Allowable Value from LCO 3.3.6.1.]

32

33 For the Table 3.3.5.2-1 Function 4.a, RWCU, System Isolation, Reactor Vessel Water Level -34 Low Low, Level 2, the function is only required to be operable when automatic isolation of the 35 associated penetration flow path is credited in the drain time calculation. The number of 36 required channels is [2 in one trip system], which retains the requirement that the two 37 instrument channels must be associated with the same trip system. Only one trip system is 38 required to be operable to ensure that automatic isolation of one of the two isolation valves will 39 occur on low reactor vessel water level. Use for Version "A" TS: [Allowable value was chosen to be the same as the ECCS Reactor Vessel Water Level - Low Low, Level 2 Allowable Value 40 41 from LCO 3.3.5.1.

42

43 For BWR/5 or /6 plant, choose the following Section 3.5.2:

44 3.5.2 Proposed Table 3.3.5.2.-1, Functions

45

For the Table 3.3.5.2-1 Functions 1.a and 2.a, LPCS and LPCI Systems, Reactor Steam Dome
 Pressure - Low (Injection Permissive), these signals are used as permissives and protection for

47 Pressure - Low (injection remissive), these signals are used as permissives and protection for
 48 these low pressure ECCS injection/spray subsystem manual initiation functions. This function

49 ensures that the reactor pressure has fallen to a value below these subsystems' maximum

For the Table 3.3.5.2-1 Functions 1.b and 2.b, CS and LPCI Systems, Pump Discharge Flow -

design pressure before permitting the operator from opening the injection valves of the low pressure ECCS subsystems. Even though during MODEs 4 and 5 the reactor steam dome

pressure is expected to virtually always be below the ECCS maximum design pumping
 pressure, the Reactor Steam Dome Pressure - Low signals are required to be operable and

5 capable of permitting initiation of the ECCS.

6

7 For the Table 3.3.5.2-1 Functions 1.b, 1.c, and 2.b, LPCS and LPCI Systems Low Pressure 8 Coolant Injection and Low Pressure Core Spray Pump Discharge Flow - Low (Bypass), these 9 instruments are provided to protect the associated low pressure ECCS pump from overheating 10 when the pump is operating and the associated injection valve is not fully open. The minimum 11 flow line valve is opened when low flow is sensed, and the valve is automatically closed when 12 the flow rate is adequate to protect the pump. Use for Version "A" TS: [Where applicable 13 allowable values specified are high enough to ensure that the pump flow rate is sufficient to 14 protect the pump, yet low enough to ensure that the closure of the minimum flow valve is

15 initiated to allow full flow into the core.

16

17 For the Table 3.3.5.2-1 Functions 1.d and 2.c, LPCS and LPCI Systems, Manual Initiation, the 18 manual initiation pushbutton channels introduce signals into the appropriate ECCS logic to 19 provide manual initiation capability. There is one pushbutton for each subsystem in the two 20 divisions of low pressure ECCS (i.e., Division 1 ECCS, LPCS and LPCI A; Division 2 ECCS, 21 LPCI B and LPCI C). There are four subsystems, thus four pushbuttons for the low pressure 22 ECCS. The only manual initiation function required to be operable is that associated with the ECCS subsystem that is required to be operable by LCO 3.5.2. Use for Version "A" TS: Since 23 24 the channels are mechanically actuated based solely on the position of the pushbuttons, there is 25 no allowable value for this function.] When this instrument function is inoperable, manual 26 initiation with the control board push buttons is inoperable. However, the ECCS pumps can be 27 started manually and valves can be opened manually by the reactor operator. This is not the 28 preferred condition.

29

For the Table 3.3.5.2-1 Functions 3.a, HPCS System Reactor Vessel Water Level - High, Level 8, the High RPV water level, Level 8 signal, is used to close the HPCS injection valve to prevent overflow into the main steam lines (MSLs). One instrument channel associated with the HPCS system is required to be operable to support LCO 3.5.2. *Use for Version "A" TS:* [The LCO 3.3.5.2 allowable value is chosen to isolate flow from the HPCS system prior to water overflowing into the MSLs.]

35 36

37 For the Table 3.3.5.2-1 Functions 3.b, HPCS System, Condensate Storage Tank Level – Low, 38 the low level signal in the Condensate Storage Tank (CST) indicates the lack of an adequate 39 supply of makeup water from this primary source for HPCS. Normally, the water source for the 40 suction for HPCS is the CST. If the water level in the CST falls below a preselected level, 41 instrumentation logic controls valves so suction is then pulled from the Suppression Pool. First 42 the Suppression Pool suction valve is automatically opened and then the CST suction valve is 43 automatically closed in a manner to ensure that an adequate supply of makeup water is 44 available to the HPCS pump. The Condensate Storage Tank Level - Low signals are initiated 45 from two level transmitters. The Condensate Storage Tank Level - Low Function Allowable 46 Value is high enough to ensure adequate pump suction head while water is being taken from 47 the CST.

