

1.0 INTRODUCTION AND INTERFACES

This chapter of the final safety evaluation report (FSER) is organized as follows:

- Section 1.1 provides an overview of the entire combined license (COL) application.
- Section 1.2 provides the regulatory basis for the COL licensing process.
- Section 1.3 provides an overview of the COL application principal review matters and describes where the staff has documented its review of the 11 parts of the COL application.
- Section 1.4 documents the staff's review of Chapter 1 of the final safety analysis report (FSAR).
- Section 1.5 documents regulatory findings that are in addition to those directly related to the staff's review of the FSAR.

1.1 Summary of Application

In a letter dated June 30, 2009, as supplemented by several letters, Florida Power and Light Company (FPL or the applicant), submitted its application to the U.S. Nuclear Regulatory Commission (NRC or the Commission) for a COL for two Westinghouse AP1000 advanced passive pressurized-water reactors (PWR) pursuant to the requirements of Sections 103 and 185(b) of the *Atomic Energy Act of 1954*, as amended (AEA), and Title 10 of the *Code of Federal Regulations* (10 CFR) Part 52, "Licenses, Certifications and Approvals for Nuclear Power Plants." These reactors would be identified as Turkey Point Plant (Turkey Point), Units 6 and 7, and would be located in unincorporated southeast Miami-Dade County, Florida (FL), on an approximately 218-acre area south of the existing Turkey Point Units 3 and 4.

Unless otherwise noted, this FSER (also referred to as the safety evaluation report (SER) in later sections of this document) is based on Revision 8 of the Turkey Point COL application (Agencywide Documents Access and Management System (ADAMS) Accession No. ML16251A127).

As indicated in the applicant's August 26, 2016, Revision 8 submission, the applicant incorporates by reference 10 CFR Part 52, Appendix D, "Design Certification Rule for the AP1000 Design," and the Westinghouse Electric Corporation's Design Control Document (DCD), Revision 19.

The AP1000 nuclear reactor design is a PWR with a power rating of 3400 megawatts thermal (MWt) and electrical output of at least 1000 megawatts electric (MWe). The AP1000 design uses safety systems that rely on passive means, such as gravity, natural circulation, condensation and evaporation, and stored energy, for accident prevention and mitigation.

In developing the FSER for Turkey Point Units 6 and 7, the staff reviewed the AP1000 DCD to ensure that the combination of the DCD and the COL application represents the complete scope of information relating to a particular review topic.

The Turkey Point COL application is organized as follows:

- **Part 1 General and Financial Information**

Part 1 provides an introduction to the application and includes certain corporate information regarding FPL pursuant to 10 CFR 50.33(a) through 10 CFR 50.33(d).

- **Part 2 Final Safety Analysis Report**

Part 2 includes information pursuant to the requirements of 10 CFR 52.79, “Contents of applications; technical information in final safety analysis report” and, in general, adheres to the content and format guidance provided in Regulatory Guide (RG) 1.206, “Combined License Applications for Nuclear Power Plants (LWR Edition).”

- **Part 3 Environmental Report**

Part 3 includes environmental information pursuant to the requirements of 10 CFR 52.80, “Contents of applications; additional technical information,” and 10 CFR 51.50(c).

- **Part 4 Technical Specifications**

Part 4 addresses how the AP1000 Generic Technical Specifications (GTS) and Bases are incorporated by reference into the Turkey Point Plant-Specific Technical Specifications (PTS) and Bases. Specifically, Section A addresses completion of bracketed information. Section B provides a complete copy of the Turkey Point PTS and Bases.

- **Part 5 Emergency Plan**

Part 5 includes the Turkey Point COL Emergency Plan, supporting information (e.g., evacuation time estimates (ETEs)), and applicable offsite State and local emergency plans.

- **Part 6 Limited Work Authorization (Revision 1)**

Part 6 of the COL application, Revision 0, included a site redress plan and environmental report related to a Limited Work Authorization (LWA) request to perform certain safety-related construction activities. Subsequently, the applicant withdrew its LWA request. As such, Part 6 of the COL application is not used.

- **Part 7 Departure and Exemption Requests**

Part 7 includes information regarding departures and exemptions. Departures refers to departures from the AP1000 DCD, Revision 19, incorporated by reference into the COL application. For each departure, Part 7 of the COL application identifies the portions of the DCD and FSAR affected and includes a description, a justification, an evaluation against criteria in 10 CFR 52.63(b), and a concluding statement about whether the departure requires NRC approval under 10 CFR Part 52, Appendix D, Section VIII.B.5.

“Exemptions” refers to requests for exemptions from NRC regulations. For each exemption request, Part 7 identifies the regulation and specific wording from which the applicant is requesting an exemption and provides a discussion supporting the request.

- **Part 8 Safeguards/Security Plans**

Part 8 addresses the Turkey Point Units 6 and 7 Safeguards/Security Plan, which consists of the Physical Security Plan (PSP), the Training and Qualification Plan, the Safeguards Contingency Plan, and the Special Nuclear Material (SNM) Physical Protection Program Description. These security plans are submitted to the NRC as separate licensing documents in order to fulfill the requirements of 10 CFR 52.79(a)(35) and 10 CFR 52.79(a)(36). These plans are categorized as security safeguards information and are withheld from public disclosure pursuant to 10 CFR 73.21, “Protection of Safeguards Information: Performance Requirements,” and 10 CFR 73.22, “Protection of Safeguard Information: Specific Requirements.” The staff’s evaluation of the Safeguards and Security Plans is documented separately from this SER and is withheld from the public in accordance with 10 CFR 73.21. Section 13.6 of this SER provides a nonsensitive summary of the staff’s evaluation of those plans.

- **Part 9 Withheld Information**

Part 9 identifies sensitive information that is withheld from public disclosure under 10 CFR 2.390, “Public inspections, exemptions, requests for withholding.” The information in this part includes sensitive unclassified non-safeguards information (SUNSI), proprietary financial information, and figures from Part 2 of the application that meet the SUNSI guidance for withholding from the public. In addition, this part of the application includes the following information:

- Turkey Point Units 6 and 7 offsite emergency response plans;
- Turkey Point Units 6 and 7 Cyber Security Plan, as required by 10 CFR 73.54, “Protection of Digital Computer and Communication Systems and Networks”;
- Turkey Point Units 6 and 7 Mitigative Strategies Description and Plans, as required by 10 CFR 52.80(d); and
- Turkey Point Site-Specific Seismic Evaluation Report.

- **Part 10 Proposed Combined License Conditions (Including Inspections, Tests, Analyses, and Acceptance Criteria)**

Part 10 includes Turkey Point applicant’s proposed license conditions and inspections, tests, analyses, and acceptance criteria (ITAAC) information in accordance with 10 CFR 52.80. A table identifying the staff’s proposed license conditions and ITAAC appears in Appendix A of this FSER.

Part 10 of the application incorporated by reference the AP1000 Tier 1 information including ITAAC. In addition, the application includes site-specific ITAAC (e.g., emergency planning, physical security, electrical, and piping).

- **Part 11 Enclosures**

Part 11 includes information submitted by the applicant in support of the Turkey Point COL application. Specifically, these sections include the following:

- New Nuclear Projects Quality Assurance Program Description (QAPD). The QAPD is the top-level policy document that establishes the quality assurance (QA) policy and assigns major functional responsibilities for COL/construction/preoperation and operation activities conducted by or for FPL.
- Geotechnical Exploration and Testing.
- Cyber Security Plan. The SUNSI version of the cyber security plan is provided in Part 9 of the application.
- Mitigative Strategies Description and Plans. The SUNSI version of the Mitigative Strategies Description and Plans is provided in Part 9 of the application.
- SNM Material Control and Accounting Program Description.
- New Fuel Shipping Plan.
- Supplemental Information in Support of 10 CFR Part 70 Special Nuclear Material License Application.
- Supplemental Field Investigation Data Report.
- Surficial Muck Deposits Field and Laboratory Investigation Data Report.

1.2 Regulatory Basis

1.2.1 Applicable Regulations

The 10 CFR Part 52, Subpart C, “Combined Licenses,” sets out the requirements and procedures applicable to Commission issuance of a COL for nuclear power facilities. The following are of particular significance:

- 10 CFR 52.79, “Contents of applications; technical information in final safety analysis report,” identifies the technical information for the FSAR.
- 10 CFR 52.79(d) provides additional requirements for a COL referencing a standard certified design.
- 10 CFR 52.80, “Contents of applications; additional technical information,” provides additional technical information outside of the FSAR (ITAAC, environmental report, and mitigative strategies plan required by 10 CFR 50.54(hh)(2)).
- 10 CFR 52.81, “Standards for review of applications,” provides standards for reviewing the application.
- 10 CFR 52.83, “Finality of referenced NRC approvals; partial initial decision on site suitability,” provides for the finality of referenced NRC approvals (i.e., standard design certification (DC)).

- 10 CFR 52.85, “Administrative review of applications; hearings,” provides requirements for administrative reviews and hearings.
- 10 CFR 52.87, “Referral to the Advisory Committee on Reactor Safeguards (ACRS),” provides for referral to the ACRS.

The NRC staff reviewed this application according to the standards set out in:

- 10 CFR Part 20, “Standards for Protection Against Radiation.”
- 10 CFR Part 30, “Rules of General Applicability to Domestic Licensing of Byproduct Material.”
- 10 CFR Part 40, “Domestic Licensing of Source Material.”
- 10 CFR Part 50, “Domestic Licensing of Production and Utilization Facilities.”
- 10 CFR Part 51, “Environmental Protection Regulations for Domestic Licensing and Related Regulatory Functions.”
- 10 CFR Part 52, “Licenses, Certifications, and Approvals for Nuclear Power Plants.”
- 10 CFR Part 55, “Operators’ Licenses.”
- 10 CFR Part 70, “Domestic Licensing of Special Nuclear Material.”
- 10 CFR Part 73, “Physical Protection of Plants and Materials.”
- 10 CFR Part 74, “Material Control and Accounting of Special Nuclear Material.”
- 10 CFR Part 100, “Reactor Site Criteria.”
- 10 CFR Part 140, “Financial Protection Requirements and Indemnity Agreements.”

The staff evaluated the application against the acceptance criteria provided in the following NUREGs:

- NUREG-0800, “Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants (LWR Edition).”
- NUREG-1555, “Standard Review Plans for Environmental Reviews for Nuclear Power Plants.”
- NUREG-1577, “Standard Review Plan on Power Reactor Licensee Financial Qualifications and Decommissioning Funding Assurance.”
- “Standard Review Plan on Foreign Ownership, Control, or Domination”

In addition, the staff considered the format and content guidance in RG 1.206¹ for the COL application.

1.2.2 Finality of Referenced NRC Approvals

In accordance with 10 CFR 52.83, if the application for a COL references a DC rule, the scope and nature of matters resolved in the DC for the application and any COL issued are governed by 10 CFR 52.63, "Finality of standard design certifications."

Based on the finality afforded to referenced certified designs, the scope of this COL application review, as it relates to the referenced certified design, is limited to items that fall outside the scope of the certified design (e.g., COL information items, design information replacing conceptual design information (CDI), programmatic elements that are the responsibility of the COL, and departures from the certified design).

The certified AP1000 design currently incorporated by reference in the Turkey Point Units 6 and 7 COL application is in 10 CFR Part 52, Appendix D, and is based on the AP1000 DCD as amended through Revision 19. The results of the NRC staff's technical evaluation of the AP1000 DC application are documented in NUREG-1793, "Final Safety Evaluation Report Related to Certification of the AP1000 Standard Design," and its supplements.

The contents of the AP1000 COL application are specified by 10 CFR 52.79(a), which requires the submission of information within the FSAR that describes the facility, presents the design bases and the limits on its operation, and presents a safety analysis of the structures, systems, and components (SSC) of the facility as a whole. For a COL application that references a DC, 10 CFR 52.79(d) requires the DCD to be included or incorporated by reference into the FSAR. A COL application that references a certified design must also include the information and analysis required to be submitted within the scope of the COL application, but which is outside the scope of the DCD. This set of information addresses plant- and site-specific information and includes all COL action or information items, design information replacing CDI, and programmatic information that was not reviewed and approved in connection with the DC rulemaking.

During its evaluation of the COL application, the staff confirmed that the complete set of information required to be addressed in the COL application was addressed in the DC, the DC as supplemented by the COL application, or completely in the COL application. Following this confirmation, the staff's review of the COL application is limited to the COL-specific review items.

