DUKE POWER COMPANY
POWER BUILDING
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28242

WILLIAM O. PARKER, JR.
VICE PRESIDENT
STEAM PRODUCTION

TELEPHONE: AREA 704
373-4083

April 27, 1979

Mr. James P. O'Reilly, Director
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

Re: Oconee Nuclear Station
Docket Nos. 50-269, -270 and -287

Dear Mr. O'Reilly:

Please find attached Reportable Occurrence Report RO-269/79-10. This report is being submitted pursuant to Oconee Nuclear Station Technical Specifications 6.2 and 6.6.2.1.b(2), which concerns operation in a degraded mode permitted by a limiting condition for operation, and describes an incident which is considered to have no significance with respect to its effect on the health and safety of the public.

My letter of April 13, 1979 addressed the delays involved in preparing this report.

Very truly yours,

William O. Parker, Jr.

SRL:scs
Attachment

cc: Director, Office of Management Information
and Program Control

A002
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1/1

DUKE POWER COMPANY
OCONEE NUCLEAR STATION

Report Number: RO-269/79-10

Report Date: April 27, 1979

Occurrence Date: March 15 and 25, 1979

Facility: Oconee Nuclear Station, Seneca, South Carolina

Identification of Occurrence: High Pressure Service Water Pump Inoperable

Conditions Prior to Occurrence: Unit 1 100% Full Power
Unit 2 100% Full Power
Unit 3 100% Full Power

Description of Occurrence:

At 2245 on March 15, 1979 during a routine inspection of the turbine building basement, the motor cooler for high pressure service water (HPSW) pump B was observed to be leaking water, and the pump was removed from service. Oconee Nuclear Station Technical Specification 3.17.2.1 requires that the pump be restored to service within 7 days, or that a report be submitted within the next 30 days outlining plans and procedures to be implemented to compensate for loss of redundancy in the HPSW system. Therefore, although the removal of the pump from service was of no significance with respect to safe operation of the units, it did constitute operation in a degraded mode. At approximately 1000 on March 17, 1979, the leak was discovered to be coming from the joint connecting the supply line to the motor cooler tubing. The leaking joint was soldered and hydrostatically tested at operating pressure. HPSW pump B was returned to service at 1529 on March 17. The pump was again removed from service at 1015 on March 25, 1979 as a result of similar leakage. The motor cooler was again repaired, and the pump was returned to service at 1904 on the same day.

Apparent Cause of Occurrence:

HPSW pump B was removed from service as a result of water leakage from the pump's motor cooler. The leakage was identified to be coming from the joint between the motor cooler tubing and the supply header. The exact cause of the leak could not be determined.

Analysis of Occurrence:

Two redundant HPSW pumps are provided to supply high pressure water for the fire suppression water system. During the times HPSW pump B was out of service, HPSW pump A was operable and capable of meeting the fire protection safety requirements of the HPSW system. In addition, investigation of the HPSW B motor cooler leakage indicated that it was not of a magnitude which could have affected the integrity of the pump motor. Therefore, these incidents were of no significance with respect to safe operation of the units, and the health and safety of the public were not endangered.

Corrective Action:

Investigation of the motor cooler leakage revealed that the source was the joint connecting the supply header to the motor cooler tubing. The joint was soldered, and the motor cooler was pressurized to assure that the leak had been repaired. The pump was returned to service well within the seven days permitted by Technical Specification 3.17.2.1 both times the leakage was observed. Replacement motor coolers have been ordered, and delivery of the coolers is expected by April 30, 1979. In addition, consideration will be given to removing the plug from the motor casing drain to prevent any future leakage from reaching the pump motor.

NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 LICENSEE CODE: S C N E E 1 ② LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 ③ LICENSE TYPE: 4 1 1 1 1 ④ CAT ⑤

CON'T 01 REPORT SOURCE: L ⑥ DOCKET NUMBER: 0 5 0 0 0 2 6 9 ⑦ EVENT DATE: 0 3 1 5 7 9 ⑧ REPORT DATE: 0 4 2 7 7 9 ⑨

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

02 On March 15 and again on March 25, 1979, HPSW pump B was removed from service
03 in order to repair leakage from the pump motor cooler. During the times the
04 pump was out of service, redundant pump HPSW A was operable. In addition,
05 the leakage was determined to be of a magnitude which would not have affected
06 pump motor performance. Therefore, safe operation of the units was not
07 affected, and the health and safety of the public were not endangered.
08

09 SYSTEM CODE: W A ⑪ CAUSE CODE: E ⑫ CAUSE SUBCODE: B ⑬ COMPONENT CODE: M O T O R X ⑭ COMP. SUBCODE: Z ⑮ VALVE SUBCODE: Z ⑯

⑰ LER/RO REPORT NUMBER: EVENT YEAR: 7 9 — SEQUENTIAL REPORT NO.: 0 1 0 / OCCURRENCE CODE: 0 3 REPORT TYPE: L — REVISION NO.: 0

ACTION TAKEN: B ⑱ FUTURE ACTION: A ⑲ EFFECT ON PLANT: Z ⑳ SHUTDOWN METHOD: Z ㉑ HOURS ㉒: 0 0 0 0 ATTACHMENT SUBMITTED: Y ㉓ NPRD-4 FORM SUB.: Y ㉔ PRIME COMP. SUPPLIER: L ㉕ COMPONENT MANUFACTURER: W 1 2 0 ㉖

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ㉗

10 The source of the leakage was determined to be the joint connecting the
11 cooler supply line to the motor cooler tubing. The exact cause of the leak-
12 age could not be determined. The joint was resoldered and the cooler was
13 pressurized to assure that the leaks were repaired. Delivery of replacement
14 coolers is expected by April 30, 1979.

15 FACILITY STATUS: E ㉘ % POWER: 1 0 0 ㉙ OTHER STATUS ㉚: NA METHOD OF DISCOVERY: B ㉛ DISCOVERY DESCRIPTION ㉜: Routine Inspection

16 ACTIVITY RELEASED: Z ㉝ CONTENT OF RELEASE: Z ㉞ AMOUNT OF ACTIVITY ㉟: NA LOCATION OF RELEASE ㊱: NA

PERSONNEL EXPOSURES
17 NUMBER: 0 0 0 ㊲ TYPE: Z ㊳ DESCRIPTION ㊴: NA

PERSONNEL INJURIES
18 NUMBER: 0 0 0 ㊵ DESCRIPTION ㊶: NA

LOSS OF OR DAMAGE TO FACILITY ㊸
19 TYPE: Z ㊷ DESCRIPTION: NA

PUBLICITY
20 ISSUED: N ㊹ DESCRIPTION ㊺: NA

NRC USE ONLY

NAME OF PREPARER S. R. Lewis

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