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1.6

MATERIAL INCORPORATED BY REFERENCE

The following topical reports are incorporated by reference.

<u>Report Number</u>	<u>Author and Title</u>	<u>Date to NRC</u>	<u>FSAR Section</u>
CENPD-26 (with Suppl. 1 through 5)	Combustion Engineering, Inc. "Description of Combustion Engineering Loss of Coolant Calculational Procedures"	August 1971	3.6, 6.2
CENPD-42	Combustion Engineering, Inc. "Dynamic Analysis of Reactor Vessel Internals Under Loss of Coolant Accident Conditions with Application to C-E 800 Mwe Class Reactors"	August 1972	3.6
CENPD-67 (with Suppl. 1 and 2 and Addenda 1 and 2)	Combustion Engineering, Inc. "Iodine Decontamination Factors During PWR Steam Generation and Steam Venting"	September 1973	10.3
CENPD-87	"Safety Related Research Development for CE PWR's, Program Summaries"		1.5
CENPD-98	Combustion Engineering, Inc. "Coast Code Description"	July 1973	4.4, 15.0
CENPD-105	Combustion Engineering, Inc. "Fast Neutron Attenuation by the ANISN-SHADRAC Analytical Method"	June 1973	4.3
CENPD-107 (with Suppl. 1 through 4)	Combustion Engineering, Inc. "CESEC"	August 1974	15.0
CENPD-118	Combustion Engineering, Inc. "Densification of Combustion Engineering Fuel"	September 1974	4.3
CENPD-132 (with Suppl. 1 and 2)	Combustion Engineering, Inc. "Calculative Methods for the C-E Large Break LOCA Evaluation Model"	September 1974	6.2, 15.6 6.3

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<u>Report Number</u>	<u>Author and Title</u>	<u>Date to NRC</u>	<u>FSAR Section</u>
CENPD-133 (with Suppl. 2)	Combustion Engineering, Inc. "CEFLASH-4A Fortran IV Digital Computer Program for Reactor Blowdown Analysis"	September 1974	6.2, 15.6 6.3
CENPD-134 (with Suppl. 1)	Combustion Engineering, Inc. "COMPERC-II A Program for Emergency Refill-Reflood of the Core"	September 1974	6.2, 15.6 6.3
CENPD-135 (with Suppl. 2 and 4)	Combustion Engineering, Inc. "STRIKIN-II A Cylindrical Geometry Fuel Rod Heat Transfer Program"	September 1974	4.2, 6.3, 15.6
CENPD-136	Combustion Engineering, Inc. "High Temperature Properties of Zircaloy and UO ₂ , for use in LOCA Evaluation Model"	September 1974	4.2, 15.6
CENPD-137 (with Suppl. 1)	Combustion Engineering, Inc. "Calculative Methods for the C-E Small Break LOCA Evaluation Model"	September 1974	6.3, 15.6
→(DRN 01-758) CENPD-138	"PARCH, A FORTAN IV Digital Program to Evaluate Pool Boiling, Axial Rod and Coolant Heatup" (with Supplement 1)	February 1975	15.6
←(DRN 01-758) CENPD-139 (with Suppl. 1)	Combustion Engineering, Inc. "C-E Fuel Evaluation Model"	September 1974	4.1, 4.2, 4.3, 6.3
CENPD-145	Combustion Engineering, Inc. "A Method of Analyzing In-Core Detector Data in Power Reactors"	April 1975	4.3, 7.7
CENPD-148	Combustion Engineering, Inc. "Review of Reactor Shutdown System (PPS Design) for Common Mode Failure Susceptibility"	November 1974	4.6
CENPD-153	Combustion Engineering, Inc. "Evaluation Uncertainty in the Nuclear Form Factor"	August 1974	4.3

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<u>Report Number</u>	<u>Author and Title</u>	<u>Date to NRC</u>	<u>FSAR Section</u>
CENPD-153 (con't)	Measured by Self Powered Fixed In-Core Detector Systems"		
CENPD-155	Combustion Engineering, Inc. "C-E Procedure for Design, Fabrication, Installation and Inspection of Surveillance Specimen Brackets Attached to Reactor Vessel Beltline Region"	October 1974	5.3
CENPD-161	Combustion Engineering, Inc, "TORC - A Computer Code for Determining the Thermal Margin of a Reactor Core"	June 1975	4.1, 4.2, 4.4, 15.0
CENPD-162 (with Suppl. 1)	Combustion Engineering, Inc. "CHF Correlation for C-E Fuel Assemblies with Standard Spacer Grids - Part 1; Uniform Axial Power Distribution"	May 1975	4.4
CENPD-168 Rev. 1	Combustion Engineering, Inc. "Design Basis Pipe Breaks for the Combustion Engineering Two Loop Reactor Coolant System"	September 1976	3.6, 6.2
CENPD-169	Combustion Engineering, Inc. "Assessment of the Accuracy of PWR Operating Limits as Determined by Core Operating Limit Supervisory System"	August 1975	4.3, 7.2 7.7
CENPD-170	Combustion Engineering, Inc. "Assessment of the Accuracy of the PWR Safety System Actuation as Performed by the Core Protection Calculators"	August 1975	7.2
CENPD-178P and 178	Combustion Engineering, Inc. "Structural Analysis of the 16 x 16 Fuel Assembly for Combined Seismic and Loss-of-Coolant-Accident Loadings"	October 1976	3.9, 4.2