1.2.3 Overview of the Design-Centered Review Approach

The design-centered review approach (DCRA) is described in Regulatory Issue Summary (RIS) 2006-06, "New Reactor Standardization Needed to Support the Design-Centered Licensing Review Approach." The DCRA is endorsed by the Commission's Staff Requirements

¹ Appendix D, Section IV.A.2.a to 10 CFR Part 52 requires the COL application to include a plant-specific DCD that includes the same type of information and uses the same organization and numbering as the generic DCD. The generic DCD used RG 1.70, Revision 3, "Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants (LWR Edition)," as a guide for the format and content. RG 1.206 was issued after the initial certification of the AP1000; thus, there are anticipated differences between the Turkey Point COL application and the guidance of RG 1.206.

Memorandum (SRM) SECY-06-0187, "Semiannual Update of the Status of New Reactor Licensing Activities and Future Planning for New Reactors," dated November 16, 2006. The DCRA, which is the Commission's policy intended to promote standardization of COL applications, is beyond the scope of information included in the DC. This policy directs the staff to perform one technical review for each standard issue outside the scope of the DC, and use this decision to support decisions on multiple COL applications. In this context, "standard" refers to essentially identical information. In some cases, the staff has expanded the use of this standard approach to other areas with essentially identical information for regulatory purposes. For example, the cyber security plans for the AP1000 COL applicants are essentially identical with the exception of title names being different. Other areas where this approach was used include technical specifications and loss of large-area fire reviews and may include information provided by the applicant(s) to resolve plant-specific issues.

The first COL application submitted for NRC staff review is designated in a design center as the reference COL (RCOL) application, and the subsequent applications in the design center are designated as subsequent COL (SCOL) applications. The Turkey Point Units 6 and 7 COL application has been designated as an SCOL application in the AP1000 design center.²

FPL, as an SCOL applicant in the AP1000 design center, organized and annotated its FSAR, Part 2 of the COL application, to clearly identify the following: a) sections that incorporate by reference the AP1000 DCD, b) sections that are standard for COL applicants in the AP1000 design center, and c) sections that are site-specific and thus only apply to Turkey Point Units 6 and 7. The applicant has used the following notations for the departures from and/or supplements to the referenced DCD included in this COL application:

- STD—standard (STD) information that is identical in each COL referencing the AP1000.
- PTN—plant-specific information that is specific to this application.
- DEP—represents a departure (DEP) from the DCD.
- COL—represents a COL information item identified in the DCD.
- SUP—represents information that supplements (SUP) information in the DCD.
- CDI—represents design information replacing CDI included in the DCD but not addressed within the scope of the DCD review.

The following text is added to the technical evaluation sections in this SER whenever the staff uses standard content evaluation material to resolve departures and/or supplements to the referenced DCD:

² In a letter dated April 28, 2009, the NuStart Energy Development, LLC, consortium informed the NRC that it had changed the RCOL designation for the AP1000 design center from Bellefonte Nuclear Plant (BLN) Units 3 and 4 to the Vogtle Electric Generating Plant (VEGP) Units 3 and 4. The transition of the RCOL from BLN Units 3 and 4 to VEGP Units 3 and 4 occurred after the issuance of the BLN Units 3 and 4 safety evaluation (SE) with open items. As part of the transition, the NRC staff concluded that the BLN evaluation material identified as Standard (STD COL, STD SUP, STD DEP, and Interfaces for Standard Design) in the BLN SE was directly applicable to the VEGP review. As a result, standard content material from the SE for the RCOL (VEGP) application and referenced in the Turkey Point FSER includes evaluation material from the SE for the BLN COL application.

Section 1.2.3 of this SER provides a discussion of the strategy used by the NRC to perform one technical review for each standard issue outside the scope of the DC and use this review in evaluating subsequent COL applications. To ensure that the staff's findings on standard content that were documented in the SER for the reference COL application (Vogtle Electric Generating Plant [VEGP] Units 3 and 4) were equally applicable to the Turkey Point Units 6 and 7 COL application, the staff undertook the following reviews:

- The staff compared the VEGP COL FSAR, Revision 5 to the Turkey Point Units 6 and 7 COL FSAR. In performing this comparison, the staff considered changes made to the Turkey Point COL FSAR (and other parts of the COL application, as applicable) resulting from requests for additional information (RAIs).
- The staff confirmed that all responses to RAIs identified in the corresponding standard content evaluation were endorsed.
- The staff verified that the site-specific differences were not relevant.

The staff has completed its review and found the evaluation performed for the standard content to be directly applicable to the Turkey Point COL application. This standard content material is identified in this SER by use of italicized, double-indented formatting. Section 1.2.3 of this SER provides an explanation of why the standard content material from the SER for the reference COL application (VEGP) includes evaluation material from the SER for the BLN Units 3 and 4 COL application.

To support the text added to the technical evaluation sections as described above, the staff evaluated any differences between the information provided by the Turkey Point applicant and that provided by the VEGP applicant regarding details in the application for the standard content material to determine whether the standard content material of the VEGP SER is still applicable to the Turkey Point application. These evaluations are in the SER sections that reference the standard content.

The staff applied the DCRA, described above, in Chapter 21 of this SER in conducting its evaluation of the five requests by the applicant to depart from the AP1000 certified design. These five departure requests were identical to departure requests in the Levy Nuclear Plant (LNP) COL review. Therefore, consistent with the NRC's DCRA, the staff referenced evaluations that were completed for the first time in the LNP review. The referenced evaluations in Chapter 21 of this SER are captured by use of italicized, double-indented formatting.

The staff performed a complete comparison of the standard content appearing in the VEGP COL FSAR, Revision 5, to the Turkey Point Units 6 and 7 FSAR, Revision 8. The staff confirmed that standard content items were incorporated in the Turkey Point Units 6 and 7 application and that the changes discussed in the standard content items were equally applicable to Turkey Point Units 6 and 7 COL FSAR. For departures reviewed in Chapter 21 of this SER, the staff also compared information in the LNP COL application with corresponding

information in the Turkey Point COL application and confirmed that the information was equally applicable to Turkey Point Units 6 and 7.

1.3 Principal Review Matters

The staff's evaluations related to the COL application review are addressed as follows:

- **Part 1 General and Financial Information**

The staff's evaluation of the corporate information regarding FPL pursuant to 10 CFR 50.33, "Contents of applications; general information," is provided in Section 1.5.1 of this SER.

- **Part 2 Final Safety Analysis Report**

The staff's evaluation of information in the Turkey Point COL FSAR is provided in the corresponding sections of this SER. The NRC staff has issued the following two SER chapters that do not have a corresponding chapter in the Turkey Point COL FSAR:

- Chapter 20, "Requirements Resulting from Fukushima Near-Term Task Force [NTTF] Recommendations," which describes the staff's evaluations and conclusions relating to the Fukushima NTTF recommendations that are applicable to the Turkey Point Units 6 and 7 COL application.
- Chapter 21, "Design Changes Proposed In Accordance with ISG-11," which describes the staff's evaluations and conclusions for departures from the certified design identified by the applicant in accordance with Interim Staff Guidance DC/COL ISG-011, "Finalizing Licensing-Basis Information."

- **Part 3 Environmental Report**

The staff's evaluation of information in an environmental report submitted pursuant to the requirements of 10 CFR 51.50(c) is provided in the Environmental Impact Statement.

- **Part 4 Technical Specifications**

Chapter 16 of this SER includes the staff's evaluation of the Turkey Point Units 6 and 7 PTS and Bases (specifically completion of bracketed text).

- **Part 5 Emergency Plan**

Chapter 13 of this SER includes the staff's evaluation of the Turkey Point Emergency Plan, supporting information such as ETEs, and the applicable offsite State and local emergency plans.

- **Part 6 Limited Work Authorization**

Part 6 of the application is not used and, therefore, has no corresponding staff evaluation.

- **Part 7 Departures and Exemption Requests**

The staff's evaluation of the departures and exemptions in Part 7 is provided in the applicable chapter of this SER. Table 1-1, below, lists the departures identified in the application and identifies where the evaluation is addressed in this SER. Several of the departures, as marked, correspond to exemptions requested by the applicant.

Table 1-1. Departures Identified in Part 7 of the COL Application

Description of Departure	Location of Evaluation in this Report
STD DEP 1.1-1. Administrative departure for organization and numbering for the FSAR sections ³	1.5.4
PTN DEP 1.8-1. Correction of an inconsistency in regulatory citation in an interface description	1.5.4
PTN DEP 2.0-1. Revision of operating basis wind speed	2.0.4
PTN DEP 2.0-2. Revision of maximum normal wet bulb (noncoincident) air temperature	2.0.4
PTN DEP 2.0-3. Revision of maximum safety wet bulb (noncoincident) air temperature ³	2.0.4
PTN DEP 2.0-4. Revision of population distribution exclusion area (site)	2.0.4
PTN DEP 3.2-1. Addition of downspouts and downspout screens to the condensate return portion of the Passive Core Cooling System	21.1
PTN DEP 3.11-1. Revision of "Envir. Zone" numbers for Spent Fuel Pool Level instruments	3.11
PTN DEP 6.2-1. Revision of ITAAC Acceptance Criteria for the in-containment Passive Core Cooling System compartment vents to reflect the current plant configuration ³	21.4
PTN DEP 6.3-1. Quantification of term "indefinitely" as used in the AP1000 DCD for maintenance of safe shutdown conditions using the Passive Residual Heat Removal Heat Exchanger during non-LOCA accidents ³	21.1
PTN DEP 6.4-1. Revision of estimated maximum doses to control room operators to meet 10 CFR Part 50, Appendix A, General Design Criterion 19, "Control Room" ³	21.2
PTN DEP 6.4-2. Revision of heat generated in the control room during accident conditions and the conditions for actuating the normal ventilation system supplemental filtration and the emergency ventilation system ³	21.3
PTN DEP 7.3-1. Modification to the engineered safety features to provide an operating bypass for the boron dilution block to meet the requirements of IEEE 603-1991, "Standard Criteria for Safety Systems for Nuclear Power Generating Stations," in accordance with 10 CFR 50.55a(h), Protection and safety systems." ³	21.5

³ These departures correspond to exemptions requests, listed below in Table 1-2.

Description of Departure	Location of Evaluation in this Report
STD DEP 8.3-1. Revision of Class 1E voltage regulating transformer current limiting features	8.3.2
PTN DEP 18.8-1. Revision of operations support center location	18.8.4
PTN DEP 18.8-2. Revision of technical support center location	18.8.4
PTN DEP 19.58-1. Revision of description of severe winds and tornadoes	19.58

Part 7 of the COL application, Part B, requests nine exemptions, as listed in Table 1-2.

Table 1-2. Exemption Requests Identified in Part 7 of the COL Application

Description of Exemption	Location of Evaluation in this Report
Exemption from 10 CFR Part 52, Appendix D, Section IV.A.2.a related to COL application organization and numbering	1.5.4
Exemption from maximum safety wet bulb (noncoincident) air temperature cited in in AP1000 DCD Tier 1, Table 5.0 1	2.0.4
Exemption from the requirements of 10 CFR 70.22(b), 10 CFR 70.32(c), 10 CFR 74.31, 10 CFR 74.41, and 10 CFR 74.51, for SNM Material Control and Accounting Program Description	1.5.4
Exemption from AP1000 DCD Tier 1 Tables 2.2.3-1 and 2.2.3-2 and Technical Specification (TS) Surveillance Requirement (SR) 3.5.4.7 related to Containment Cooling Changes in regard to Passive Core Cooling System Condensate Return	21.1
Exemption from AP1000 DCD Tier 1 Subsection 2.7.1 and Tables 2.2.5-1 and 2.2.5-5 and TS Limiting Condition for Operation 3.7.4 and TS SR 3.7.4.1 related to Main Control Room Dose	21.2
Exemption from AP1000 DCD Tier 1 Tables 2.2.5-1, 2.2.5-4, 2.5.2-3 and 2.5.2-4, and TS 3.3.2 and 3.7.6 related to Main Control Room Heatup	21.3
Exemption from AP1000 Tier 1 Table 2.3.9-3 related to Combustible Gas Control in Containment	21.4
Exemption from AP1000 TS Table 3.3.2-1 related to Source Range Neutron Flux Doubling Block Permissive	21.5
Exemption from 10 CFR 52.93(a)(1) ⁴	1.5.4

- **Part 8 Safeguards/Security Plans**

The staff's evaluation of the Safeguards and Security Plans is documented separately from this SER and is withheld from the public in accordance with 10 CFR 73.21 and 10 CFR 73.22. A

⁴ Part 7 of the Turkey Point Units 6 and 7 COL application does not include an exemption request related to the requirements found in 10 CFR 52.93(a)(1). As discussed in Section 1.5.4 of this report, the staff determined that an exemption from this regulation is necessary.

non-sensitive summary of the staff's evaluation of those plans is provided in Section 13.6 of this SER.

The staff's evaluation of the SNM Physical Protection Plan is documented in Section 1.5.5.1 of this SER.