48

For the Table 3.3.5.2-1 Functions 3.c and 3.d, HPCS System, HPCS Pump Discharge Pressure 1 2 - High (Bypass) and HPCS System Flow Rate - Low (Bypass), the minimum flow instruments 3 are provided to protect the HPCS pump from overheating when the pump is operating and the 4 associated injection value is not fully open. The minimum flow line value is opened when low 5 flow and high pump discharge pressure are sensed, and the valve is automatically closed when 6 the flow rate is adequate to protect the pump or the discharge pressure is low (indicating the 7 HPCS pump is not operating).

8

9 For the Table 3.3.5.2-1 Function 3.e, HPCS System, Manual Initiation, the Manual Initiation 10 push button channel introduces a signal into the HPCS logic to provide manual initiation 11 capability. There is one pushbutton for the HPCS system.

12

13 For the Table 3.3.5.2-1 Function 4.a, BWR/6 RHR System Isolation, Reactor Vessel Water

14 Level - Low, Level 3, the Function is only required to be operable when automatic isolation of

15 the associated RHR system penetration flow path is credited in calculating drain time. The

16 definition of drain time allows crediting the closing of penetration flow paths that are capable of

17 being automatically isolated by RPV water level isolation instrumentation prior to the RPV water

18 level dropping below the TAF, but if the instrument function is inoperable, a closed path cannot 19 be credited and a drain time calculation must be re-performed.

20

21 For the Table 3.3.5.2-1 Function 5.a, RWCU System Isolation, Reactor Vessel Water Level -22 Low Low, Level 2, the Function is only required to be Operable when automatic isolation of the 23 associated RWCU system penetration flow path is credited in calculating drain time. The 24 definition of drain time allows crediting the closing of penetration flow paths that are capable of 25 being automatically isolated by RPV water level isolation instrumentation prior to the RPV water 26 level dropping below the TAF, but if the instrument function is inoperable, a closed path cannot 27 be credited and a drain time calculation must be re-performed. This function is not applicable in 28 MODEs 4 or 5 in TS 3.3.6.1, but is being added to TS 3.3.5.2 to support crediting the automatic 29 isolation of the RWCU system in calculating drain time.

30

OTHER DIFFERENCES BETWEEN THE CURRENT AND PROPOSED TS 31 3.6 32 REQUIREMENTS 33

34 Sections [2.3.3.3 through 2.3.3.5] [NOTE: If there are licensee specific changes, adjust section 35 reference as needed] of this SE describe additional changes to the TSs in which references to 36 OPDRVs are deleted. The NRC staff has determined that deletion of these references is 37 appropriate because the specifications governing Reactor Pressure Vessel Water Inventory 38 Control and associated Instrumentation specifications provide an acceptable alternative set of 39 controls for ensuring water level is maintained above the TAF.

40

41 3.7 TS 3.5.2 – REACTOR PRESSURE VESSEL (RPV) WATER INVENTORY CONTROL 42 **REVIEW CONCLUSIONS**

43

44 Safety Limit 2.1.1.3 requires that reactor vessel water level shall be greater than the top of 45 active irradiated fuel. Maintaining water level above the TAF ensures that the fuel cladding 46 fission product barrier is protected during shutdown conditions. The changes to the TS 47 establish new LCO requirements that address the preventive and mitigative equipment and 48 associated instrumentation that provide an alternative means to support Safety Limit 2.1.1.3 49 during MODE 4 and 5 operations.

1

2 NOTE: NRC staff shall confirm statements in this paragraph are true for the plant. This 3 information should be available in the plant's LAR or FSAR.

3 4 LOCAs are postulated accidents that would result from the loss of reactor coolant, at a rate in 5 excess of the capability of the normal reactor coolant makeup system, from piping breaks in the 6 reactor coolant pressure boundary. During operation in Modes 4 and 5, the reactor coolant 7 system is at a low operating temperature (</200] degrees Fahrenheit) and is depressurized. An 8 event involving a loss of inventory while in the shutdown condition is judged to not exceed the 9 capacity of one ECCS subsystem. The accidents that are postulated to occur during shutdown 10 conditions, the Fuel Handling Accident and the Waste Gas Decay Tank Rupture, do not involve 11 a loss of inventory. The equipment and instrumentation associated with the Reactor Vessel 12 Water Inventory Control TS do not provide detection or mitigation related to these design basis 13 accidents. 14

The revised TS LCO 3.5.2 contains requirements for operability of one ECCS subsystem along with requirements to maintain a sufficiently long drain time that plant operators would have time to diagnose and mitigate an unplanned draining event. The NRC staff has determined that the LCO 3.5.2 and 3.3.5.2 provide for the lowest functional capability or performance levels of equipment required for safe operation of the facility, and therefore, meet the LCO requirements of 10 CFR 50.36(c)(2)(i).

