

D880607

MEMORANDUM FOR: Victor Stello, Jr.
Executive Director for Operations

FROM: Raymond F. Fraley
Executive Director, ACRS

SUBJECT: PROPOSED PRIORITY RANKINGS OF GENERIC ISSUES:
FIFTH GROUP

During the 338th meeting of the Advisory Committee on Reactor Safeguards, June 2-4, 1988, the members reviewed the adequacy of the proposed priority rankings for a group of Generic Issues identified in the attached Table A, and their comments are contained in the following attachments.

- ~ Attachment 1 lists those issues for which the ACRS agrees with the priority rankings proposed by the NRC Staff.
- ~ Attachment 2 includes those issues for which the ACRS agrees with the priority rankings proposed by the NRC Staff, but has comments.
- ~ Attachment 3 identifies the Generic Issue for which the ACRS disagrees with the NRC Staff's proposed priority ranking along with the reasons therefor.

The members have requested that the NRC Staff provide written responses to the comments identified in Attachments 2 and 3.

The ACRS will continue its review of the adequacy of the proposed priority rankings for additional Generic Issues when they become available.

Attachments: As Stated

ATTACHMENT 1

LIST OF GENERIC ISSUES FOR WHICH
THE ACRS AGREES WITH THE
PRIORITY RANKINGS PROPOSED BY THE NRC STAFF

GENERIC ISSUE NO.	TITLE
88	Earthquakes and Emergency Planning
106	Piping and the Use of Highly Combustible Gases in Vital Areas
113	Dynamic Qualification Testing of Large Bore Hydraulic Snubbers
125.I.1	Availability of the Shift Technical Advisor
125.I.3	SPDS Availability
125.I.4	Plant Specific Simulator
125.I.6	Valve Torque Limit and Bypass Switch Settings
125.I.7.a	Recover Failed Equipment
125.I.7.b	Realistic Hands-on Training
125.I.8	Procedures and Staffing for Reporting to NRC Emergency Response Center
125.II.1.a	Two-Train AFW Unavailability
125.II.1.b	Review Existing AFW Systems for Single Failures
125.II.1.c	NUREG-0737 Reliability Improvements
125.II.1.d	AFW Steam and Feedwater Rupture Control System/ICS Interactions in B&W Plants
125.II.2	Assess the Adequacy of Existing Maintenance Requirements and Their Impact on the Resulting Reliability of Safety-Related Systems
125.II.5	Thermal-Hydraulic Effects of Loss and Restoration of Feedwater on Primary System Components
125.II.6	Reexamine PRA-based Estimates of the Likelihood of a Severe Core Damage Accident Based on Loss of All Feedwater
125.II.8	Reassess Criteria for Feed-and-Bleed Initiation
125.II.10	Hierarchy of Impromptu Operator Actions
125.II.12	Adequacy of Training Regarding PORV Operation
125.II.13	Operator Job Aids
126	Reliability of PWR Main Steam Safety Valves

128	Electrical Power Reliability
130	Essential Service Water Pump Failures at Multi-Plant Sites
133	Update Policy Statement on Nuclear Power Plant Staff Working Hours
136	Storage and Use of Large Quantities of Cryogenic Combustibles on Site

ATTACHMENT 2

GENERIC ISSUES FOR WHICH THE ACRS AGREES
WITH THE PROPOSED PRIORITY RANKING
BUT WITH COMMENTS

Generic Issue No:	127
Title:	Maintenance and Testing of Manual Valves in Safety-Related Systems
Priority Ranking Proposed By The NRC Staff:	LOW
ACRS Comments:	<p>The ACRS agrees with the proposed priority ranking for this issue, but offers the comment given below.</p> <p>Certain manual valves which may or may not be located in safety-related systems are identified in safety analyses or emergency procedures for manipulation during low-probability situations such as transients or accidents involving multiple component or system failures. There should be a requirement for surveillance and testing of such manual valves. This could be covered under a new generic issue or included as an extension of the severe accident program.</p>
Generic Issue No:	135
Title:	Steam Generator and Steam Line Overfill
Priority Ranking Proposed By The NRC Staff:	MEDIUM
ACRS Comments:	The ACRS agrees with the proposed priority ranking for this issue, but offers the comment given below.

With the sudden, complete, and unexpected steam generator tube rupture event that occurred at the North Anna Nuclear Plant on July 15, 1987, it would appear that another mechanism for initiating an overfill scenario has been discovered. This would indicate that the mechanisms for initiating steam generator and steam line overfill are not yet completely understood. The Staff should consider such mechanisms under this Generic Issue, and should recognize that new operating experience could change current expectations of the frequency of steam generator and steam line overfill events.

ATTACHMENT 3

GENERIC ISSUE FOR WHICH THE ACRS DISAGREES WITH THE PRIORITY RANKING PROPOSED BY THE NRC STAFF

Generic
Issue No: 134

Title: Rule on Degree and Experience Requirement

Priority Ranking
Proposed By The
NRC Staff: HIGH

ACRS Recommendation: DROP

Reasons: In its report of August 12, 1987 to the Commission regarding degree requirements for senior operators, the ACRS commented:

A number of Job and Task Analyses (JTAs) have been performed by licensees as part of the conversion to performance based training; analysis of these JTAs has not shown that a college degree is necessary for Senior Reactor Operators (SROs) to perform the tasks of their jobs to ensure safety of plant operations. A Peer Advisory Panel appointed by the Commission came to the same conclusion in 1982 and recommended against a degree requirement for SROs. We have not been informed of any technical rationale for requiring a degree for SROs at nuclear power plants; we conclude, therefore, that a degree requirement for all SROs is primarily a policy issue.

Based on the above, the ACRS believes that this issue should be dropped from further consideration.

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