- **Part 9 Withheld Information**

The staff's evaluation of the withheld information occurs in the context of the specific subject being reviewed and is documented accordingly. A summary of the staff's evaluation of the Mitigative Strategies Description and Plans for loss of large areas of the plant due to explosions or fires is provided in Appendix 19A of this SER. The staff's complete evaluation is documented separately from this SER and is withheld from the public in accordance with 10 CFR 2.390.

The staff's evaluation of the Turkey Point Units 6 and 7 Cyber Security Plan is provided in Section 13.8 of this SER.

- **Part 10 Proposed Combined License Conditions (Including ITAAC)**

The staff's evaluation of the proposed COL conditions and ITAAC is provided in the applicable SER chapters. Appendix A identifies the proposed license conditions and ITAAC and the location of the evaluations. Each license condition is sequentially numbered in individual chapters of this SER. The license conditions and ITAAC are based on the provisions of 10 CFR 52.97, "Issuance of combined license."

- **Part 11 Enclosures**

Part 11 includes enclosures submitted by the applicant in support of the Turkey Point Units 6 and 7 COL application.

Organization of the SER

The staff's SER is structured as follows:

- The SER adheres to the finality afforded to COL applications that incorporate by reference a standard certified design. As such, this SER does not repeat any technical evaluation of material incorporated by reference; rather, it points to the corresponding review findings of NUREG-1793 and its supplements. However, the referenced DCD and the Turkey Point COL FSAR are considered in the staff's SER to the extent necessary to ensure that the expected scope of information to be included in a COL application is addressed adequately in either the DCD or COL FSAR or in both.
- For sections that were completely incorporated by reference without any supplements or departures, the SER simply points to the DCD and related staff evaluation in NUREG-1793 and its supplements and confirms that all the relevant review items were addressed in the AP1000 DCD.

- For subject matter within the scope of the COL application that supplements or departs from the DCD, this SER generally follows a six-section organization as follows:
 - “Introduction” section provides a brief overview of the specific subject matter.
 - “Summary of Application” section identifies whether portions of the review have received finality and clearly identifies the scope of review for the COL.
 - “Regulatory Basis” section identifies the regulatory criteria for the information addressed by the COL application.
 - “Technical Evaluation” section focuses on the information addressed by the COL application.
 - “Post Combined License Activities” section identifies the proposed license conditions, ITAAC, or FSAR information commitments that are post-COL activities.
 - “Conclusion” section summarizes how the technical evaluation resulted in a reasonable assurance determination by the staff that the relevant acceptance criteria have been met.

1.4 Staff Review of Turkey Point Units 6 and 7 COL FSAR Chapter 1

1.4.1 Introduction

There are two types of information provided in Chapter 1 of the Turkey Point COL FSAR:

- General information that enables the reviewer or reader to obtain a basic understanding of the overall facility without having to refer to the subsequent chapters. A review of the remainder of the application can then be completed with a better perspective and recognition of the relative safety significance of each individual item in the overall plant description.
- Specific information relating to qualifications of the applicant, construction impacts, and regulatory considerations that applies throughout the balance of the application (e.g., conformance with the acceptance criteria in NUREG-0800).

This section of the SER will identify the information incorporated by reference, summarize all of the new information provided, and document the staff’s evaluation of the sections addressing regulatory considerations.

1.4.2 Summary of Application

The information related to COL/SUP items included in Chapter 1 of the Turkey Point Units 6 and 7 COL FSAR encompasses the statements of fact or information recommended by RG 1.206. No staff technical evaluation was necessary where the statements were strictly background information. However, where technical evaluation of these COL/SUPs was necessary, the evaluation is not in this SER section, but in subsequent sections as referenced below.

FSAR Section 1.1, "Introduction"

Section 1.1 of the Turkey Point COL FSAR, Revision 8, incorporates by reference Section 1.1, "Introduction," of the AP1000 DCD, Revision 19, with the following supplements, and Includes proprietary information and safeguards information referenced in the AP1000 DCD.

- STD SUP 1.1-1

The applicant specified the incorporation of Revision 19 of the Westinghouse AP1000 DCD in all sections of the Turkey Point COL FSAR. Additionally, the applicant incorporated by reference Nuclear Energy Institute (NEI) technical reports as identified in Table 1.6-201, "Additional Material Referenced," of the Turkey Point COL FSAR.

- PTN SUP 1.1-2

The applicant clarified that the FSAR was submitted to NRC by FPL under Section 103 of the AEA to construct and operate two nuclear power plants under the provisions of 10 CFR Part 52, Subpart C, "Combined Licenses."

- PTN COL 2.1-1

The applicant provided additional information in Turkey Point COL 2.1-1 to address COL Information Item 2.1-1 (COL Action Item 2.1.1-1). Specifically, Turkey Point Units 6 and 7 are to be located in Miami-Dade County, FL approximately 25 miles south of Miami, FL. This is a brief introductory summary of the plant location. An expanded discussion of PTN COL 2.1-1 is included in Turkey Point COL FSAR Section 2.1.

- PTN COL 1.1-1

The applicant provided the anticipated schedule for construction and operation of Turkey Point Units 6 and 7 in Turkey Point COL FSAR Table 1.1-203, "Schedule for Construction and Operation of Units 6 & 7." The applicant committed to provide a site-specific construction plan and startup schedule after issuance of the COL.

- STD SUP 1.1-6

The applicant identified that, while the Turkey Point COL FSAR generally follows the AP1000 DCD organization and numbering, there were some organization and numbering differences that were adopted, where necessary, to include additional material, such as additional content identified in RG 1.206.

Related to this is STD DEP 1.1-1, "Administrative departure for organization and numbering of the FSAR sections," in Turkey Point COL FSAR Section 1.8 and Part 7 of the Turkey Point COL application. The staff's evaluation of this departure is included in Section 1.5.4 of this SER.

- STD SUP 1.1-3

The applicant provided additional information to describe annotations used in the left hand column of the Turkey Point COL FSAR to identify departures, supplementary information, COL items, and CDI.

- STD SUP 1.1-4

The applicant provided additional information to indicate how proprietary, personal, or sensitive information and withheld from public disclosure pursuant to 10 CFR 2.390 and RIS 2005-026, "Control of Sensitive Unclassified Nonsafeguards Information Related to Nuclear Power Reactors," is identified in the Turkey Point COL FSAR. Proprietary and sensitive material was provided in Part 9 of the COL application.

- PTN SUP 1.1-5

The applicant provided additional information to identify acronyms and abbreviations used in the Turkey Point COL FSAR that are in addition to the acronyms identified in the AP1000 DCD.

FSAR Section 1.2, General Plant Description

Section 1.2 of the Turkey Point COL FSAR, Revision 8, incorporates by reference Section 1.2, "General Plant Description," of the AP1000 DCD, Revision 19 with the following departures and supplements:

- PTN DEP 18.8-1

The applicant modified the AP1000 DCD Figure 1.2-18, "Annex Building General Arrangement Plan at Elevation 100'-0" & 107'-2"," to reflect the proposed relocation of the Operations Support Center (OSC). The staff's evaluation of the location of the OSC is discussed in Section 18.8.4 of this SER.

- PTN COL 2.1-1; PTN COL 3.3-1; and PTN COL 3.5-1

The applicant provided additional information on the site plan for Turkey Point Units 6 and 7 summarizing the principal structures and facilities, parking areas, and roads. The applicant also describes the location and orientation of the power block complex. These COL information items are expanded in other sections of the Turkey Point COL FSAR.⁵

FSAR Section 1.3, Comparisons with Similar Facility Designs

Section 1.3 of the Turkey Point COL FSAR, Revision 8, incorporates by reference Section 1.3, "Comparisons with Similar Facility Designs," of the AP1000 DCD, Revision 19 with no departures or supplements.

⁵ Table 1.8-202 of the Turkey Point COL FSAR provides a COL information item index of occurrences in the Turkey Point COL FSAR.

FSAR Section 1.4, Identifying Agents and Contractors

Section 1.4 of the Turkey Point COL FSAR, Revision 8, incorporates by reference Section 1.4, "Identification of Agents and Contractors," of the AP1000 DCD, Revision 19 with the following departures and/or supplements:

- PTN SUP 1.4-1

The applicant provided additional information to identify FPL as the COL applicant for Turkey Point Units 6 and 7. Additionally, the applicant identified FPL as the owner and operator of Turkey Point Units 6 and 7.

- PTN SUP 1.4-2

The applicant states that contractors participating in preparing the COL application are addressed in Subsection 1.4.2.8 of the Turkey Point COL FSAR. However, the applicant has not yet contracted the nuclear steam supply system (NSSS) provider, the architect-engineer, and the constructor. Subsection 1.4.2.8 of the FSAR will be revised to include information identifying the NSSS provider, the architect-engineer, and the constructor.

- PTN SUP 1.4-3

The applicant provided additional information related to specialized consulting firms that assisted in preparing the COL application.

FSAR Section 1.5, Requirements for Further Technical Information

Section 1.5 of the Turkey Point COL FSAR, Revision 8, incorporates by reference Section 1.5, "Requirements for Further Technical Information," of the AP1000 DCD, Revision 19 with no departures or supplements. This section of the DCD provides information related to testing conducted during the AP600 conceptual design program to provide input into the plant design and to demonstrate the feasibility of unique design features. The DCD also describes the analyses performed to show that the AP600 and AP1000 exhibit a similar range of conditions such that the AP600 tests are sufficient to support the AP1000 safety analysis.

FSAR Section 1.6, "Material Referenced"

Section 1.6 of the Turkey Point COL FSAR, Revision 8, incorporates by reference Section 1.6, "Material Referenced," of the AP1000 DCD, Revision 19 with the following supplements:

- STD SUP 1.6-1

The applicant identified Table 1.6-201 as providing a list of the technical documents incorporated by reference in the Turkey Point COL FSAR in addition to those technical documents incorporated by reference in the AP1000 DCD.

- PTN SUP 1.6-2

The applicant identified supplemental portions of Table 1.6-201 as site specific and identified them as PTN SUP 1.6-2.

- PTN DEP 6.4-1

The applicant provided information about PTN DEP 6.4-1 in Section 1.6 of the FSAR related to design changes affecting habitability of the main control room and changes to the calculated doses to control room operators. This information, as well as related PTN DEP 6.4-1 information appearing in other chapters of the FSAR, is reviewed in Section 21.2 of this report.

FSAR Section 1.7, "Drawings and Other Detailed Information"

Section 1.7 of the Turkey Point COL FSAR, Revision 8, incorporates by reference Section 1.7, "Drawings and Other Detailed Information," of the AP1000 DCD, Revision 19, with the following supplements:

- PTN SUP 1.7-1

The applicant identified the site-specific piping and instrumentation diagrams or system drawings. These are the circulating water system, raw water system, and transmission switchyard and offsite power system.

FSAR Section 1.8, Interfaces for Standard Design

Section 1.8 of the Turkey Point COL FSAR, Revision 8, incorporates by reference Section 1.8, "Interfaces for Standard Design," of the AP1000 DCD, Revision 19 with the following departures and/or supplements:

- PTN DEP 1.8-1

The applicant provided a departure to address an error in the DCD Table 1.8-1 listing of plant interfaces where Item 13.1 incorrectly references Appendix O of 10 CFR Part 50. This departure is evaluated in Section 1.4 of this document.

- PTN SUP 1.8-1

The applicant identified departures in Turkey Point COL FSAR Table 1.8-201, "Summary of FSAR Departures from the DCD." The departures are listed above in Table 1-1.

- PTN SUP 1.8-2

The applicant provided a list of the COL information items in the AP1000 DCD. In Turkey Point COL FSAR Table 1.8-202, "COL Item Tabulation," FPL provides the sections of the application addressing these issues. The table further identifies the AP1000 COL items as an applicant item, a holder item, or both. An applicant item is completely addressed in the application. FPL's definition of a COL holder item is an item that cannot be resolved prior to issuance of the COL. These items are regulatory commitments of the COL holder and will be completed as specified in the appropriate section of the referenced DCD and their completion is the subject of a COL license condition presented in Part 10 of this COL application.

- PTN SUP 1.8-3

The applicant provided in Turkey Point COL FSAR Table 1.8-203, "Summary of FSAR Discussions of AP1000 Plant Interfaces," a list of interface items from the AP1000 DCD and the corresponding Turkey Point COL FSAR section(s) that address those interface items.