21

Additionally, the revised TS LCOs 3.5.2 and 3.3.5.2 provide remedial actions to be taken in the event the LCO is not satisfied, therefore meeting the requirements of 10 CFR 50.36(c)(2)(i). The NRC staff has found that the remedial actions provide reasonable assurance that an unexpected draining event can be prevented or mitigated before the RPV water level would be lowered to the TAF.

27

28 The regulation at 10 CFR 50.36(c)(3) requires TSs to include items in the category of SRs,

which are requirements relating to test, calibration, or inspection to assure that the necessary

quality of systems and components is maintained, that facility operation will be within safety
limits, and that the LCOs will be met. The NRC staff reviewed the SRs associated with the new
LCOs 3.5.2 and 3.3.5.2. The NRC staff reviewed the revised SRs and determined that they are
appropriate for ensuring the operability of the equipment and instrumentation specified in LCOs
3.5.2. Therefore, the NRC staff concludes that 10 CFR 50.36(c)(3) is satisfied.

35

NOTE: NRC staff shall confirm that the licensee did provide TS Bases consistent with the STS
 Bases changes approved in TSTF-542 and that the any bracketed information in the STS Bases
 has been filled in with plant-specific information.

39 The regulation at 10 CFR 50.36(a)(1) states that a summary statement of the bases or reasons

40 for such specifications, other than those covering administrative controls, shall also be included

41 in the application, but shall not become part of the TSs. In accordance with the 10 CFR

42 50.36(a)(1) requirement, the licensee provided TS Bases changes in Attachment 4 of the

43 licensee's amendment request. The NRC staff has concluded that the TS Bases changes

44 provided describe the basis for the affected TS and follow the Final Policy Statement on

45 Technical Specifications Improvements for Nuclear Power Reactors (58 Federal Register46 39132).

47

48 The NRC staff's guidance for review of TSs is in Chapter 16, *Technical Specifications*, of

49 NUREG-0800, Revision 3, Standard Review Plan (March 2010) (ADAMS Accession

1 No. ML100351425). As described therein, as part of the regulatory standardization effort, the

NRC staff has prepared STS for each of the light–water reactor nuclear designs. [For [PLANT],
 the representative STS is in [NUREG-1433][NUREG-1434][for BWR/5: NUREG-1433 and

4 NUREG-1434]For BWR/4 plants: [NUREG-1433, Revision 4, contains the STS for BWR/4

plants.] For BWR/6 plants: [NUREG 1434, Revision 4, contains the STS for BWR/6 plants.]

6 The changes to the TS were reviewed for technical clarity and consistency with customary

7 terminology and format with the existing requirements. The staff found that the proposed

8 changes were consistent with TSTF-542 and [NUREG-1433] and/or [NUREG-1434].

9

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10 3.8 <u>TECHNICAL CONCLUSION</u>

The NRC staff evaluated the proposed changes to the TS of proposed drain time definition and TS 3.5.2 related to RPV WIC and TS 3.3.5.2 which contains the instrumentation necessary to support TS 3.5.2. Based on the considerations discussed above, the NRC staff concludes that the proposed revisions to the TS via adding a "DRAIN TIME" definition and TS 3.5.2 and TS 3.3.5.2 respectively are acceptable.

18 NOTE: Include other TS changes as necessary.19

20 4.0 STATE CONSULTATION

22 This section is to be prepared by the PM.

23 24 The requirements with respect to State consultation are contained in 10 CFR 50.91(b). 10 CFR 25 50.91(b)(3) and (b)(4) require that: (1) the NRC make a good faith effort to telephone the State 26 official, prior to amendment issuance, to determine if the State has any comments; and (2) 27 consider any comments of the State official. If there are State comments, they should be 28 addressed in this section. Comments received from members of the public should be addressed 29 within the technical evaluation section or in a separate section of the safety evaluation. See 30 ADAMS Accession No. ML102710156 (Safety Evaluation Section 5.0, "Public Comments") for 31 an example of a safety evaluation which addresses public comments. 32

In accordance with the Commission's regulations, the [Name of State] State official was notified
 of the proposed issuance of the amendment. The State official had [no] comments. [If
 comments were provided, they should be addressed here.]

37 5.0 ENVIRONMENTAL CONSIDERATION 38

39 This section is to be prepared by the PM in accordance with current procedures.

41 **6.0** <u>CONCLUSION</u> 42

43 This section is to be prepared by the PM.

The Commission has concluded, based on the considerations discussed above, that: (1) there

46 is reasonable assurance that the health and safety of the public will not be endangered by

- 47 operation in the proposed manner, (2) there is reasonable assurance that such activities will be
- 48 conducted in compliance with the Commission's regulations, and (3) the issuance of the

amendment will not be inimical to the common defense and security or to the health and safety 1 of the public.

