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ACCESSION NBR: 9202040117 DOC. DATE: 92/01/28 NOTARIZED: NO DOCKET #
FACIL: 50-269 Oconee Nuclear Station, Unit 1, Duke Power Co. 05000269
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HAMPTON, J.W. Duke Power Co.
RECIP. NAME RECIPIENT AFFILIATION
 Document Control Branch (Document Control Desk)

SUBJECT: Forwards Request for Relief 92-01 from requirements of Section XI of ASME Boiler & Pressure Vessel Code in ref to second 10-yr interval ISI of piping between valves 1HP-398 & 1HP-446, 447, 448 & 449.W/two oversize drawings.

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TITLE: OR Submittal: Inservice Inspection/Testing/Relief from ASME Code

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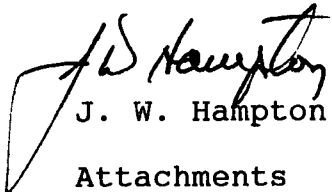
January 28, 1992

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Subject: Oconee Nuclear Station
Docket Nos. 50-269
Second Ten Year Interval
Request for Relief No. 92-01

Pursuant to 10 CFR 50.55a, please find attached request for relief number 92-01 from the requirements of Section XI of the ASME Boiler and Pressure Vessel Code (with Addenda through Winter 1980). This request is being submitted due to the impracticality of pressure testing as required by the code. The attached request concerns the inservice inspection at Oconee Unit 1 being performed during the second ten year interval. I am requesting approval on this request as soon as possible.

Very truly yours,


J. W. Hampton

Attachments

rr9201

xc: Mr. S. D. Ebnetter
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Duke Power Company
Oconee Nuclear Station
Second Ten Year Interval
Request for Relief No. 92-01

I. Component for Which Relief is Requested:

(a) Name and Number:

Piping between 1HP-398 and 1HP-446, 447, 448, and 449. (OFDs 101A-1.4 and 101A-1.5)

(b) Function:

Emergency flow path to the Reactor Coolant Pump seals from the RC Makeup pump.

(c) ASME Code Class:

ISI Class B and C, Duke Class B

(d) IWV-2200 Valve Category:

N/A

II. Reference Code Requirement that has been Determined to be Impractical:

ASME Boiler and Pressure Vessel Code Section XI, 1980 Edition (with Addenda through Winter 1980) Article IWC-5210(a)(2) which requires that the pressure retaining components within each system boundary shall be subjected to visual examination by the method specified in Table IWC-2500-1, Examination Category C-H and a system pressure test (IWA-5211(d) for each system or portions of systems and for repaired or replaced components, or altered portions of systems.

III. Basis for Requesting Relief:

This section of line is currently rated for 2500 PSI and has been hydrostatically tested to 2750 psi and 3137 psi depending on the specific section of pipe. The new pressure rating of this pipe will be 2790 psi. To perform the required ASME Code hydrostatic test would require shutting the Unit down to cold shutdown and securing flow to the Reactor Coolant Pump seals.

The valves, 1HP-446, 447, 448, and 449 are 1" manually operated valves in the reactor building. These valves are normally open. As long as these valves are open the RCS would prevent over-pressurizing the section of pipe that would not be hydrostatically tested until the Unit is in cold shutdown and flow secured to the seals.

IV. Alternate Examination:

None proposed. This line will be tested at the next cold shutdown when flow is secured to the reactor coolant pump seals.

V. Evaluation of Acceptability of Proposed Alternate Testing with Respect to the Level of Quality and Safety as well as Public Health and Safety:

With the four isolation valves in the open position, this line will not experience the full pressure of 2790 psi. The Code Relief Valve has a set point of 2500 psi and would open prior to the system pressure reaching 2790 psi. All of this piping is located within the Reactor Building and provides an acceptable level of quality and safety to the general public.

VI. Implementation Schedule:

January, 1992. This line was last hydrostatically tested during the last refueling outage on Unit 1 in 1991.