FSAR Section 1.9, Compliance with Regulatory Criteria

Section 1.9 of the Turkey Point COL FSAR, Revision 8, incorporates by reference Section 1.9, "Compliance with Regulatory Criteria," of the AP1000 DCD, Revision 19 with the following supplements:

- STD COL 1.9-1 and PTN COL 1.9-1

The applicant provided additional information related to NRC regulatory guides cited in the Turkey Point COL FSAR. Table 1.9-201, "Regulatory Guide/FSAR Section Cross-References," identifies the regulatory guide revision and provides Turkey Point COL FSAR cross-references. In addition, Appendix 1AA, "Conformance with Regulatory Guides," was developed by the applicant to supplement the detailed discussion presented in Appendix 1A, "Conformance with Regulatory Guides," of the referenced DCD. Specifically, Appendix 1AA delineates conformance of design aspects as stated in the DCD and conformance with programmatic and/or operational issues as presented in the Turkey Point COL FSAR. In certain regulatory guides, design aspects were beyond the scope of the DCD and are presented in the Turkey Point COL FSAR.

- STD COL 1.9-2 and PTN COL 1.9-2

The applicant provided additional information related to operational experience. Turkey Point COL FSAR Table 1.9-204, "Generic Communications Assessment," provides a list of Bulletins and Generic Letters (GLs), the appropriate Turkey Point COL FSAR cross-references, and whether the subject matter was addressed in the AP1000 DCD.

- STD COL 1.9-3

The applicant provided additional information related to the review of Unresolved Safety Issues and Generic Safety Issues (GSIs). Specifically, Table 1.9-203, "Listing of Unresolved Safety Issues and Generic Safety Issues," lists Three Mile Island (TMI) Action Plan items, Task Action Plan items, New Generic Issues, Human Factors Issues, and Chernobyl Issues and states how they were considered in the AP1000 DCD and COL application. In addition, the applicant provided discussion on four new generic issues: Issue 186 related to heavy load drops, Issue 189 related to susceptibility of certain containments to early failure from hydrogen combustion, Issue 191 related to PWR sump performance, and Issue 196 related to the use of Boral in long-term dry storage casks for spent reactor fuel.

- STD SUP 1.9-1 and PTN SUP 1.9-1

The applicant provided additional information related to conformance with NUREG-0800. Specifically Turkey Point COL FSAR Table 1.9-202, "Conformance with SRP Acceptance Criteria," delineates conformance with NUREG-0800 for design aspects as stated in the

AP1000 DCD and conformance for subjects beyond the scope of the DCD as presented in the Turkey Point COL FSAR.

- PTN SUP 1.9-2

The applicant clarified that the severe accident mitigation design alternatives (SAMDA) evaluation for the AP1000 in Appendix 1B to the DCD is not incorporated into the Turkey Point COL FSAR; but is addressed in the COL application Environmental Report.

- STD SUP 1.9-3

The applicant provided information related to station blackout (SBO) procedures and training for operators to include actions necessary to restore offsite power after 72 hours by addressing alternating current (ac) power restoration and severe weather guidance in accordance with NUMARC-87-00, "Guidelines and Technical Bases for NUMARC Initiatives Addressing Station Blackout at Light-Water Reactors."

FSAR Section 1.10, Nuclear Power Plants to Be Operated On Multi-Unit Sites

The applicant identified this as a new section in the Turkey Point COL application that was not part of the referenced DCD.

- STD SUP 1.10-1

The applicant provided an assessment of the potential impacts of construction of one unit on SSCs important to safety for an operating unit, in accordance with 10 CFR 52.79(a)(31). This section addresses the review of an evaluation of potential hazards to the SSCs important to safety of the operating units resulting from construction activities, as well as a description of the managerial and administrative controls to be used to provide assurance that the limiting conditions for operation (LCOs) are not exceeded as a result of construction activities at a multi-unit site.

- PTN SUP 1.10-1

The applicant identified that the power blocks for Turkey Point Units 6 and 7 have a minimum separation of at least 850 feet between plant centerlines. The standard portion of the application discusses the primary consideration in setting this separation distance as the space needed to support plant construction via the use of a heavy-lift crane.

License Conditions

- The applicant proposed that the ITAAC identified in the tables in Appendix B of Part 10 of the Turkey Point COL application be incorporated into the COL.

1.4.3 Regulatory Basis

The regulatory basis of the information incorporated by reference is addressed in NUREG-1793 and its supplements.

In addition, the acceptance criteria associated with the relevant requirements of the Commission regulations for the introductory information in Turkey Point COL FSAR Chapter 1 are given in Section 1.0 of NUREG-0800.

The applicable regulatory requirements for the introductory information are as follows:

- 10 CFR 50.43(e), “Additional standards and provisions affecting class 103 licenses and certifications for commercial power,” as it relates to requirements for approval of applications for a DC, COL, manufacturing license, or operating license that propose nuclear reactor designs that differ significantly from light-water reactor (LWR) designs that were licensed before 1997, or use simplified, inherent, passive, or other innovative means to accomplish their safety functions.
- 10 CFR 52.77, “Contents of applications; general information,” and 10 CFR 52.79, as they relate to general introductory matters.
- 10 CFR 52.79(a)(17), as it relates to compliance with technically relevant positions of the TMI requirements.
- 10 CFR 52.79(a)(20), as it relates to proposed technical resolutions of those unresolved safety issues and medium- and high-priority GSIs that are identified in the version of NUREG-0933, “Resolution of Generic Safety Issues (Formerly entitled ‘A Prioritization of Generic Safety Issues’),” current on the date up to 6 months before the docket date of the application and, which are technically relevant to the design.
- 10 CFR 52.79(a)(31) regarding nuclear power plants to be operated on multi-unit sites, as it relates to an evaluation of the potential hazards to the SSCs important to safety of operating units resulting from construction activities, as well as a description of the managerial and administrative controls to be used to provide assurance that the LCOs are not exceeded as a result of construction activities at the multi-unit sites.
- 10 CFR 52.79(a)(37), as it relates to the information necessary to demonstrate how operating experience insights have been incorporated into the plant design.
- 10 CFR 52.79(a)(41), as it relates to an evaluation of the application against the applicable NRC review guidance in effect 6 months before the docket date of the application.
- 10 CFR 52.79(d)(2) requiring that, for a COL referencing a standard DC, the FSAR demonstrate that the interface requirements established for the design under 10 CFR 52.47, “Contents of applications; technical information,” have been met.
- 10 CFR 52.97(a)(1)(iv), “Issuance of combined licenses,” regarding technical and financial qualifications.

The related acceptance criteria from NUREG-0800, Chapter 1 are as follows:

- For regulatory considerations, acceptance is based on addressing the regulatory requirements as discussed in FSAR Chapter 1 or within the referenced FSAR section.

The NUREG-0800 acceptance criteria associated with the referenced section will be reviewed in the context of that review.

- For performance of new safety features, the information is sufficient to provide reasonable assurance that: (1) these new safety features will perform as predicted in the applicant's FSAR, (2) the effects of system interactions are acceptable, and (3) the applicant provides sufficient data to validate analytical codes. The design qualification testing requirements may be met with either separate effects or integral system tests; prototype tests; or a combination of tests, analyses, and operating experience.

In conformance with the regulatory acceptance criteria in RG 1.206 the applicant provided an evaluation for conformance with guidance in RGs in effect 6 months prior to the submittal of the COL application.

1.4.4 Technical Evaluation

The NRC staff reviewed Section 1 of the Turkey Point COL FSAR and checked the referenced DCD to ensure that the combination of the DCD and the COL application represents the complete scope of information relating to this review topic.⁶ The NRC staff's review confirmed that the information in the application and incorporated by reference addresses the required information relating to this introduction. The results of the NRC staff's evaluation of the information incorporated by reference in the Turkey Point COL application are documented in NUREG-1793 and its supplements.

Section 1.2.3 of this SER provides a discussion of the strategy used by the NRC to perform one technical review for each standard issue outside the scope of the DC and use this review in evaluating subsequent COL applications. To ensure that the staff's findings on standard content that were documented in the SER for the reference COL application (VEGP Units 3 and 4) were equally applicable to the Turkey Point Units 6 and 7 COL application, the staff undertook the following reviews:

- The staff compared the VEGP COL FSAR, Revision 5 to the Turkey Point COL FSAR. In performing this comparison, the staff considered changes made to the Turkey Point COL FSAR (and other parts of the COL application, as applicable) resulting from RAIs.
- The staff confirmed that all responses to RAIs identified in the corresponding standard content evaluation were endorsed.
- The staff verified that the site-specific differences were not relevant.

The staff has completed its review and found the evaluation performed for the standard content to be directly applicable to the Turkey Point COL application. This standard content material is identified in this SER by use of italicized, double-indented formatting. Section 1.2.3 of this SER provides an explanation of why the standard content material from the SER for the reference COL application (VEGP) includes evaluation material from the SER for the Bellefonte Nuclear Station (BLN) Units 3 and 4 COL application. Any confirmatory items in the standard content material retain the numbers assigned in the VEGP SER. Confirmatory items that are first

⁶ See Section 1.2.2, "Finality of Referenced NRC Approvals" for a discussion of the staff's review related to verification of the scope of information to be included within a COL application that references a DC.

identified in this SER section have a Turkey Point designation (e.g., Turkey Point Confirmatory Item 1.4-1).

The staff reviewed the information in the Turkey Point COL FSAR:

Turkey Point COL FSAR Sections 1.1, 1.2, and 1.3

There are no specific NUREG-0800 acceptance criteria related to the general information presented in Sections 1.1, 1.2, and 1.3, and no specific regulatory findings. The information provides the reader with a basic overview of the nuclear power plant and the construct of the Turkey Point COL FSAR, itself.

- PTN COL 1.1-1

In Turkey Point COL FSAR Section 1.1, PTN COL 1.1-1 states that a site-specific construction plan and startup schedule will be provided after issuance of the COL. This is identified as **Turkey Point Commitment Number 1.4-1**.

The following portion of this technical evaluation section is reproduced from Section 1.4.4 of the VEGP FSER:

*In a letter dated November 11, 2010, the applicant added a discussion of incorporation of the proprietary information and safeguards information referenced in the AP1000 DCD. This information is included to meet the requirements of 10 CFR Part 52, Appendix D, Section IV.A.3, which indicates the applicant must “include, in the plant specific DCD, the proprietary information and safeguards information referenced in the AP1000 DCD” and, therefore, is acceptable. The incorporation of the above information into a future revision of the VEGP COL FSAR is **Confirmatory Item 1.4-1**.*

Resolution of Standard Content Confirmatory Item 1.4-1

Confirmatory Item 1.4-1 is an applicant commitment to revise FSAR Section 1.1 to include a discussion of incorporation of the proprietary information and safeguards information referenced in the AP1000 DCD. The staff verified that the VEGP COL FSAR was appropriately revised. As a result, Confirmatory Item 1.4-1 is now closed.

Turkey Point COL FSAR Section 1.4

- PTN SUP 1.4-1

This evaluation is limited to FPL's technical qualification to hold a 10 CFR Part 52 license in accordance with 10 CFR 52.97(a)(1)(iv). The financial qualifications that are also a requirement of 10 CFR 52.97(a)(1)(iv) are evaluated in Section 1.5.1 of this SER.

Turkey Point COL FSAR Section 1.4 states that FPL will own and operate Turkey Point Units 6 and 7. Part 1 of the COL application, Section 1.0, states that FPL, the applicant for the Turkey Point Units 6 and 7 COLs, is primarily engaged in the generation, transmission, and distribution of electricity. The service territory covers the southern third and almost the entire

eastern seaboard of the state of Florida. FPL supplies electric service to approximately 4.5 million customer accounts. FPL owns and operates the following four nuclear power plants:

- St. Lucie Unit 1, near Ft. Pierce, FL.
- St. Lucie Unit 2, near Ft. Pierce, FL (85 percent ownership, FPL is authorized to act as agent for the Orlando Utilities Commission of the city of Orlando, FL and Florida Municipal Power Agency).
- Turkey Point Units 3 and 4, near Florida City, FL.

FPL is a regulated public utility and is subject to the regulatory provisions of the Florida Public Service Commission (FPSC), the NRC, and the Federal Energy Regulatory Commission. Since FPL holds 10 CFR Part 50 licenses for nuclear power plants and has demonstrated its ability to build and operate these plants, the staff finds that FPL is qualified to hold a 10 CFR Part 52 license. The staff notes that Section 17.5 of the Turkey Point COL FSAR discusses the QA program to be implemented at the receipt of the COL. The staff's evaluation of Section 17.5 of the Turkey Point COL FSAR is in Section 17.5 of this SER. Based on FPL's experience with building and operating a nuclear power plant and the staff's evaluation of FPL's QA program, the staff finds that FPL is technically qualified to hold a 10 CFR Part 52 license in accordance with 10 CFR 52.97(a)(1)(iv).

- PTN SUP 1.4-2 and PTN SUP 1.4-3

In PTN SUP 1.4-2 and PTN SUP 1.4-3, the applicant provided the names of contractors and description of the specialized services provided in the preparation of the COL application.