7.0 REFERENCES

2 3 4 5 6 7 8 9 Optional section to be prepared by the PM and primary reviewers. If document is publicly available, the ADAMS Accession No. should be listed.

- Principal Contributor:
- 10
- 11 Date:

Attachment 3

Editorial Corrections to TSTF-542, Revision 2

Summary Table

Location	Comment
Justification Sections 3.3.4.7, 3.3.4.8,	The third bullet refers to Action E, but the correct
3.3.4.11, and 3.3.4.12.	reference is Action F.
Justification Section 3.3.4.8	The title only references function 2.c, but the
	discussion applies to functions 1.d and 2.c as
	pointed out in the draft SE.
Justification Sections 3.3.4.10 and 3.3.4.13	The last bullet refers to LCO 3.3.5.1A, but the
	correct reference is LCO 3.3.5.2A.
NUREG-1433, TS 3.3.5.2A and TS 3.3.5.2B	Function 2.b, added the opening parenthesis to
markup	the word "Bypass".
NUREG-1434, TS 3.3.5.2B markup	Function 2.c, changed number of channels from
	"11" to "1"
NUREG-1434, TS 3.3.5.2A Bases, page	The Bases for Required Action G.1 states
B 3.3.5.2A-10	"Conditions C, D, E, or f" "F" is capitalized.
NUREG-1433 and NUREG-1434, TS 3.5.2	Reference section, Reference 3, states
Bases	"10 CFR 50.54(F)". The "F" is made lower case.

Revised TSTF-542 Pages Showing Editorial Corrections

3.3.4.7. <u>1.b, 1.c, and 2.b, BWR/6 Low Pressure Core Spray and Low Pressure Coolant Injection</u> <u>Systems Low Pressure Coolant Injection and Low Pressure Core Spray Pump</u> <u>Discharge Flow - Low (Bypass)</u>

The minimum flow instruments are provided to protect the associated low pressure ECCS pump from overheating when the pump is operating and the associated injection valve is not fully open. The minimum flow line valve is opened when low flow is sensed, and the valve is automatically closed when the flow rate is adequate to protect the pump.

One flow transmitter per ECCS pump is used to detect the associated subsystems' flow rates. The logic is arranged such that each transmitter causes its associated minimum flow valve to open. The logic will close the minimum flow valve once the closure setpoint is exceeded. The LPCI minimum flow valves are time delayed such that the valves will not open for 10 seconds after the switches detect low flow. The time delay is provided to limit reactor vessel inventory loss during the startup of the RHR shutdown cooling mode (for RHR A and RHR B).

The Allowable Values specified in LCO 3.3.5.2A are high enough to ensure that the pump flow rate is sufficient to protect the pump, yet low enough to ensure that the closure of the minimum flow valve is initiated to allow full flow into the core.

One channel of the Pump Discharge Flow - Low Function is required to be operable in Modes 4 and 5 when the associated LPCS or LPCI pump is required to be operable by LCO 3.5.2 to ensure the pumps are capable of injecting into the Reactor Pressure Vessel when manually initiated.

These Functions were moved from TS 3.3.5.1, Function 1.e, 1.f, and 2.e. The following changes are made:

- The Applicability is changed. The TS 3.3.5.1 Applicability for these Functions is Modes 4 and 5 when the associated ECCS subsystem(s) are required to be operable per LCO 3.5.2, "ECCS Shutdown." The proposed Applicability is Modes 4 and 5 without exception, to be consistent with the Applicability of LCO 3.5.2, "RPV Water Inventory Control."
- The number of required channels per Function is changed from "[1]" to "[1 per pump]" and is modified by a note stating "Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory Control'." TS 3.5.2 only requires a single ECCS subsystem and the change in required channels reflects that requirement.
- The TS 3.3.5.1 Required Actions E.1 and E.2 for an inoperable channel is to declare the supported feature(s) inoperable within 1 hour of discovery when its redundant feature ECCS initiation capability is inoperable, and to restore the channel to operable status within 7 days. The proposed TS 3.3.5.2, Action *EF*, requires restoring the channel to operable status within 24 hours. No redundant ECCS subsystem is required by TS 3.5.2. When this Function is inoperable, the ECCS subsystem is capable of injecting but is vulnerable to a low flow condition if the injection valve is not open. This equipment

protective function can be performed by the operator by manually starting and stopping the pump and opening the injection valve. Therefore, allowing 24 hours to restore the equipment protective function is a reasonable period.

- A Channel Check and Channel Functional Test are required at the existing Frequency. Calibrating the trip unit, Channel Calibration and Logic System Functional Test are no longer required in Modes 4 and 5, as discussed in Section 3.3.3.
- In LCO 3.3.5.2A, the allowable value is unchanged.

