Charles R. Pierce
Regulatory Affairs Director

Southern Nuclear
Operating Company, Inc.
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Southern Nuclear

OCT 26 2016

Docket Nos.: 50-321
50-366

NL-16-2013

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555-0001

Edwin I. Hatch Nuclear Plant – Units 1 & 2
Transmittal of Revised Technical Specification Pages

Ladies and Gentlemen:

By letter dated March 14, 2016, and as supplemented by letter dated May 17, 2016, Southern Nuclear Operating Company (SNC) submitted a license amendment request (LAR) to adopt TSTF-65-A, Rev. 1. SNC would like to amend this request to include three administrative corrections to the Edwin I. Hatch Nuclear Plant (HNP) Technical Specifications (TSs).

1. Include a missing "(Continued)" at the bottom of Unit 1 page 3.4-20 to signify that the Surveillance Requirements for TS 3.4.9 continue onto the following page.
2. Add the term STAGGERED TEST BASIS back to the Unit 2 Definitions. This term was added to page 1.1-5 of the definitions section in Unit 2 Amendment No. 220. Due to an unrelated change to this same page, this term was inadvertently deleted with the issuance of Amendment 221.
3. Update Unit 1 and Unit 2 Surveillance Requirement (SR) 3.6.4.1.3 to incorporate approved changes in Unit 1 Amendments 280 and 279 and in Unit 2 Amendments 224 and 223. By letter dated September 29, 2016, the Nuclear Regulatory Commission (NRC) issued HNP Amendments 279 (Unit 1) and 223 (Unit 2). By letter dated September 30, 2016, the NRC issued HNP Amendments 280 (Unit 1) and 224 (Unit 2). These Unit 1 and Unit 2 Amendments both made unrelated changes to SR 3.6.4.1.3. The changes approved in Amendments 279 (Unit 1) and 223 (Unit 2), however, were not incorporated in the TS pages issued in Amendments 280 (Unit 1) and 224 (Unit 2).

All of these administrative corrections simply incorporate changes to the TS that the NRC has previously approved. There is no proposed expansion of scope from what was previously approved by the NRC. As such, the conclusion of the No Significant Hazards Consideration (NSHC) provided in the March 14, 2016 LAR stating that a finding of "no significant hazards consideration" is justified remains unchanged.

This letter contains no NRC commitments. If you have any questions, please contact Ken McElroy at 205.992.7369.

Mr. C. R. Pierce states he is Regulatory Affairs Director of Southern Nuclear Operating Company, is authorized to execute this oath on behalf of Southern Nuclear Operating Company and, to the best of his knowledge and belief, the facts set forth in this letter are true.

Respectfully submitted,

C. R. Pierce

C. R. Pierce
Regulatory Affairs Director

CRP/RMJ

Sworn to and subscribed before me this 26 *day of* October, *2016.*

Laura L. Crumpton
Notary Public

My commission expires: 10-8-2017

Enclosures:
1. Marked-up Technical Specification Pages
2. Clean Typed Technical Specification Pages

cc: Southern Nuclear Operating Company
Mr. S. E. Kuczynski, Chairman, President & CEO
Mr. D. G. Bost, Executive Vice President & Chief Nuclear Officer
Mr. D. R. Vineyard, Vice President – Hatch
Mr. M. D. Meier, Vice President – Regulatory Affairs
Mr. D. R. Madison, Vice President – Fleet Operations
Mr. B. J. Adams, Vice President – Engineering
Mr. G. L. Johnson, Regulatory Affairs Manager – Hatch
RType: CHA02.004

U. S. Nuclear Regulatory Commission
Ms. C. Haney, Regional Administrator
Mr. M. D. Orenak, NRR Project Manager – Hatch
Mr. D. H. Hardage, Senior Resident Inspector – Hatch

State of Georgia
Mr. J. H. Turner, Director – Environmental Protection Division

Edwin I. Hatch Nuclear Plant – Units 1 & 2
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Enclosure 1

Marked-up Technical Specification Pages

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.4.9.4	----------NOTE---------- Only required to be met in MODES 1, 2, 3, and 4 during startup of a recirculation pump. ---------- Verify the difference between the reactor coolant temperature in the recirculation loop to be started and the RPV coolant temperature is ≤ 50°F.	Once within 15 minutes prior to starting an idle recirculation pump
SR 3.4.9.5	----------NOTE---------- Only required to be met when tensioning/ detensioning the reactor vessel head bolting studs. ---------- Verify reactor vessel flange and head flange temperatures are within the limits specified in the PTLR.	Once within 30 minutes prior to tensioning/ detensioning the reactor vessel head bolting studs and in accordance with the Surveillance Frequency Control Program

(continued)

1.1 Definitions (continued)

PHYSICS TESTS	PHYSICS TESTS shall be those tests performed to measure the fundamental nuclear characteristics of the reactor core and related instrumentation. These tests are: a. Described in Chapter 14, Initial Tests and Operation, of the FSAR; b. Authorized under the provisions of 10 CFR 50.59; or c. Otherwise approved by the Nuclear Regulatory Commission.
PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)	The PTLR is the unit specific document that provides the reactor vessel pressure and temperature limits, including heatup and cooldown rates, for the current reactor vessel fluence period. These pressure and temperature limits shall be determined for each fluence period in accordance with Specification 5.6.7.
RATED THERMAL POWER (RTP)	RTP shall be a total reactor core heat transfer rate to the reactor coolant of 2804 MWt.
REACTOR PROTECTION SYSTEM (RPS) RESPONSE TIME	The RPS RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its RPS trip setpoint at the channel sensor until de-energization of the scram pilot valve solenoids. The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured.
SHUTDOWN MARGIN (SDM)	SDM shall be the amount of reactivity by which the reactor is subcritical or would be subcritical assuming that: a. The reactor is xenon free; b. The moderator temperature is 68°F; and c. All control rods are fully inserted except for the single control rod of highest reactivity worth, which is assumed to be fully withdrawn. With control rods not capable of being fully inserted, the reactivity worth of these control rods must be accounted for in the determination of SDM.
THERMAL POWER	THERMAL POWER shall be the total reactor core heat transfer rate to the reactor coolant.
STAGGERED TEST BASIS	A STAGGERED TEST BASIS shall consist of the testing of one of the systems, subsystems, channels, or other designated components during the interval specified by the Surveillance Frequency, so that all systems, subsystems, channels, or other designated components are tested during *n* Surveillance Frequency intervals, where *n* is the total number of systems, subsystems, channels, or other designated components in the associated function.

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ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. (continued)	C.2 Suspend CORE ALTERATIONS. AND	Immediately
	C.3 Initiate action to suspend OPDRVs.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.4.1.1	Verify all secondary containment equipment hatches are closed and sealed.	In accordance with the Surveillance Frequency Control Program
SR 3.6.4.1.2	Verify one secondary containment access door in each access opening is closed.	In accordance with the Surveillance Frequency Control Program
SR 3.6.4.1.3	----------NOTE---------- The number of standby gas treatment (SGT) subsystem(s) required for this Surveillance is dependent on the secondary containment configuration, and shall be one less than the number required to meet LCO 3.6.4.3, "Standby Gas Treatment (SGT) System," for the given configuration. ---------- Verify ~~required SGT subsystem(s) will draw down the~~ can be drawn down secondary containment to ≥ 0.20 inch of vacuum water gauge in ≤ 10 minutes using required standby gas treatment (SGT) subsystem(s).	In accordance with the Surveillance Frequency Control Program

(continued)

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Enclosure 2

Clean Typed Technical Specification Pages

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