FPL received support from the following contractors in preparing the COL application:

- Bechtel Power Corporation
- Contingency Management Consulting Group, LLC
- Environmental Consulting & Technology, Inc.
- Golder Associates, Inc.
- KLD Associates, Inc.
- MACTEC Engineering and Consulting, Inc.
- McNabb Hydrogeologic Consulting, Inc.
- NuStart Energy, Inc.
- Risk Engineering, Inc.
- Tetra Tech NUS, Inc.
- William Lettis & Associates, Inc.
- Westinghouse Electric Company LLC
- AMEC Environmental & Infrastructure, Inc.
- Paul C. Rizzo Associates, Inc.
- Fugro Consultants, Inc.
- KLD Engineering, P.C.

The staff finds this acceptable because the applicant identified contractors beyond those identified in the DCD and provided a description of the specialized consulting services rendered in preparation of the COL application.

Turkey Point COL FSAR Section 1.5

The 10 CFR 50.43(e) requires additional testing or analysis for applications for a DC or COL that propose nuclear reactor designs that differ significantly from LWR designs that were licensed before 1997, or use simplified, inherent, passive, or other innovative means to accomplish their safety functions. This requirement was addressed in the AP1000 DCD and evaluated by the staff in NUREG-1793, Chapter 21, "Testing and Computer Code Evaluation." The Turkey Point COL application does not include any additional design features that require additional testing.

Turkey Point FSAR Section 1.6

In this section, the applicant provides a tabulation of all technical documents that are incorporated by reference as part of the application. There are no specific NUREG-0800 acceptance criteria related to the information presented in Section 1.6 and no specific regulatory findings. The staff finds that the applicant's supplemental information in FSAR Section 1.6 is acceptable within the review scope of Chapter 1 and satisfies RG 1.206, Regulatory Position C.I.1.6.

Turkey Point COL FSAR Section 1.7

In this section, the applicant identified the site-specific piping and instrumentation diagrams or system drawings cross-referenced to the related application sections. There are no specific NUREG-0800 acceptance criteria related to the information presented in Section 1.7 and no specific regulatory findings. The staff finds that the applicant's supplemental information in FSAR Section 1.7 is acceptable within the review scope of Chapter 1 and satisfies RG 1.206, Regulatory Position C.I.1.7.

Turkey Point COL FSAR Section 1.8

- PTN SUP 1.8-1

As discussed in SER Section 1.4.2, the applicant identifies departures in Turkey Point COL FSAR Table 1.8-201 from the referenced AP1000 DCD. Section 1.3 of this SER provides a cross-reference to where these departures are discussed in this SER.

- PTN SUP 1.8-2

PTN SUP 1.8-2 contains the same type of information as VEGP SUP 1.8-2. Therefore, the following portion of this technical evaluation section is reproduced from Section 1.4.4 of the VEGP FSER:

In Sections 1.3 and 1.4.4 of the BLN SER, the staff identified a standard content Open Item 1-2 related to the decision regarding which of the BLN COL FSAR commitments, if any should become a license condition. On January 21, 2010, the NRC issued ISG-15, "Final Interim Staff Guidance on the Post-Combined License Commitments," ESP/DC/COL-ISG-15. This guidance discusses options regarding completion of COL items that cannot be completed until after issuance of the COL. The VEGP applicant identified that certain COL information items

cannot be resolved prior to the issuance of a COL. The applicant has identified proposed License Condition 2 in Part 10 of the COL application to ensure these COL items will be completed by the identified implementation milestones through completion of the action identified. The determination that these COL information items cannot be resolved prior to issuance of a COL is discussed in the relevant SER section related to the topic. In addition, using the guidance of ISG-15, the staff has identified certain FSAR commitments in individual sections of this SER and these FSAR commitments are listed in Appendix A.3 of this SER. The staff considers Open Item 1-2 is resolved.

- PTN SUP 1.8-3

The AP1000 DCD Table 1.8-1 presents interface items for the AP1000. This section of the DCD identifies certain interfaces with the standard design that have to be addressed in accordance with 10 CFR 52.47(a)(1)(vii).⁷ As required by 10 CFR 52.79(d)(2), the COL application must demonstrate how these interface items have been met. In the Turkey Point COL FSAR, the applicant did not explicitly identify how these interface items have been met. Table 1.8-203 of the Turkey Point COL FSAR identifies the FSAR location of information addressing the interface items identified in Section 1.8 of the AP1000 DCD. The staff's review of the identified FSAR locations confirmed that interface items are adequately addressed in the Turkey Point COL FSAR. The technical discussions related to specific interface requirements are addressed in related sections of this SER (e.g., SER Sections 8.2.4 and 11.3.2).

- PTN DEP 1.8-1

This Tier 2 departure, appearing in the FSAR Table 1.8-203 listing of AP1000 plant interfaces, corrects an error in DCD Table 1.8-1, Item 13.1. This interface addresses the design features that affect plans for coping with emergencies in the operation of the reactor facility or a major portion thereof. The departure changes the incorrect regulatory reference from Appendix O of 10 CFR Part 50 to 10 CFR 52.137(a)(11). In issuing the final rule for 10 CFR Part 52 in the *Federal Register (FR)* (see 72 FR 49352), the requirement relating to providing this interface information was moved from Appendix O of 10 CFR Part 50 to a new location in 10 CFR 52.137 (see 72 FR 49391). The applicant determined that this departure did not involve a change to or departure from Tier 1 or Tier 2* information or the technical specifications, nor did it require a license amendment under 10 CFR Part 52, Appendix D, § VIII.B.5.b or c. Accordingly, the applicant determined that the departure does not require prior NRC approval. While such determinations are subject to staff inspection, such departures are not material to the NRC decision on the acceptability of a COL application, and staff review of the departure is not required.

Turkey Point COL FSAR Section 1.9

In this section of the application, the applicant demonstrates conformance with regulatory guides and NUREG-0800 and addresses unresolved safety issues, GSIs, TMI action items, and operating experience.

⁷ Following the update to 10 CFR Part 52 (72 FR 49517), this provision has changed to 10 CFR 52.47(a)(25).

The following portion of this technical evaluation section is reproduced from Section 1.4.4 of the VEGP FSER⁸:

AP1000 COL Information Item

- STD COL 1.9-1

Regarding RGs, the applicant provides in BLN COL FSAR Table 1.9-201 a cross-reference between the RG and where it is discussed in the application, and Appendix 1AA, "Conformance with Regulatory Guides," to supplement the detailed discussion presented in Appendix 1A, "Conformance with Regulatory Guides," of the referenced DCD. The technical discussions related to this appendix are addressed in the related technical sections of the BLN COL FSAR. In addition, BLN COL FSAR Table 1.9-201 provides a listing of all RGs, the specific revision, and provides BLN COL FSAR and DCD cross-references.

The staff issued three RAIs associated with how the RG information in Table 1.9-201 and Appendix 1AA of the BLN COL FSAR is presented. In addition, there were two specific RAIs associated with how an individual RG is discussed in Table 1.9-201 and Appendix 1AA. A description of the RAIs and their responses follows.

RAI 1-5

In RAI 1-5, the staff noted that BLN COL FSAR Appendix 1AA lists the later version of the RG when compared with DCD Table 1.9-1 but in some cases does not discuss compliance with the later version. In other cases, exceptions to the RG were identified but not justified.

RAI 1-7

In RAI 1-7, the staff noted that not all RGs listed in Appendix 1AA provided a cross-reference to where they were discussed in accordance with the guidance in Section 1 of NUREG-0800.

RAI 1-11

In RAI 1-11, the staff noted that the information that TVA provided in response to RAIs 1-5 and 1-7 conflicted with information that TVA provided in response to another RAI. TVA was requested to reconcile these differences.

RAIs 1-1 and 1-10

These RAIs are associated with specific RGs and RAI 1-1 and RAI 1-10 are evaluated in Chapters 13 and 12, of this SER, respectively.

⁸ The text reproduced from Section 1.4.4 of the VEGP is unaltered, but is presented in sequential order of the COL and SUP items.

In TVA's response to RAIs 1-5 and 1-7, TVA committed to make changes to BLN COL FSAR Table 1.9-201 and Appendix 1AA to:

- *Add an additional statement to Appendix 1AA that specifically addresses the later version of the RG.*
- *Revise BLN COL FSAR Sections 1.9.1.1, 1.9.1.2, 1.9.1.3, and 1.9.1.4, to reflect that one method of identifying and justifying an alternative to an RG is the use of previous revisions of the RG for design aspects as stated in the DCD in order to preserve the finality of the certified design.*
- *Revise BLN COL FSAR Table 1.9-201 to address the RG listed in Appendix 1AA, thereby providing a more complete cross reference of where each RG is discussed in the COL application.*

In response to RAI 1-11, TVA committed to revising BLN COL FSAR Table 1.9-201 and Appendix 1AA to ensure that they are consistent with commitments made in other RAI responses.

The staff's evaluation of the RGs is addressed in Chapters 2 through 19 of this SER as needed. At a minimum the NRC staff's FSER sections will discuss any RG that involves an exception.

The staff finds TVA's responses to RAIs 1-5 and 1-7 acceptable. However, the staff notes that BLN COL FSAR Table 1.9-201 and Appendix 1AA will most likely need additional changes based on the staff's evaluation of the RGs in this SER and TVA's response to RAI 1-11. The NRC staff is still evaluating TVA's response to RAI 1-11 and has not yet made a determination of whether the response is acceptable. This is Open Item 1.4-2. The updating of BLN COL FSAR Table 1.9-201 to reflect changes committed to by TVA in response to RAI 1-11 and the updating of this information to reflect TVA's commitments in other RAI responses is Confirmatory Item 1.4-2.

Resolution of Standard Content Confirmatory Item 1.4-2

The NRC staff verified that VEGP COL FSAR Table 1.9-201 was updated to provide an acceptable cross reference of where each RG is discussed in the COL application. As a result, Confirmatory Item 1.4-2 is resolved for VEGP.

Resolution of Standard Content Open Item 1.4-2

In a letter dated September 21, 2009, the VEGP applicant provided clarification to a previously submitted response dated January 27, 2009 from the BLN applicant. Specifically, the applicant proposed to revise the discussion in the "General comment" portion related to preserving the finality of the certified design in VEGP COL FSAR Sections 1.9.1.1, 1.9.1.2, 1.9.1.3, 1.9.1.4 and Appendix 1AA Note (b); to clarify in VEGP COL FSAR Section 17.5 the "DCD scope" and the "remaining scope" discussion for QA-related RGs (including RG 1.28; RG 1.30, Quality Assurance Requirements for the Installation, Inspection, and Testing of

Instrumentation and Electric Equipment (Safety Guide 30)”; RG 1.33, “Quality Assurance Program Requirements (Operation),” Revision 2; RG 1.38, “Quality Assurance Requirements for Packaging, Shipping, Receiving, Storage, and Handling of Items for Water-Cooled Nuclear Power Plants,” Revision 2; RG 1.39, “Housekeeping Requirements for Water-Cooled Nuclear Power Plants,” Revision 2; RG 1.94, “Quality Assurance Requirements for Installation, Inspection, and Testing of Structural Concrete and Structural Steel During the Construction Phase of Nuclear Power Plants,” Revision 1; and RG 1.116, “Quality Assurance Requirements for Installation, Inspection, and Testing of Mechanical Equipment and Systems”). In addition, the applicant proposed to revise the VEGP COL FSAR, Appendix 1AA Note (c) to clarify the purpose of a “General” entry under the column labeled “section Criteria” discussion. It is stated that a “Criteria Section” entry of “General” indicates a scope for the conformance statement of “all regulatory guide positions related to programmatic and/or operational aspects.” Thus an associated conformance statement of “Conforms” indicates that the applicant “complies with all regulatory guide positions related to programmatic and or/or operational aspects.” The proposed clarifications clearly provide the scope of conformance to the RGs and, therefore, they are acceptable. The staff verified that the VEGP COL FSAR was updated to reflect above. The staff considers Open Item 1.4-2 resolved for VEGP.

- PTN COL 1.9-1

In comparing VEGP COL FSAR Table 1.9-201 and Appendix 1AA to the respective tables in the Turkey Point Units 6 and 7 COL FSAR, the staff notes that there are several differences. These differences are associated with site-specific information and are reflected in the Turkey Point Units 6 and 7 COL FSAR by a “Turkey Point COL 1.9-1” designation. The staff reviewed the site-specific differences in Table 1.9-201 and Appendix 1AA and has determined that the Turkey Point COL 1.9-1 information in these tables was updated consistently with the update provided for the standard information; therefore, the staff considers the standard content open item as it relates to issues associated with the site-specific information resolved.