3.3.4.8. <u>*1.d and* 2.c, BWR/6 Low Pressure Core Spray and Low Pressure Coolant Injection</u> Systems Manual Initiation

The Manual Initiation push button channels introduce signals into the appropriate ECCS logic to provide manual initiation capability. There is one push button for each of the two Divisions of low pressure ECCS (i.e., Division 1 ECCS, LPCS and LPCI A; Division 2 ECCS, LPCI B and LPCI C). The only the manual initiation function required to be OPERABLE is that associated with the ECCS subsystem required to be OPERABLE by LCO 3.5.2.

There is no Allowable Value for this Function in LCO 3.3.5.2A or LCO 3.3.5.2B since the channels are mechanically actuated based solely on the position of the push buttons.

These Functions were moved from TS 3.3.5.1, Function 1.g and Function 2.f. The following changes are made:

- The Applicability is changed. The TS 3.3.5.1 Applicability for these Functions in Modes 4 and 5 is modified by a Note that limits the Applicability to when the associated ECCS subsystem(s) are required to be operable per LCO 3.5.2, "ECCS Shutdown." The proposed Applicability is Modes 4 and 5 without exception, to be consistent with the Applicability of LCO 3.5.2, "RPV Water Inventory Control."
- The number of required channels per Function is changed from "[1]" to "[1 per subsystem]" and is modified by a note stating "Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory Control'." TS 3.5.2 only requires a single ECCS subsystem and the change in required channels reflects that requirement.
- The TS 3.3.5.1 Required Actions C.1 and C.2 for an inoperable channel is to declare the supported feature(s) inoperable within 1 hour of discovery when its redundant feature ECCS initiation capability is inoperable, and to restore the channel to operable status within 24 hours. The proposed TS 3.3.5.2, Action *EF*, requires restoring the channel to operable status within 24 hours. No redundant ECCS subsystem is required by TS 3.5.2. When this Function is inoperable, the ECCS pumps can be started manually and valves can be opened manually, but manual initiation with the control board push buttons is inoperable. This is not the preferred condition. Therefore, allowing 24 hours to restore the function to operable status is a reasonable period.

- A Channel Check and Channel Functional Test are required at the existing Frequency. Calibration of the trip units, Channel Calibration, and Logic System Functional Test are no longer required in Modes 4 and 5, as discussed in Section 3.3.3.
- The allowable value in LCO 3.3.5.1A-2A is unchanged.

3.3.4.11. <u>3.c and 3.d, BWR/6 High Pressure Core Spray System HPCS Pump Discharge</u> Pressure - High (Bypass) and HPCS System Flow Rate - Low (Bypass)

The minimum flow instruments are provided to protect the HPCS pump from overheating when the pump is operating and the associated injection valve is not fully open. The minimum flow line valve is opened when low flow and high pump discharge pressure are sensed, and the valve is automatically closed when the flow rate is adequate to protect the pump or the discharge pressure is low (indicating the HPCS pump is not operating).

One flow transmitter is used to detect the HPCS System's flow rate. The logic is arranged such that the transmitter causes the minimum flow valve to open, provided the HPCS pump discharge pressure, sensed by another transmitter, is high enough (indicating the pump is operating). The logic will close the minimum flow valve once the closure setpoint is exceeded. (The valve will also close upon HPCS pump discharge pressure decreasing below the setpoint.)

In LCO 3.3.5.2A, the HPCS System Flow Rate - Low and HPCS Pump Discharge Pressure - High Allowable Values are chosen to ensure that pump flow rate is sufficient to protect the pump, yet low enough to ensure that the closure of the minimum flow valve is initiated to allow full flow into the core. The HPCS Pump Discharge Pressure - High Allowable Value is set high enough to ensure that the valve will not be open when the pump is not operating.

These Functions were moved from TS 3.3.5.1, Function 3.f and 3.g. The following changes are made:

- The Applicability is changed. The TS 3.3.5.1 Applicability for this Function is Modes 4 and 5 when the associated ECCS subsystem(s) are required to be operable per LCO 3.5.2, "ECCS Shutdown." The proposed Applicability is Modes 4 and 5. The proposed Applicability is Modes 4 and 5 without exception, to be consistent with the Applicability of LCO 3.5.2, "RPV Water Inventory Control."
- The number of required channels per Function is changed from [1] to [1 per pump] and is modified by a note stating "Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory Control'." TS 3.5.2 only requires a single ECCS subsystem and the change in required channels reflects that requirement.
- The TS 3.3.5.1 Required Actions E1 and E.2 for an inoperable channel is to declare the supported feature(s) inoperable within 1 hour of discovery when its redundant feature ECCS initiation capability is inoperable, and to restore the channel to operable status within 7 days. The proposed TS 3.3.5.2, Action **E***F*, requires restoring the channel to operable status within 24 hours. No redundant ECCS subsystem is required by TS 3.5.2.

