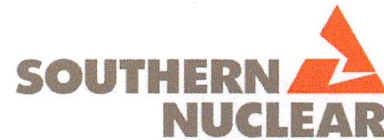


M. J. Yox  
Regulatory Affairs Director  
Vogtle 3&4  
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Southern Nuclear  
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Tel 706.848.6459



A SOUTHERN COMPANY

Docket No.: 52-025

**SEP 29 2016**

ND-16-1848  
10 CFR 52.99(c)(3)

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555-0001

Southern Nuclear Operating Company  
Vogtle Electric Generating Plant Unit 3  
Notice of Uncompleted ITAAC 225-days Prior to Initial Fuel Load  
Item 2.2.03.09a.iii [Index Number 203]

Ladies and Gentlemen:

Pursuant to 10 CFR 52.99(c)(3), Southern Nuclear Operating Company hereby notifies the NRC that as of September 30, 2016, Vogtle Electric Generating Plant (VEGP) Unit 3 Uncompleted Inspection, Test, Analysis, and Acceptance Criteria (ITAAC) Item 2.2.03.09a.iii [Index Number 203] has not been completed greater than 225-days prior to initial fuel load. Enclosure 1 describes the plan for completing ITAAC 2.2.03.09a.iii [Index Number 203]. Southern Nuclear Operating Company will at a later date provide additional notifications for ITAAC that have not been completed 225-days prior to initial fuel load.

This notification is informed by the guidance described in NEI-08-01, *Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52*, which was endorsed by the NRC in Regulatory Guide 1.215. In accordance with NEI 08-01, this notification includes ITAAC for which required inspections, tests, or analyses have not been performed or have been only partially completed. All ITAAC will be fully completed and all Section 52.99(c)(1) ITAAC Closure Notifications will be submitted to NRC to support the Commission finding that all acceptance criteria are met prior to plant operation, as required by 10 CFR 52.103(g).

This letter contains no new NRC regulatory commitments.

If there are any questions, please contact David Woods at 706-848-6903.

Respectfully submitted,

A handwritten signature in black ink, appearing to read "Michael J. Yox".

Michael J. Yox  
Regulatory Affairs Director Vogtle 3&4

MJY/KMS/amm

U.S. Nuclear Regulatory Commission

ND-16-1848

Page 2 of 4

**Enclosure:**

1. Vogtle Electric Generating Plant (VEGP) Unit 3 Completion Plan for Uncompleted ITAAC  
Item 2.2.03.09a.iii [Index Number 203]

**To:**

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Document Services RTYPE: VND.LI.L06

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Mr. S. Blanton, *Balch Bingham*

ND-16-1848  
Enclosure 1  
Completion Plan

**Southern Nuclear Operating Company**

**ND-16-1848**

**Enclosure 1**

**Vogtle Electric Generating Plant (VEGP) Unit 3**

**Completion Plan for Uncompleted ITAAC  
Item 2.2.03.09a.iii [Index No. 203]**

**Subject: Uncompleted ITAAC 2.2.03.09a.iii [Index No. 203]**

### **ITAAC Statement**

#### **Design Commitment**

9.a) *The PXS provides a function to cool the outside of the reactor vessel during a severe accident.*

#### **Inspections/Tests/Analyses**

iii) *Inspections will be conducted of the flow path(s) from the loop compartments to the reactor vessel cavity.*

#### **Acceptance Criteria**

iii) *A flow path with a flow area not less than 6 ft<sup>2</sup> exists from the loop compartment to the reactor vessel cavity.*

### **ITAAC Completion Description**

Multiple ITAAC are performed to verify the Passive Core Cooling System (PXS) provides a function to cool the outside of the Reactor Vessel during a severe accident. The subject ITAAC requires that inspections of the of the flow path from the loop compartments to the Reactor Vessel Cavity be conducted to verify that a flow path with a flow area not less than 6 ft<sup>2</sup> exists.

The flow path inspected for this ITAAC is from the Steam Generator loop compartments, through the Vertical Access Room, down into the Reactor Coolant Drain Tank Room, into the Reactor Vessel Cavity access tunnel and into the Reactor Vessel Cavity. The actual flow area of the identified flow path is determined from measurements taken at each opening between rooms in the flow path to determine the width and height of the openings. Multiplication of the width and height measurements at each measurement point will enable determination of the area of the flow path at that measurement point. Calculations are then performed to determine the flow area for the identified flow path from the loop compartments to the Reactor Vessel Cavity.

The flow area inspection results are documented in the Principal Closure Document XXX (Reference 1) supporting the ITAAC 2.2.03.09a.iii Completion Package (Reference 2). The calculated flow area for the flow path from the loop compartments to the Reactor Vessel Cavity is x ft<sup>2</sup>. This confirms that a flow path with a flow area not less than 6 ft<sup>2</sup> exists from the loop compartments to the Reactor Vessel Cavity and meets the ITAAC acceptance criteria.

Principal Closure Document XXX is available for NRC inspection as part of the ITAAC 2.2.03.09a.iii Completion Package.

**List of ITAAC Findings**

In accordance with plant procedures for ITAAC completion, Southern Nuclear Operating Company (SNC) performed a review of all findings pertaining to the subject ITAAC and associated corrective actions. This review found there are no relevant ITAAC findings associated with this ITAAC.

**References (available for NRC inspection)**

1. Principal Closure Document XXX
2. ITAAC 2.2.03.09a.iii Completion Package
3. NEI 08-01, "Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52"