- STD COL 1.9-2, PTN COL 1.9-2, STD COL 1.9-3, and STD SUP 1.9-1

The following portion of this technical evaluation section is reproduced from Section 1.4.4 of the VEGP SER:

- *STD COL 1.9-2 (related to the first un-numbered COL information item identified at the end of DCD Table 1.8-2)*

Regarding demonstration of operating experience from Bulletins and GLs, as required by 10 CFR 52.79(a)(37), BLN COL FSAR Table 1.9-204 provides a list of Bulletins and GLs, the appropriate BLN COL FSAR cross-references, and whether the subject matter was addressed in the DCD. The technical discussions related to the specific safety issues are addressed in the related sections of the BLN COL FSAR and are addressed in Chapters 2 through 19 of this SER as needed.

The evaluation of GSI 163, "Multiple Steam Generator Tube Leakage," is described below because otherwise its evaluation would be spread across several SER chapters.

GSI 163 identified a safety concern associated with the potential multiple steam generator (SG) tube leaks triggered by a main steamline break outside containment that cannot be isolated. The issue was evaluated as part of the AP1000 DCD review and was resolved for the AP1000 design. The evaluation was documented in NUREG-1793, Chapter 20. The evaluation states in part the following:

The staff agrees that the issue should be closed for the AP1000 design. Issue 163 concerns the possibility that a multiple steam generator tube rupture (SGTR), resulting from a main steam line break and degraded SG tubes, could result in core damage due to depletion of the reactor coolant and safety injection fluid in the refueling water storage tank. For the AP1000 design, an SGTR is mitigated using the passive core cooling system, initially through the passive residual heat removal heat exchanger, and the core makeup tanks (CMTs). After the CMTs drain to the low level to actuate the automatic depressurization system, the reactor coolant depressurization would result in gravity injection from the in containment refueling water storage tank (IRWST), and eventually from the containment recirculation. The scenario that the safety injection from the refueling water storage tank, which is outside the containment in the existing plants, will be depleted to result in core damage is not likely for the AP1000 design because the IRWST and containment recirculation will continue to provide core cooling.

Since the resolution of Issue 163 is an ongoing NRC effort, any future requirements for the resolution of this issue will be required of the COL applicant, if applicable to the AP1000 design.

Subsequent to the original issuance of NUREG-1793, GSI 163 was closed via a July 16, 2009, memorandum. In the safety evaluation accompanying the closure of the issue, the following is stated:

the staff concludes that the technical specification requirements relating to SG tube integrity provide reasonable assurance that all tubes will exhibit acceptable structural margins against burst or rupture during normal operation and DBAs (including MSLB [main steam line break]), and that leakage from one or multiple tubes under DBAs will be limited to very small amounts, consistent with the applicable regulations for offsite and control room dose.

Therefore, in addition to the unique design features of the AP1000 cited in NUREG-1793 and its supplements as a basis for closure of the issue, the staff notes that for PWR designs in general the issue is resolved based on the technical specification requirements. The staff discusses these technical

specification requirements in Section 5.4, "Component and Subsystem Design," of this SER. Based on the evaluation in NUREG-1793 and its supplements, and based on the staff's evaluation of the SG tube surveillance program in Section 5.4 of this SER, the staff considers GSI 163 resolved for VEGP.

- STD COL 1.9-3

Regarding consideration of new and generic safety issues as required by 10 CFR 52.79(a)(17) and 10 CFR 52.79(a)(20), BLN COL FSAR Table 1.9-203, provides a listing of the TMI Action Plan items, Task Action Plan items, New Generic Issues, Human Factors issues, and Chernobyl Issues and states how they were considered in the DCD and COL application. The technical discussions related to the specific safety issues are addressed in the related sections of the BLN COL FSAR.

In addition, the applicant provided discussion of four new generic issues: Issue 186 related to heavy load drops; Issue 189 related to susceptibility of certain containments to early failure from hydrogen combustion; Issue 191 related to PWR sump performance; and Issue 196 related to the use of Boral in long-term dry storage casks for spent reactor fuel.

The applicant identified that neither Issue 189 nor Issue 196 is applicable to the design or application and that therefore neither is addressed in the BLN COL FSAR. Issue 186 states that there are not any planned heavy load lifts outside those described in the DCD; nonetheless, special procedures to address heavy loads are discussed in Subsection 9.1.5.3. Related to Issue 191, the applicant provided a reference to the protective coatings program and containment cleanliness program in Subsections 6.1.2.1.6 and 6.3.8.1 of the BLN COL FSAR, respectively.

Issue 186 and Issue 196 are evaluated in Chapter 9 of this SER. Issues 189 and 191 are evaluated in Chapter 6 of this SER.

- STD SUP 1.9-1

Regarding conformance with regulatory review criteria as required by 10 CFR 52.79(a)(41), BLN COL FSAR Table 1.9-202 provides the applicant's review of conformance with the acceptance criteria of NUREG-0800. The technical discussions related to the specific acceptance criteria of NUREG-0800 are addressed in the related sections of the BLN COL FSAR and addressed in Chapters 2 through 19 of this SER as needed.

- PTN SUP 1.9-1

Turkey Point COL FSAR Table 1.9-202 contains both site-specific and standard information about the application's conformance with NUREG-0800. The technical discussions related to the specific acceptance criteria of NUREG-0800 are addressed in the related sections of the Turkey Point COL FSAR and addressed in Chapters 2 through 19 of this SER, as needed.

- PTN SUP 1.9-2

The applicant clarified that the SAMDA evaluation for the AP1000 in Appendix 1B to the DCD is not incorporated into the Turkey Point COL FSAR; but is addressed in the Turkey Point COL Environmental Report. The staff reviewed this information as part of its development of the Final Environmental Impact Statement. Therefore, no further evaluation is needed for PTN SUP 1.9-2.

The following portion of this technical evaluation section is reproduced from Section 1.4.4 of the VEGP SER:

- *STD SUP 1.9-3*

This COL supplemental item is addressed as VEGP SUP 8.1-2 [PTN SUP 8.1-3] in SER Section 8.1.

Turkey Point COL FSAR Section 1.10

In this section of the application, the applicant provides an assessment of the potential hazards due to construction of one unit on SSCs important to safety for an operating unit, in accordance with 10 CFR 52.79(a)(31).

The following portion of this technical evaluation section is reproduced from Section 1.4.4 of the VEGP SER:

- *STD SUP 1.10-1*

The NRC staff reviewed the information in BLN COL FSAR Table 1.10-201, identifying the potential hazards from construction activities, BLN COL FSAR Table 1.10-202 that cross-references the construction hazard with the impacted SSCs, and BLN COL FSAR Table 1.10-203, identifying the specific managerial and administrative controls to preclude or mitigate the construction hazard. There is the potential that review of other areas of the application could impact the hazards and management programs identified in the Bellefonte application. For example, site runoff from construction of Unit 4, if not properly controlled, could impact the operation of Unit 3. Site runoff is evaluated in Section 2.4 of this report. The staff has not yet completed its review of this application against the requirements of 10 CFR 52.79(a)(31). This is part of Open Item 1.4-3.

In the application, TVA stated that controls within Section 1.10 of the FSAR are not required unless there is an operating unit on the site. To clarify this FSAR commitment, the staff requests TVA to revise the application to positively state these programs will be in place when there is an operating unit on the site. This is Open Item 1.4-4.

Resolution of Standard Content Open Item 1.4-4

In a letter dated July 29, 2009, the applicant proposed to revise VEGP COL FSAR Section 1.10.3 to positively state that these programs will be in place when there is an operating unit on the site. The staff verified that the VEGP COL

FSAR was appropriately updated to include the above. As a result, Open Item 1.4-4 is resolved.

- PTN SUP 1.10-1

The supplemental information states that the power blocks for Turkey Point Units 6 and 7 have a minimum separation of at least 850 feet between plant centerlines and notes that SSCs important to safety are described in Turkey Point COL FSAR Chapter 3 and the LCOs for Turkey Point Units 6 and 7 are identified in Part 4 of the COL application. In the standard portion of Turkey Point COL FSAR Section 1.10, there is a discussion that the primary consideration in setting the 850-foot separation distance is the space needed to support plant construction via the use of a heavy-lift crane.

The site-specific supplemental information is provided to supplement the standard information above and provides with specificity the location of the SSCs and LCOs required by 10 CFR 52.79(a)(31). The staff's review of this SUP item is included in resolution of Open Item 1.4-3.

The following portion of this technical evaluation section is reproduced from of Section 1.4.4 of the VEGP SER:

Resolution of Standard Content Open Item 1.4-3

A new draft ISG-22 has been issued to assist the staff with the evaluation of COL applicants' compliance with the requirements of 10 CFR 52.79(a)(31). The above draft ISG document was made available to the public including the applicant and was discussed at a public meeting on August 26, 2010.

The regulation at 10 CFR 52.79(a)(31) requires, in part, that applicants for a COL intending to construct and operate new nuclear power plants on multi-unit sites provide an evaluation of the potential hazards to the structures, SSCs important to safety for operating units resulting from construction activities on the new units. The requirement in 10 CFR 52.79(a)(31) can be viewed as having two subparts:

- 1. The COL applicant must evaluate the potential hazards from constructing new plants on SSCs important to safety for existing operating plants that are located at the site.*
- 2. The COL applicant must evaluate the potential hazards from constructing new plants on SSCs important to safety for newly constructed plants that begin operation at the site.*

The interim guidance recommends that the applicant provide a construction impact evaluation plan that includes:

- *A discussion of the construction activity identification process and the impact evaluation criteria used to identify and evaluate the construction activities that may pose potential hazards to the SSCs important to safety for operating unit(s).*

- *A table of those construction activities and the potential hazards that are identified using that construction impact evaluation plan, the SSCs important to safety for the operating unit potentially impacted by the construction activity, and expected mitigation method.*
- *Identification of the managerial and administrative controls, such as proposed license conditions that may involve construction schedule constraints or other restrictions on construction activities, that are credited to preclude and/or mitigate the impacts of potential construction hazards to the SSCs important to safety for the operating unit(s).*
- *A discussion of the process for communications and interactions planned and credited between the construction organization and the operations organization to ensure appropriate coordination and authorization of construction activities and implementation of the prevention or mitigation activities as necessary.*
- *A memorandum of understanding or agreement (MOU or MOA) between the COL applicant and the operating unit(s) licensee as a mechanism for communications, interactions, and coordination to manage the impact of the construction activities.*
- *An implementation schedule corresponding to construction tasks or milestones to ensure the plan is reviewed on a recurring basis and maintained current as construction progresses.*

The staff reviewed the VEGP COL FSAR Section 1.10, which provides information to address compliance with 10 CFR 52.79(a)(31). In order to complete the staff's review, in RAI 1.5-2, the staff requested that the applicant to provide a construction impact evaluation plan that includes:

- *A discussion of the process for communications and interactions planned and credited between the construction organization and the operations organization to ensure appropriate coordination and authorization of construction activities and implementation of the prevention or mitigation activities as necessary.*
- *A memorandum of understanding or agreement (MOU or MOA) between the COL applicant and the operating unit(s) licensee as a mechanism for communications, interactions, and coordination to manage the impact of the construction activities.*
- *An implementation schedule corresponding to construction tasks or milestones to ensure the plan is reviewed on a recurring basis and maintained current as construction progresses.*

In addition, the applicant was requested to identify the managerial and administrative controls (VEGP COL FSAR Table 1.10-203) that are credited to

preclude and/or mitigate the impacts of potential construction hazards to the SSCs important to safety for the operating units (VEGP Units 1 and 2).

In a letter dated November 2, 2010, the applicant stated:

- *VEGP COL FSAR Sections 1.10.2 and 13AA will be revised to include the discussion of the process for communications and interactions planned and credited between the construction organization and the operations organization.*
- *The COL applicant and the operating unit(s) licensee are the same entity, thus, no MOU or MOA is considered necessary.*
- *VEGP COL FSAR Sections 1.10.3 and 13AA will be revised to include the discussion of the implementation schedule corresponding to construction tasks or milestones.*
- *VEGP COL FSAR will be revised to indicate that managerial and administrative controls are developed and implemented as work progresses on site. These controls are intended to preclude and/or mitigate the impacts of potential construction hazards to the SSCs important to safety for the operating units.*

*The proposed changes to the VEGP COL FSAR meet the draft guidance of ISG-22 and, therefore, meet the requirements of 10 CFR 50.79(a)(31). The incorporation of the above proposed changes into a future revision of the VEGP COL FSAR is **Confirmatory Item 1.4-2**.*

Resolution of Standard Content Confirmatory Item 1.4-2

Confirmatory Item 1.4-2 is an applicant commitment to revise FSAR Sections 1.10.2 and 1.10.3 and Appendix 13A to address guidance included in ISG-22. The staff verified that the VEGP COL FSAR was appropriately revised. As a result, Confirmatory Item 1.4-2 is now closed.