When this Function is inoperable, the ECCS subsystem is capable of injecting but is vulnerable to a low flow condition if the injection valve is not open. This equipment protective function can be performed by the operator by manually starting and stopping the pump and opening the injection valve. Therefore, allowing 24 hours to restore the equipment protective function is a reasonable period.

- A Channel Check and Channel Functional Test are required at the existing Frequency. Calibration of the trip units, Channel Calibration, and Logic System Functional Test are no longer required in Modes 4 and 5, as discussed in Section 3.3.3.
- The allowable value is unchanged.

3.3.4.12. 3.e, BWR/6 High Pressure Core Spray System Manual Initiation

The Manual Initiation push button channel introduces a signal into the HPCS logic to provide manual initiation capability. There is one push button for the HPCS System. One channel of the Manual Initiation Function is only required to be operable in Modes 4 and 5 when the associated ECCS subsystem is required to be operable per LCO 3.5.2.

There is no Allowable Value for this Function in LCO 3.3.5.2A or LCO 3.3.5.2B since the channel is mechanically actuated based solely on the position of the push button.

This Function is moved from TS 3.3.5.1, Function 3.h. The following changes are made:

- The Applicability is changed. The TS 3.3.5.1 Applicability for these Functions in Modes 4 and 5 is modified by a Note that limits the Applicability to when the associated ECCS subsystem(s) are required to be operable per LCO 3.5.2, "ECCS Shutdown." The proposed Applicability is Modes 4 and 5 without exception, to be consistent with the Applicability of LCO 3.5.2, "RPV Water Inventory Control."
- The number of required channels per Function is changed from [1] to [1 per subsystem] and is modified by a note stating "Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, 'Reactor Pressure Vessel Water Inventory Control'." TS 3.5.2 only requires a single ECCS subsystem and the change in required channels reflects that requirement.
- The TS 3.3.5.1 Required Action C.2 for an inoperable channel is to restore the channel to operable status within 24 hours. The proposed TS 3.3.5.2, Action *EF*, requires restoring the channel to operable status within 24 hours. When this Function is inoperable, the ECCS pumps can be started manually and valves can be opened manually, but manual initiation with the control board push buttons is inoperable. This is not the preferred condition. Therefore, allowing 24 hours to restore the Function to operable status is a reasonable period.
- Both the existing TS 3.3.5.1 and the proposed TS 3.3.5.2 require a Logic System Functional Test on this Function at the same Frequency.

affected penetration flow paths. The proposed Actions are consistent with the definition of Drain Time and the requirements of LCO 3.5.2.

- A Channel Check and Channel Functional Test are required at the existing Frequency. A calibration of the trip unit, Channel Calibration, Logic System Functional Test, and Isolation System Response Time tests are no longer required in Modes 4 and 5, as discussed in Section 3.3.3.
- The LCO 3.3.5.1A-2A allowable value is unchanged.

3.3.4.14. <u>5.a, BWR/6 Reactor Water Cleanup (RWCU) System Isolation, Reactor Vessel Water</u> Level - Low Low, Level 2

The definition of DRAIN TIME allows crediting the closing of penetration flow paths that are capable of being automatically isolated by RPV water level isolation instrumentation prior to the RPV water level being equal to the TAF. The Reactor Vessel Water Level - Low Low, Level 2 Function associated with RWCU System isolation may be credited for automatic isolation of penetration flow paths associated with the RWCU System.

This Function is not applicable in Modes 4 or 5 in TS 3.3.6.1, but is being added to TS 3.3.5.2 to support crediting the automatic isolation of the RWCU system in calculating Drain Time.

Reactor Vessel Water Level - Low Low, Level 2 is initiated from two channels per trip system that sense the difference between the pressure due to a constant column of water (reference leg) and the pressure due to the actual water level (variable leg) in the vessel. Each trip system isolates one of two redundant isolation valves and only one trip system is required to be operable when automatic isolation of the associated penetration flow path(s) is credited in calculating Drain Time to meet LCO 3.5.2.

The Reactor Vessel Water Level - Low Low, Level 2 Allowable Value was chosen to be the same as the ECCS Reactor Vessel Water Level - Low Low, Level 2 Allowable Value (LCO 3.3.5.1), since the capability to cool the fuel may be threatened.

The Reactor Vessel Water Level - Low Low, Level 2 Function is only required to be operable when automatic isolation of the associated penetration flow path is credited in calculating DRAIN TIME.

