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Docket No.: 52-025

SEP 28 2016

ND-16-1887
10 CFR 52.99(c)(3)

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555-0001

Southern Nuclear Operating Company
Vogtle Electric Generating Plant Unit 3
Notice of Uncompleted ITAAC 225-days Prior to Initial Fuel Load
Item 3.3.00.02f [Index Number 774]

Ladies and Gentlemen:

Pursuant to 10 CFR 52.99(c)(3), Southern Nuclear Operating Company hereby notifies the NRC that as of September 30, 2016, Vogtle Electric Generating Plant (VEGP) Unit 3 Uncompleted Inspection, Test, Analysis, and Acceptance Criteria (ITAAC) Item 3.3.00.02f [Index Number 774] has not been completed greater than 225-days prior to initial fuel load. Enclosure 1 describes the plan for completing ITAAC 3.3.00.02f [Index Number 774]. Southern Nuclear Operating Company will at a later date provide additional notifications for ITAAC that have not been completed 225-days prior to initial fuel load.

This notification is informed by the guidance described in NEI-08-01, *Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52*, which was endorsed by the NRC in Regulatory Guide 1.215. In accordance with NEI 08-01, this notification includes ITAAC for which required inspections, tests, or analyses have not been performed or have been only partially completed. All ITAAC will be fully completed and all Section 52.99(c)(1) ITAAC Closure Notifications will be submitted to NRC to support the Commission finding that all acceptance criteria are met prior to plant operation, as required by 10 CFR 52.103(g).

This letter contains no new NRC regulatory commitments.

If there are any questions, please contact David Woods at 706-848-6903.

Respectfully submitted,

Michael J. Yox
Regulatory Affairs Director Vogtle 3&4

U.S. Nuclear Regulatory Commission

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MJY/KMS/amm

Enclosure:

1. Vogtle Electric Generating Plant (VEGP) Unit 3 Completion Plan for Uncompleted ITAAC
Item 3.3.00.02f [Index Number 774]

To:

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Enclosure 1
Completion Plan

Southern Nuclear Operating Company

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Enclosure 1

Vogtle Electric Generating Plant (VEGP) Unit 3

**Completion Plan for Uncompleted ITAAC
Item 3.3.00.02f [Index No. 774]**

Subject: Uncompleted ITAAC 3.3.00.02f [Index No. 774]

ITAAC Statement

Design Commitment

2.f) *The key dimensions of nuclear island structures are defined on Table 3.3-5.*

Inspections/Tests/Analyses

An inspection will be performed of the as-built configuration of the nuclear island structures.

Acceptance Criteria

A report exists and concludes that the key dimensions of the as-built nuclear island structures are consistent with the dimensions defined on Table 3.3-5.

ITAAC Completion Description

Inspection of the as-built configuration of the nuclear island structures is conducted to confirm that the key dimensions of the as-built nuclear island structures are consistent with the dimensions defined on VEGP Unit 3 Combined License (COL) Appendix C Table 3.3-5 (Attachment A).

Inspection of the distance from bottom of containment sump to top surface of embedded containment shell key dimension is performed per ITAAC 3.3.00.09 and is documented in the ITAAC Completion Package for ITAAC 3.3.00.09 (Reference 1). Inspection of the site grade level relative to the design plant grade floor elevation key dimension is performed per ITAAC 3.3.00.02b and is documented in the ITAAC Completion Package for ITAAC 3.3.00.02b (Reference 2). For the remainder of the key dimensions identified in Attachment A an inspection is performed by surveying the as-built configuration of the nuclear island structures using survey equipment in accordance with site survey and measurement procedures. The results of the as-built measured dimensions are summarized in Attachment A. The as-built measured dimension inspection results are compared to the dimensions defined in Attachment A to validate that the dimensions of the as-built nuclear island structures are consistent with the dimensions defined on Attachment A.

The as-built measured dimension inspection results are documented in the Principal Closure Document XXX (Reference 3) supporting the ITAAC 3.3.00.02f Completion Package (Reference 4) and verify that the key dimensions of the as-built nuclear island structures are consistent with the dimensions defined on VEGP Unit 3 COL Appendix C Table 3.3-5.

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Enclosure 1
Completion Plan

Principal Closure Document XXX is available for NRC inspection as part of the ITAAC 3.3.00.02f Completion Package.

List of ITAAC Findings

In accordance with plant procedures for ITAAC completion, Southern Nuclear Operating Company (SNC) performed a review of all findings pertaining to the subject ITAAC and associated corrective actions. This review found there are no relevant ITAAC findings associated with this ITAAC.

References (available for NRC inspection)

1. ITAAC 3.3.00.09 Completion Package
2. ITAAC 3.3.00.02b Completion Package
3. Principal Closure Document XXX
4. ITAAC 3.3.00.02f Completion Package
5. NEI 08-01, "Industry Guideline for the ITAAC Closure Process Under 10 CFR Part 52"

Attachment A: Excerpt from VEGP Unit 3 COL Appendix C - Table 3.3-5

Key Dimension	Nominal Dimension	Tolerance	As-built Measured Dimension
Distance between Outside Surface of walls at Column Line I & N when Measured at Column Line 1	91 ft-0 in	+3 ft -1 ft	xx ft-yy in
Distance from Outside Surface of wall at Column Line 1 to Column Line 7 when Measured at Column Line I	138 ft-0 in	+3 ft -1 ft	xx ft-yy in
Distance from Outside Surface of wall at Column Line 11 to Column Line 7 when Measured at Column Line I	118 ft-0 in	+3 ft -1 ft	xx ft-yy in
Distance between Outside Surface of walls at Column Line I & Q when Measured at Column Line 11	117 ft-6 in	+3 ft -1 ft	xx ft-yy in
Distance from Outside Surface of wall at Column Line Q to Column Line N when Measured at Column Line 11	29 ft-0 in	+3 ft -1 ft	xx ft-yy in
Distance between Outside Surface of shield building wall to shield building centerline when Measured on West Edge of Shield Building	72 ft-6 in	+3 ft -1 ft	xx ft-yy in
Distance between shield building centerline to Reactor Vessel centerline when Measured along Column Line N in North-South Direction	7 ft-6 in	± 3 in	xx ft-yy in
Distance from Bottom of Containment Sump to Top Surface of Embedded Containment Shell	2 ft-8 in	± 3 in	See Reference 1
Distance from top of Basemat to Design Plant Grade	33 ft-6 in	± 1 ft	xx ft-yy in
Distance of Design Plant Grade (Floor elevation 100'-0") relative to Site Grade	0 ft	± 3 ft-6 in	See Reference 2
Distance from Design Plant Grade to Top Surface of Shield Building Roof	229 ft-0 in	± 1 ft	xx ft-yy in