License Conditions

- *Part 10, License Condition 1, ITAAC*

The applicant proposed that the ITAAC identified in the tables in Appendix B of Part 10 of the VEGP COL application be incorporated into the COL. The proposed license condition also states that after the Commission has made the finding required by 10 CFR 52.103(g), "Operation under a combined license," the ITAAC do not constitute regulatory requirements; except for specific ITAAC, which are subject to a hearing under 10 CFR 52.103(a), their expiration will occur upon final Commission action in such proceeding.

The ITAAC identified in tables in Appendix B of Part 10 of the VEGP COL application are evaluated throughout this SER. The remaining text of the

proposed license condition is already covered by regulatory requirements of 10 CFR 52.103(h). Therefore, there is no need for a license condition.

1.4.5 Post Combined License Activities

For the reasons discussed in the technical evaluation section above, the NRC staff has identified the following FSAR commitment as the responsibility of the licensee:

- **Turkey Point Commitment Number 1.4-1**—A site-specific construction plan and startup schedule will be provided after issuance of the COL.

1.4.6 Conclusion

The NRC staff reviewed the application and checked the referenced DCD. The NRC staff's review confirmed that the applicant addressed the required information relating to principal review matters, and there is no outstanding information expected to be addressed in the Turkey Point COL FSAR related to this section. The results of the NRC staff's technical evaluation of the information incorporated by reference in the Turkey Point COL application are documented in NUREG-1793 and its supplements.

1.5 Additional Regulatory Considerations

1.5.1 10 CFR 52.97(a)(1)(iv) Applicant Financial Qualifications and Evaluation of Financial Qualification in Accordance with 10 CFR 50.33

BACKGROUND:

Florida Power and Light Company

According to the COL application, FPL is an investor-owned utility primarily engaged in the generation, transmission, and distribution of electricity in the southern third and almost the entire eastern seaboard of Florida. FPL supplies electric service to approximately 4.5 million customer accounts. There are five FPL power plants at the Turkey Point site: two natural gas/oil steam electric generating units (Units 1 and 2), two pressurized-water reactor nuclear units (Units 3 and 4), and one natural gas, combined-cycle, steam electric generating unit (Unit 5). Additionally, FPL owns and operates the following nuclear power plants:

- St. Lucie Unit 1, near Ft. Pierce, FL
- St. Lucie Unit 2, near Ft. Pierce, FL (85 percent ownership, FPL is authorized to act as agent for the Orlando Utilities Commission of the city of Orlando, Florida and Florida Municipal Power Agency)

The address of the applicant is Florida Power & Light Company, 700 Universe Boulevard, Post Office Box 14000, Juno Beach, FL 33408.

REGULATORY EVALUATION:

The applicant's request for the NRC to issue two combined licenses under Section 103 of the AEA for construction and operation is subject to, among other things, the requirements of the AEA, Subpart C to 10 CFR Part 52, 10 CFR Part 50, and 10 CFR Part 140.

In its review, the NRC staff used guidance in NUREG-1577, "Standard Review Plan on Power Reactor Licensee Financial Qualifications and Decommissioning Funding Assurance," Revision 1, issued February 1999, to evaluate the financial qualifications of the applicant to construct, operate, and decommission the proposed facility.

In addressing foreign ownership, control, or domination (FOCD), the NRC staff used guidance in the Standard Review Plan (SRP), "Foreign Ownership, Control, and Domination of applicants for Reactor Licenses," dated June 1999 (SRP on FOCD) to determine whether the applicant is owned, controlled, or dominated by an alien, a foreign corporation, or a foreign government. The NRC published the SRP on FOCD in the *Federal Register* on September 28, 1999 (64 FR 52357-52359).

The staff also used guidance in NUREG-1307, Revision 15, "Report on Waste Burial Charges: Changes in Decommissioning Waste Disposal Costs at Low-Level Waste Burial Facilities," to independently validate the licensee's calculation of the minimum funding needed for decommissioning.

The safety evaluation which follows documents the staff's review and analysis of financial qualifications, decommissioning funding assurance, FOCD, and nuclear insurance and indemnity. FPL has chosen to pursue this application under 10 CFR Part 52.

FINANCIAL QUALIFICATIONS:

Pursuant to 10 CFR 52.77, the application must include all of the information required by 10 CFR 50.33.

Construction:

Pursuant to 10 CFR 50.33(f)(1):

[T]he applicant shall submit information that demonstrates that the applicant possesses or has reasonable assurance of obtaining the funds necessary to cover estimated construction costs and related fuel cycle costs. The applicant shall submit estimates of the total construction costs of the facility and related fuel cycle costs, and shall indicate the source(s) of funds to cover these costs.

Construction Cost Estimate:

Under 10 CFR Part 50, Appendix C, "A Guide for the Financial Data and Related Information Required To Establish Financial Qualifications for Construction Permits and Combined Licenses," Section I.A.1:

[E]ach applicant's estimate of the total cost of the proposed facility has been broken down as follows and be accompanied by a statement describing the bases from which the estimate is derived:

- (a) Total nuclear production plant costs; [and]
- (b) Transmission, distribution, and general plant costs; [and]
- (c) Nuclear fuel inventory cost for first core.

If the fuel is to be acquired by lease or other arrangement than purchase, the application should so state. The items to be included in these categories should be the same as those defined in the applicable electric plant and nuclear fuel inventory accounts prescribed by the Federal Energy Regulatory Commission or an explanation given as to any departure therefrom.

In accordance with 10 CFR 50.33(f) and 10 CFR Part 50, Appendix C, the projected overnight costs for the construction of two AP1000 advanced light water reactors Turkey Point Units 6 and 7 site are outlined below (from Turkey Point Units 6 and 7 COLA, Part 1, Appendix 1A):

ESTIMATED TOTAL CONSTRUCTION COSTS
FOR TURKEY POINT UNITS 6 AND 7
(In 2015 \$)

	TOTAL	
	<u>Low Range</u>	<u>High Range</u>
Total Nuclear Production Plant Costs	\$6,679,486,952	\$9,729,206,579
Transmission, Distribution & General Plant Costs	\$1,739,757,497	\$2,520,144,553
Nuclear Fuel Inventory & Cost for First Core	\$39,598,094	\$48,370,595
TOTAL (OVERNIGHT COST)	\$8,458,842,544	\$12,297,721,727
Interest & Escalation	\$5,241,656,375	\$7,696,339,598
Total w/ Interest & Escalation	\$13,700,498,919	\$19,994,061,325

FPL initially developed the cost estimate by coupling information in the 2005 TVA Bellefonte Study for the ABWR technology with FPL site-specific information. Next, a technology-specific (AP1000) cost estimate was developed in 2010 to reflect current pricing and project features. This cost estimate was consistent with, and at the high end of, the original cost estimate range, following adjustment for the specific reactor technology and annual escalation. The cost estimate range remains consistent with the publicly available cost estimates of other U.S. AP1000 projects.

The construction cost estimate is expressed in term of “overnight cost,” which is a term commonly used in describing the cost of large capital projects.⁹ The applicant presented a combined Unit 6 and Unit 7 cost estimate for plant construction ranging from \$8,458M to \$12,297M. The applicant also escalated these values thereby resulting in an overnight cost ranging between \$13,700M and \$19,994M. Turkey Point Units 6 and 7 are expected to operate at an estimated combined gross electrical power output of approximately 2,200 MWe. Therefore the total overnight cost, including fuel costs as described above, is between \$3,845 and \$5,590 per kWe installed. If staff considered the escalated values, the cost range would be \$6,227 and \$9,088 per kWe. This value is derived by dividing the overnight cost per unit by the MWe output per unit. As noted above, the applicant has used, in part, a Westinghouse/Shaw consortium to develop technology-specific cost estimate in 2010.

The NRC staff compared the FPL AP1000 overnight construction cost estimate to cost estimates from a number of studies¹⁰ that have been conducted by governmental agencies, universities, and other entities. In particular, the U.S. Energy Information Administration’s (EIA) June 2012 report, “Annual Energy Outlook 2012 with Projections to 2035,” (DOE/EIA-0383(2012)), states that “the overnight capital costs associated with building a nuclear power plant planned in 2012 are assumed to be \$5,335 per kilowatt of capacity...” The staff applied a conservative annual adjustment factor ranging from 3% to 10% to the EIA overnight capital cost estimate to account for inflation beyond 2012, and determined that the EIA projected 2015 overnight cost would range from \$5,830 to \$7,101/kWe installed. The applicant’s overnight cost estimate range is in line with the most recent EIA 2012 range of overnight costs as adjusted for inflation. The staff determined that the FPL overnight construction cost estimate was consistent with studies of these costs identified above. Further, the FPL cost estimate is consistent with the publicly available cost estimates of other U.S. AP1000 projects. Accordingly, the NRC staff finds FPL’s overnight cost estimate to be reasonable as presented in its COL application.

Sources of Construction Funds:

Pursuant to 10 CFR Part 50, Appendix C, Section I.A.2:

The application should include a brief statement of the applicant's general financial plan for financing the cost of the facility, identifying the source or sources upon which the applicant relies for the necessary construction funds, e.g., internal sources such as undistributed earnings and depreciation accruals, or external sources such as borrowings.

According to the COL application, FPL expects to recover the cost of constructing the facility in accordance with Florida Statute 366.93, “Cost Recovery for the Siting, Design, Licensing, and Construction of Nuclear and Integrated Gasification Combined Cycle Power Plants,” and Florida Administrative Code R.25-6.0423, “Nuclear or Integrated Gasification Combined Cycle Power Plant Cost Recovery.” FPL expects to finance this project through a mixture of internally generated cash and external funding. The external funding will come from a mix of debt and equity capital. FPL currently uses first mortgage bonds and equity contributions from NextEra

⁹ Overnight cost is the cost of a construction project if no interest was incurred during construction, as if the project was completed “overnight.” An alternate definition is: the present value cost that would have to be paid as a lump sum up front to completely pay for a construction project. The overnight cost is frequently used when describing power plants.

¹⁰ See, e.g., the 2003 (updated in 2009) the Massachusetts Institute of Technology (MIT) interdisciplinary study entitled *The Future of Nuclear Power*; the U.S. Department of Energy’s Energy Information Agency (EIA) 2012 Annual Energy Outlook (AEO); the Nuclear Energy Agency (NEA) of the Organization for Economic Cooperation and Development 2005 update on *Projected Costs of Generating Electricity*; and the Keystone Center 2007 report entitled *Nuclear Power Joint Fact-Finding*.

Energy, Inc. to finance long-term utility assets. The staff concludes that both FPL and NextEra Energy have sufficient financing capacity to fund this project from the following sources: internally generated operating cash flows, commercial paper and bank facilities, and long-term debt and equity capital markets; and will recover the cost of constructing the facility in accordance with Florida Statute 366.93 and Florida Administrative Code R.25-6.0423.

Financial Qualifications

Pursuant to 10 CFR Part 50, Appendix C, Section I.A.3:

The application should also include the applicant's latest published annual financial report, together with any current interim financial statements that are pertinent. If an annual financial report is not published, the balance sheet and operating statement covering the latest complete accounting year together with all pertinent notes thereto and certification by a public accountant should be furnished.

FPL filed financial statements with the FPSC and the U.S. Securities and Exchange Commission (SEC) at the time they submitted the COL application. FPL submitted, pursuant to 10 CFR Part 50, Appendix C, Section I.A.3, annual financial statements. The NRC staff did not identify anything in FPL's financial statements that warranted further inquiry.

In consideration of the foregoing, the NRC staff finds that the applicant has demonstrated that it possesses or has reasonable assurance of obtaining the funds necessary to cover estimated construction costs and related fuel cycle costs. Therefore, the NRC staff finds that the applicant is financially qualified to construct the facilities.

Operating License

Pursuant to 10 CFR 50.33(f)(3),

If the application is for a combined license under subpart C of part 52 of this chapter, the applicant shall submit the information described in paragraphs (f)(1) and (f)(2) of this section.

10 CFR 50.33(f) provides that each application shall state:

[e]xcept for an electric utility applicant for a license to operate a utilization facility of the type described in 10 CFR 50.21(b) or 50.22, information sufficient to demonstrate to the Commission the financial qualification[s] of the applicant to carry out, in accordance with the regulations in this chapter, the activities for which the permit or license is sought.

10 CFR 50.2, "Definitions" states, in part, that an electric utility is:

[a]ny entity that generates or distributes electricity and which recovers the cost of this electricity, either directly or indirectly, through rates established by the entity itself or by a separate regulatory authority.

According to the information provided in the application, FPL is an electric utility as defined in 10 CFR 50.2. FPL is an investor-owned electric utility primarily engaged in the generation, transmission, and distribution of electricity and serves approximately 4.5 million customer accounts in the state of Florida. FPL is a regulated public utility and is subject to the regulatory provisions of the FPSC, the NRC, and the Federal Energy Regulatory Commission.