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<u>Report Number</u>	<u>Author and Title</u>	<u>Date to NRC</u>	<u>FSAR Section</u>
CENPD-179	Combustion Engineering, Inc. "C-E Thermo-Structural Fuel Evaluation Method"	April 1976	4.2
CENPD-183	Combustion Engineering, Inc. "C-E Methods for Loss of Flow Analysis"	August 1975	15.0 15.3
CENPD-187 (with Suppl. 1)	Combustion Engineering, Inc. "Method of Analyzing Creep Collapse of Oval Cladding"	October 1975	4.2
CENPD-190	Combustion Engineering, Inc. "C-E Method for Control Element Assembly Ejection Analysis"	January 1976	15.4
CENPD-198P and 198	Combustion Engineering, Inc. "Zircaloy Growth-In-Reactor Dimensional Changes in Zircaloy-4 Fuel Assemblies"	December 1975	4.2
CENPD-206	Combustion Engineering, Inc. "Comparison of TORC Code Predictions with Experimental Data"	December 1976	4.4
CENPD-207	Combustion Engineering, Inc. "Critical Heat Flux Correlation for C-E Fuel Assemblies with Standard Spacer Grids, Part 2, Non-Uniform Axial Power Distributions"	June 1976	4.4
CENPD-213	Combustion Engineering, Inc. "Application of FLECHT Reflood Heat Transfer Coefficients to Combustion Engineering 16 x 16 Fuel Bundles"	February 1976	6.3, 15.6
CENPD-225P and 225	Combustion Engineering, Inc. "Fuel and Poison Rod Bowing"	October 1976	4.2, 4.4
CENPD-252	"Method for the Analysis of Blowdown"	July 1979	3.9E

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<u>Report Number</u>	<u>Author and Title</u>	<u>Date to NRC</u>	<u>FSAR Section</u>
WCAP 7709-L	Electric Hydrogen Recombiners for PWR Containments	April 1972	6.2.5
→(DRN 03-2054, R14) CEN-367-A	Combustion Engineering, Inc. Leak-Before-Break of Primary Coolant Loop Piping in Combustion Engineering Designed Nuclear Steam Supply System	February 1991	3.6
←(DRN 03-2054, R14) →(EC-13881, R304) WCAP-11596-P-A	Westinghouse Electric Company, "Qualification of the PHOENIX – P/ANC Nuclear Design System for Pressurized Water Reactor Cores"	June 1988	4.2, 4.3A, 15.1
WCAP-10965-P-A	Westinghouse Electric Company, "ANC: A Westinghouse Advanced Nodal Computer Code"	September 1986	4.2, 4.3A, 15.1
WCAP-10965-P-A Addendum 1	Westinghouse Electric Company, "ANC: A Westinghouse Advanced Nodal Computer Code: Enhancements to ANC Rod Power Recovery"	April 1989	4.2, 4.3A, 15.1
WCAP-16045-P-A	Westinghouse Electric Company, "Qualification of the Two-Dimensional Transport Code PARAGON,"	August 2004	4.3A
WCAP-16072-P-A	Westinghouse Electric Company, "Implementation of Zirconium Diboride Burnable Absorber Coatings in CE Nuclear Power Fuel Assembly Designs"	August 2004	4.2, 4.3A
CENPD-404-P-A	Combustion Engineering, Inc., "Implementation of ZIRLO Material Cladding in CE Nuclear Power Fuel Assembly Designs,"	November 2001	4.2, 4.3A, 15.0
WCAP-16500-P-A	Westinghouse Electric Company, "CE 16 x 16 Next Generation Fuel Core Reference Report, "	August 2007	4.2, 4.3A
WCAP-12610-P-A and CENPD-404-P-A Addendum 1-A ←(EC-13881, R304)	Westinghouse Electric Company, "Optimized ZIRLO™,"	July 2006	4.2, 4.3A, 15.0

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→(EC-13881, R304)

<u>Report Number</u>	<u>Author and Title</u>	<u>Date to NRC</u>	<u>FSAR Section</u>
WCAP-16523-P-A	Westinghouse Electric Company, "Westinghouse Correlations WSSV and WSSV-T for Predicting Critical Heat Flux in Rod Bundles with Side-Supported Mixing Vaness,"	August 2007	4.4, 15.0, 15.1, 15.2, 15.3, 15.4
CENPD-387-P-A	Combustion Engineering, Inc., "ABB Critical Heat Flux Correlations for PWR Fuel,"	May 2000	4.3A, 15.0
WCAP-15996-P-A, Rev. 1	Westinghouse Electric Company, "Technical Manual for the CENTS Code,"	March 2005	15.0, 15.1, 15.2, 15.3, 15.4
CEN-356(V)-P-A, Revision 01-P-A	Combustion Engineering, Inc., "Modified Statistical Combination of Uncertainties,	May 1988	4.2, 4.3A
←(EC-13881, R304) →(EC-19087, R305) WCAP-17817-P	Technical Justification for Eliminating Pressurizer Surge Line Rupture as the Structural Design Basis for Waterford Steam Electric Station, Unit 3 Using Leak-Before- Break Methodology	February 2010	3.6.3

←(EC-19087, R305)