This Function was copied from TS 3.3.6.1, Function 4.k. The following changes are made:

- The Applicability is changed. The TS 3.3.6.1 Applicability for this Function is Modes 1, 2, and 3. The proposed Applicability is "when automatic isolation of the associated penetration flow path is credited in calculating Drain Time." In other words, if the Drain Time calculation assumes the RWCU system would be automatically isolated, this Function must be operable to perform that function. This is consistent with the definition of Drain Time and the TS 3.5.2 requirements.
- The number of required channels is changed from [2], with a column header that states "Required Channels per Trip System," to [2 in one trip system]. This retains the

Table 3.3.5.2-1 (page 1 of 1)
RPV Water Inventory Control Instrumentation

	FUNC	TION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER FUNCTION	CONDITIONS REFERENCED FROM REQUIRED ACTION A.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
1.	Core Spra	ay System					
	Pres (Injec	m Dome sure - Low	4, 5	[4]	С	SR 3.3.5.2.1 SR 3.3.5.2.2	≤ [500] psig
	Discl	Spray Pump narge Flow - (Bypass)	4, 5	[1 per pump(a)]	D	SR 3.3.5.2.1 SR 3.3.5.2.2	≥ [] gpm and ≤ [] gpm]
	c. Manı	ual Initiation	4, 5	[1 per subsystem (a)]	D	SR 3.3.5.2.3	NA
2.		sure Coolant tion (LPCI) em					
	Dom Low	ctor Steam e Pressure - (Injection nissive)	4, 5	[4]	С	SR 3.3.5.2.1 SR 3.3.5.2.2	≤ [500] psig
	Cool Pum	Pressure ant Injection p Discharge - Low <mark>(</mark> Bypass)	4, 5	[1 per pump(a)]	D	SR 3.3.5.2.1 SR 3.3.5.2.2	≥ [] gpm and ≤ [] gpm]
	c. Manı	ual Initiation	4, 5	[1](a)	D	SR 3.3.5.2.3	NA
3.	RHR Sys	tem Isolation					
		ctor Vessel er Level - Low, I 3	(b)	[2 in one trip system]	В	SR 3.3.5.2.1 SR 3.3.5.2.2	≥ [10] inches
4.	Reactor V (RWCU) Isolation	Vater Cleanup System					
	Wate	ctor Vessel er Level - Low Level 2	(b)	[2 in one trip system]	В	SR 3.3.5.2.1 SR 3.3.5.2.2	\geq [-47] inches

(a) Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, "Reactor Pressure Vessel Water Inventory Control."

(b) When automatic isolation of the associated penetration flow path(s) is credited in calculating DRAIN TIME.

Table 3.3.5.2-1 (page 1 of 1) RPV Water Inventory Control Instrumentation

		FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER FUNCTION	CONDITIONS REFERENCED FROM REQUIRED ACTION A.1	SURVEILLANCE REQUIREMENTS
1.	Со	re Spray System				
	a.	Reactor Steam Dome Pressure - Low (Injection Permissive)	4, 5	[4]	С	SR 3.3.5.2.1 SR 3.3.5.2.2
	b.	Core Spray Pump Discharge Flow - Low (Bypass)	4, 5	[1 per pump(a)]	D	SR 3.3.5.2.1 SR 3.3.5.2.2
	C.	Manual Initiation	4, 5	[1 per subsystem s(a)]	D	SR 3.3.5.2.3
2.	Lov	v Pressure Coolant Injection (LPCI) System				
	а.	Reactor Steam Dome Pressure - Low (Injection Permissive)	4, 5	[4]	С	SR 3.3.5.2.1 SR 3.3.5.2.2
	b.	Low Pressure Coolant Injection Pump Discharge Flow - Low <mark>(B</mark> ypass)	4, 5	[1 per pump(a)]	D	SR 3.3.5.2.1 SR 3.3.5.2.2
	C.	Manual Initiation	4, 5	[1](a)	D	SR 3.3.5.2.3
3.	RH	R System Isolation				
	a.	Reactor Vessel Water Level - Low, Level 3	(b)	[2 in one trip system]	В	SR 3.3.5.2.1 SR 3.3.5.2.2
4.	(R\	actor Water Cleanup NCU) System lation				
	a.	Reactor Vessel Water Level - Low Low, Level 2	(b)	[2 in one trip system]	В	SR 3.3.5.2.1 SR 3.3.5.2.2

(a) Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, "Reactor Pressure Vessel Water Inventory Control."

(b) When automatic isolation of the associated penetration flow path(s) is credited in calculating DRAIN TIME.

