

LR-N16-0190

10 CFR 50.73

October 11, 2016

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555-0001

Salem Nuclear Generating Station Unit 1

Renewed Facility Operating License No. DPR-70

NRC Docket No. 50-272

SUBJECT:

LER 272/2016-005-000

Instrumentation Channels Inoperable for longer than allowed by

Technical Specification

Licensee Event Report, "Instrumentation Channels Inoperable for longer than allowed by Technical Specification" is being submitted pursuant to 10 CFR 50.73 (a)(2)(i)(B), "Any operation or condition which was prohibited by the plant's Technical Specifications....."

Should you have any questions or comments regarding the submittal, please contact Mr. Thomas Cachaza of Regulatory Affairs at 856-339-5038.

There are no regulatory commitments contained in this letter.

MARTINO FOR GROVER

Sincerely,

F. Kenneth Grover Salem Plant Manager

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Enclosure - LER 272/2016-005-000

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NRC FORM 366

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB: NO. 3150-0104

EXPIRES: 10/31/2018

(11-2015)



LICENSEE EVENT REPORT (LER)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Privacy and Information Collections Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and

(See Page 2 for required number of digits/characters for each block)							Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.									
1. FACILITY NAME								I				3. [3. PAGE			
Salem Generating Station – Unit 1							05000272 1 OF 3									
4. TITLE Instrumentation Channels Inoperable for longer								than allowed by Technical Specification								
5. EVENT DATE			6. LER NUMBER			7. REPORT DAT			8. OTHER FACILITIES INV							
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9. OP	ERATIN	MODE	HIS REPORT	IS SUBN	SUBMITTED PURSUANT				HE REQUIREMENTS OF 10 CFR §: (Check all that apply					pply)		
1			20.2201(b)			20.2203(a)(3)(i)			50.73(a)(2)(ii)(A)				50.73(a)(2)(viii)(A)			
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						50.73(a)(2)(i)(C)			OTHER Specify in Abstract below or in NRC Form 366A							
12. LICENSEE CONTACT FOR THIS LER																
Thomas J. Cachaza, Senior Regulatory Compliance Engineer 856-339-5																
13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT																
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This report is being made in accordance with 10CFR50.73 (a)(2)(i)(B), "Any operation or condition which was prohibited by the plant's

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LICENSEE EVENT REPORT (LER) CONTINUATION SHEET

APPROVED BY OMB: NO. 3150-0104

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NARRATIVE

PLANT AND SYSTEM IDENTIFICATION

Westinghouse-Pressurized Water Reactor (PWR/4)

Plant Protection System {JC} Engineered Safety Feature Actuation System {JE}

*Energy Industry Identification System (EIIS) codes and component function identifier codes appear as {SS/CCC}.

IDENTIFICATION OF OCCURRENCE

Event Date: 08/11/2016 Discovery Date: 08/11/2016

CONDITIONS PRIOR TO OCCURRENCE

Salem Unit 1 was in Mode 1 at 99.9 percent rated thermal power (RTP).

DESCRIPTION OF OCCURRRENCE

On August 11, 2016, the channel check for the 13 Loop Over Temperature Delta Temperature (OTDT) was unsat and the channel was declared inoperable. Investigation revealed that the Pressurizer (PZR) Pressure Channel III, input to 13 OTDT, had not been calibrated to the correct acceptance criteria. The PZR pressure Channel III was declared inoperable. During 1R24 on April 21, 2016, PZR channel III had been out of specification during calibration and was adjusted in accordance with the procedure acceptance criteria. Evaluation of the out of specification condition identified that the procedure acceptance criteria was incorrect. The need to reperform the calibration was identified. Due to inadequate communication of the surveillance status and inaccurate assumptions by individuals during the calibration, the channel calibration with the correct values was not re-performed. The unit was returned to service on July 30, 2016 (entered mode 1) without the channel recalibration being performed.

The 13 loop OTDT channel and the PZR pressure channel III were required to be operable in modes 1, 2, and 3. The inoperable channels impacted three Technical Specifications (TS). TS 3.3.2.1, Engineered Safety Feature Actuation System (ESFAS) and TS 3.3.1.1, Reactor Trip System Instrumentation both require that the inoperable channel be placed in the tripped condition within 6 hours. The inoperability of PZR pressure channel III also impacted the function of 1PR1 power operated relief valve (PORV). TS 3.4.3, Relief Valves, requires that with one inoperable PORV, that the PORV is restored to operable within 1 hour or close its associated block valve, remove power from the block valve, and restore the PORV to operable status within the following 72 hours....... The action times for each of these technical specifications were not met.

This report is being made in accordance with 10CFR50.73 (a)(2)(i)(B), "Any operation or condition which was prohibited by the plant's Technical Specifications....."

CAUSE OF EVENT

The cause of this event is inadequate turnover and communication of surveillance status and inaccurate assumptions regarding the requirement to perform the necessary surveillance's prior to clearing a mode change restriction.

SAFETY CONSEQUENCES AND IMPLICATIONS

There was no safety consequence associated with this event. This event only impacted one channel of OTDT and PZR Pressure. The other three channels of each were not affected by this issue and were operable and capable of performing their function. Additionally the 1PR2 (PORV) was operable and capable of performing its function. A review of the UFSAR chapter 15 accidents identified that opening one PORV is adequate to limit pressurizer pressure and prevent the pressurizer safety valves from opening.

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NARRATIVE

SAFETY SYSTEM FUNCTIONAL FAILURE

This condition did not result in a safety system functional failure as defined in NEI 99-02, Regulatory Assessment Performance Indicator Guidelines.

PREVIOUS EVENTS

A review of previous events for the past three years did not identify any similar events.

CORRECTIVE ACTIONS

The PZR Channel III procedure was corrected and the channel was calibrated satisfactorily. Both the PZR Channel III and the 13 loop OTDT were restored to operability.

An extent of condition review of approximately 125 operability determinations was completed with no additional errors found.

Commitments

There are no regulatory commitments contained in this LER.