Table 3.3.5.2-1 (page 1 of 2) RPV Water Inventory Control Instrumentation

		FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER FUNCTION	CONDITIONS REFERENCED FROM REQUIRED ACTION A.1	SURVEILLANCE REQUIREMENTS
1.	Inje Lov Spi	w Pressure Coolant ection-A (LPCI) and w Pressure Core ray (LPCS) osystems				
	a.	Reactor Steam Dome Pressure - Low (Injection Permissive)	4, 5	[3(a)]	С	SR 3.3.5.2.1 SR 3.3.5.2.2
	b.	[LPCS Pump Discharge Flow - Low (Bypass)]	4, 5	[1 per pump(a)]	F	SR 3.3.5.2.1 SR 3.3.5.2.2
	C.	[LPCI Pump A Discharge Flow - Low (Bypass)]	4, 5	[1 per pump(a)]	F	SR 3.3.5.2.1 SR 3.3.5.2.2
	d.	Manual Initiation	4, 5	[1 per subsystem (a)]	F	SR 3.3.5.2.3
2.		CI B and LPCI C osystems				
	a.	Reactor Steam Dome Pressure - Low (Injection Permissive)	4, 5	[3(a)]	С	SR 3.3.5.2.1 SR 3.3.5.2.2
	b.	[LPCI Pump B and LPCI Pump C Discharge Flow - Low (Bypass)]	4, 5	[1 per pump(a)]	F	SR 3.3.5.2.1 SR 3.3.5.21.2
	C.	Manual Initiation	4, 5	[<mark>1</mark> per subsystem (a)]	F	SR 3.3.5.2.3

(a)] (a) Associated with an ECCS subsystem required to be OPERABLE by LCO 3.5.2, "Reactor Pressure Vessel Water Inventory Control."

BASES

ACTIONS (continued)

<u>F.1</u>

If an LPCI or LPCS Discharge Flow - Low bypass function or HPCS System Discharge Pressure - High or Flow Rate - Low bypass function is inoperable, there is a risk that the associated ECCS pump could overheat when the pump is operating and the associated injection valve is not fully open. In this condition, the operator can take manual control of the pump and the injection valve to ensure the pump does not overheat. If a manual initiation function is inoperable, the ECCS subsystem pumps can be started manually and the valves can be opened manually, but this is not the preferred condition.

The 24 hour Completion Time was chosen to allow time for the operator to evaluate and repair any discovered inoperabilities. The Completion Time is appropriate given the ability to manually start the ECCS pumps and open the injection valves and to manually ensure the pump does not overheat.

<u>G.1</u>

With the Required Action and associated Completion Time of Conditions C, D, E, or F not met, the associated ECCS injection/spray subsystem may be incapable of performing the intended function, and must be declared inoperable immediately.

SURVEILLANCE As noted in the beginning of the SRs, the SRs for each RPV Water REQUIREMENTS Inventory Control instrument Function are found in the SRs column of Table 3.3.5.2-1.

<u>SR 3.3.5.2.1</u>

Performance of the CHANNEL CHECK ensures that a gross failure of instrumentation has not occurred. A CHANNEL CHECK is normally a comparison of the parameter indicated on one channel to a similar parameter on other channels. It is based on the assumption that instrument channels monitoring the same parameter should read approximately the same value. Significant deviations between the instrument channels could be an indication of excessive instrument drift in one of the channels or something even more serious. A CHANNEL CHECK guarantees that undetected outright channel failure is limited; thus, it is key to verifying the instrumentation continues to operate properly between each CHANNEL FUNCTIONAL TEST.

BASES		
REFERENCES	1.	FSAR, Section [6.3.2]. Information Notice 84-81 "Inadvertent Reduction in Primary Coolant Inventory in Boiling Water Reactors During Shutdown and Startup," November 1984.
	2.	Information Notice 86-74, "Reduction of Reactor Coolant Inventory Because of Misalignment of RHR Valves," August 1986.
	3.	Generic Letter 92-04, "Resolution of the Issues Related to Reactor Vessel Water Level Instrumentation in BWRs Pursuant to 10 CFR 50.54(<mark>f),</mark> " August 1992.
	4.	NRC Bulletin 93-03, "Resolution of Issues Related to Reactor Vessel Water Level Instrumentation in BWRs," May 1993.
	5.	Information Notice 94-52, "Inadvertent Containment Spray and Reactor Vessel Draindown at Millstone 1," July 1994.
	6.	General Electric Service Information Letter No. 388, "RHR Valve Misalignment During Shutdown Cooling Operation for BWR 3/4/5/6," February 1983.

BASES		
REFERENCES	1.	FSAR, Section [6.3.3.4].Information Notice 84-81 "Inadvertent Reduction in Primary Coolant Inventory in Boiling Water Reactors During Shutdown and Startup," November 1984.
	2.	Information Notice 86-74, "Reduction of Reactor Coolant Inventory Because of Misalignment of RHR Valves," August 1986.
	3.	Generic Letter 92-04, "Resolution of the Issues Related to Reactor Vessel Water Level Instrumentation in BWRs Pursuant to 10 CFR 50.54 <mark>(f)</mark> , " August 1992.
	4.	NRC Bulletin 93-03, "Resolution of Issues Related to Reactor Vessel Water Level Instrumentation in BWRs," May 1993.
	5.	Information Notice 94-52, "Inadvertent Containment Spray and Reactor Vessel Draindown at Millstone 1," July 1994.
	6.	General Electric Service Information Letter No. 388, "RHR Valve Misalignment During Shutdown Cooling Operation for BWR 3/4/5/6," February 1983.