In consideration of the foregoing, the NRC staff finds that the applicant is an electric utility as defined in 10 CFR 50.2 and therefore is not subject to the financial qualification information requirements of 10 CFR 50.33(f)(2). Accordingly, the applicant is not required to provide financial qualification information related to operating cost recovery.

DECOMMISSIONING FUNDING ASSURANCE:

Regulatory Requirements:

Pursuant to 10 CFR 50.33(k)(1):

[A]n application for [a ...] combined license for a production or utilization facility, information in the form of a report, as described in 10 CFR 50.75, indicating how reasonable assurance will be available to decommission the facility.

Under 10 CFR 50.75, "Reporting and recordkeeping for decommissioning planning," the report must include a certification that the applicant will provide financial assurance for decommissioning using one or more of the methods allowed under the regulation at 10 CFR 50.75(e) no later than 30 days after the Commission publishes notice in the *Federal Register* under 10 CFR 52.103(a). In addition, the amount of the financial assurance may be more, but not less, than the amount stated in the table in 10 CFR 50.75(c)(1), as adjusted under 10 CFR 50.75(c)(2). Under 10 CFR 50.75(b)(4), a COL applicant need not obtain a financial instrument appropriate to the method to be used or submit a copy of the instrument to the Commission. (Once the COL is granted, the holder of a COL must submit an instrument as provided in 10 CFR 50.75(e)(3)).

Additionally, the staff used the guidance in NUREG-1577, Rev. 1, in its review of the Turkey Point Units 6 and 7 COL application.

Decommissioning Funding Estimate:

Turkey Point is a two-unit PWR site that is incorporating by reference the Westinghouse AP1000 certified design, as documented in the referenced DCD including any supplemental material.

FPL stated that it will provide decommissioning funding assurance in an amount of \$517 million per unit. The NRC staff independently calculated the minimum funding acceptable under 10 CFR 50.75(c), and found the applicant's amounts acceptable.

Decommissioning Funding Mechanism:

FPL stated in the application that it would use an external sinking fund in the form of a trust as the method to provide decommissioning funding assurance. Under 10 CFR 50.75(e)(1)(ii), an external sinking fund may be used as an exclusive method by a:

. . . licensee that recovers, either directly or indirectly, the estimated total cost of decommissioning through rates established by 'cost of service' or similar ratemaking regulation.

The NRC staff will verify the adequacy of the decommissioning funding mechanism and prospective financial instrument in the future consistent with the schedule, set forth in 10 CFR 50.75(e)(3), for the submission of reports by a holder of a combined license.

Therefore, the NRC staff finds that FPL has complied with applicable decommissioning funding assurance requirements.

ANTITRUST REVIEW:

The *Energy Policy Act of 2005* (EPAct) removed the antitrust review authority in Section 105c. of the AEA, as amended, regarding license applications for production or utilization facilities submitted under Sections 103 or 104b. of the AEA after the date of enactment of the EPAct. Accordingly, the NRC is not authorized to conduct an antitrust review in connection with this COL application.

FOREIGN OWNERSHIP, CONTROL, OR DOMINATION (FOCD):

Section 103 of the AEA, in relevant part, prohibits the Commission from issuing a license for a nuclear power plant to:

an alien or any corporation or other entity if the Commission knows or has reason to believe it is owned, controlled, or dominated by an alien, a foreign corporation or a foreign government.

10 CFR 50.38, "Ineligibility of certain applicants," is the regulatory provision that implements this statutory prohibition.

The NRC staff reviewed the application pursuant to the guidance provided in the SRP on FOCD to determine whether the applicant is owned controlled, or dominated by an alien, a foreign corporation, or a foreign government.

According to the application, FPL is not owned, controlled, or dominated by any alien, foreign corporation, or foreign government. FPL is an investor-owned public utility incorporated under the laws of the state of Florida, with its principal office located in Juno Beach, Florida. FPL is a wholly-owned subsidiary of NextEra Energy, Inc., an investor-owned electric utility. FPL's common stock is held solely by NextEra Energy, Inc. NextEra Energy, Inc. is investor-owned, with 27,994 common stockholders as of January 31, 2010. The shares of common stock of NextEra Energy, Inc. are publicly traded and widely held. The application also contains the names and addresses of the directors and officers of FPL and indicates that all are United States citizens.

Staff conducted an independent analysis, including open-source research and verification of the information provided in the application related to ownership of FPL, and found no evidence of FOCD.

Based on the foregoing, the staff does not know or have reason to believe that FPL is owned, controlled, or dominated by a foreign interest. Therefore, FPL conforms to the guidance provided in the SRP for FOCD and meets the requirements of 10 CFR 50.38.

NUCLEAR INSURANCE AND INDEMNITY:

This section of the SER addresses the applicant's offsite and onsite insurance requirements in Title 10 of the *Code of Federal Regulations* (10 CFR) Part 140, "Financial Protection Requirements and Indemnity Agreements," and 10 CFR 50.54(w), respectively.

The provisions of the Price-Anderson Act (Section 170 of the Atomic Energy Act of 1954, as amended) and the Commission's regulations in 10 CFR Part 140 require, in part, that each holder of a license issued pursuant to 10 CFR Part 52 have and maintain financial protection. Further, 10 CFR 50.54(w) establishes requirements for each power reactor licensee to obtain insurance or provide an equivalent amount of protection for the onsite costs of an accident. Under these regulations, FPL is required to provide satisfactory documentation that it has obtained the amount of financial protection required by (1) 10 CFR 140.13, "Amount of financial protection required of certain holders of construction permits and combined licenses under 10 CFR Part 52," (2) 10 CFR 140.11(a)(4), and (3) 10 CFR 50.54(w). In addition, each licensee is required to have and maintain financial protection under 10 CFR 140.11(a)(4) and shall provide evidence that it maintains a guarantee of payment of deferred premiums pursuant to 10 CFR 140.21, "Licensee guarantees of payment of deferred premiums." Finally, as required by 10 CFR 140.20, "Indemnity agreements and liens," the NRC staff will amend FPL's current indemnity agreement for existing Turkey Point Units 3 and 4 to include the addition of Turkey Point Units 6 and 7.

The regulation in 10 CFR 140.13 provides the amount of financial protection required by a Part 52 license holder, who also holds a license under 10 CFR Part 70, "Domestic Licensing of Special Nuclear Material," during the period before the Commission makes the finding under 10 CFR 52.103(g) (i.e., a finding that the acceptance criteria in the license are met, which allows the licensee to initially load fuel and operate). Because the 10 CFR Part 70 license will be issued with the COL, FPL must have and maintain \$1,000,000 in financial protection from issuance of the COL until the 10 CFR 52.103(g) finding is made. In addition, as required by 10 CFR 140.11(a)(4), after the 10 CFR 52.103(g) finding is made, each licensee must have and maintain financial protection in an amount equal to the sum of primary financial protection (\$375,000,000) and the amount available as secondary financial protection. By letter dated June 14, 2016 (ADAMS Accession No. ML16169A294), FPL provided proof of insurance coverage from American Nuclear Insurers (ANI) in the amount of \$375 million. Currently, ANI insures Turkey Point Units 3 and 4, Turkey Point Units 6 and 7 are covered under the same policy. This coverage will be effective concurrent with the NRC's issuance of a COL to FPL. Therefore, the staff concludes that the \$375,000,000 coverage satisfies the \$1,000,000 requirement of 10 CFR 140.13, and the primary financial protection requirement in 10 CFR 140.11(a)(4).

The staff notes that although licensees of large operating reactors under 10 CFR Part 50 and 10 CFR Part 52 must have and maintain financial protection under 10 CFR 140.11(a)(4) upon NRC action authorizing operation, the timing provisions for reporting under 10 CFR 140.21 do not explicitly address the 10 CFR Part 52 process. Under the requirements in 10 CFR 140.11(a)(4) and 10 CFR 140.21, the coverage for secondary financial protection and the guarantee of payment of deferred premiums are only required for reactors authorized to load fuel and operate. Under the 10 CFR Part 52 COL process, the license authorizes operation only upon a Commission finding pursuant to 10 CFR 52.103(g). Therefore, these requirements apply as of the date that the Commission makes such a finding. While 10 CFR 50.54(w) by its terms applies upon a Commission finding under 10 CFR 52.103(g), FPL also included a reporting requirement for 10 CFR 50.54(w) in its proposed condition.

FPL proposed the following license condition to address the reporting of 10 CFR Section 140.11(a)(4) requirements for secondary financial protection, and the reporting of 50.54(w) requirements for onsite financial protection. The staff agrees with the proposed license condition with modifications. The staff's recommended license condition is as follows:

License Condition (1-1) - Before the scheduled date for initial fuel load, and within ninety (90) days after the NRC publishes the notice of intended operation in the *Federal Register*, FPL shall provide satisfactory documentary evidence to the Director of the Office of Nuclear Reactor Regulation or the Director's designee that it has obtained the appropriate amount of primary and secondary financial protection required of licensees pursuant to 10 CFR 140.11(a)(4) and the appropriate amount of financial protection required by 10 CFR 50.54(w).

With the license condition as described above, the staff concludes that FPL will satisfy the requirements of 10 CFR 140.11(a)(4) with respect to obtaining an appropriate amount of secondary financial protection and 10 CFR 50.54(w) with respect to obtaining the appropriate amount of financial protection. The staff notes that it will conform any license condition to the correct format if the Commission determines to issue the license. For example, the staff may change "the Director of the Office of Nuclear Reactor Regulation" to "the Director of the Office of New Reactors" and other similar changes.

FPL also proposed the following license condition to address the reporting of 10 CFR 140.21 for guarantee of payment of deferred premiums. The staff agreed with the proposed license condition with modifications. The staff's recommended license condition is stated below:

License Condition (1-2) - Before the scheduled date of initial fuel load, and within 90 days after the NRC publishes the notice of intended operation in the *Federal Register*, the licensees shall provide evidence to the NRC that they would have the ability to pay into the nuclear industry retrospective rating plan in the event of a nuclear incident and in the amount specified in 10 CFR 140.11(a)(4) for one calendar year using one of the following methods:

- a. Surety bond
- b. Letter of credit
- c. Revolving credit/term loan arrangement
- d. Maintenance of escrow deposits of government securities, or
- e. Annual certified financial statement showing either that a cash flow (i.e., cash available to a company after all operating

expenses, taxes, interest charges, and dividends have been paid) can be generated and would be available for payment of retrospective premiums within three (3) months after submission of the statement, or a cash reserve or a combination of cash flow and cash reserve.

Thereafter, FPL shall provide evidence of the guarantees of payment of deferred premiums in accordance with the provisions specified in 10 CFR 140.21.

With the license condition as described above, the staff concludes that that FPL will satisfy the requirement in 10 CFR 140.21.

In consideration of the staff's evaluation and license conditions as described above, the staff concludes that FPL will satisfy the provisions of the Price-Anderson Act (Section 170 of the Atomic Energy Act of 1954, as amended) and the Commission's applicable regulations in 10 CFR Part 140, 10 CFR Part 52, and 10 CFR Part 50 for insurance and indemnity.

CONCLUSION:

In consideration of the staff's evaluation and license conditions as described above, the staff finds that FPL satisfies the provisions of the Price-Anderson Act (Section 170 of the Atomic Energy Act of 1954, as amended) and the Commission's applicable regulations in 10 CFR Part 140, 10 CFR Part 50, and 10 CFR Part 52 for insurance and indemnity. As presented in its application, FPL also conforms with the staff's guidance in NUREG-1577 and the Standard Review Plan on Foreign Ownership, Control or Domination. Accordingly, staff concludes that FPL is financially qualified to engage in the proposed activities for Turkey Point Units 6 and 7, and that there are no decommissioning funding assurance or foreign ownership, control, or domination issues.

1.5.2 Nuclear Waste Policy Act

Section 302(b) of the Nuclear Waste Policy Act of 1982, as amended, states,

The Commission, as it deems necessary or appropriate, may require as a precondition to the issuance or renewal of a license under section 103 or 104 of the Atomic Energy Act of 1954 [42 U.S.C. 2133, 2134] that the applicant for such license shall have entered into an agreement with the Secretary for the disposal of high-level radioactive waste and spent nuclear fuel that may result from the use of such license.

As listed in Table 1.2-1 of the Turkey Point COL application Environmental Report, FPL has contracts with the U.S. Department of Energy (DOE) establishing the terms and conditions applicable to the DOE's responsibility for disposal of spent nuclear fuel generated at the proposed Turkey Point Units 6 and 7. The DOE contract numbers referenced in the table are DE-CR01-09RW9012 for Turkey Point Unit 6 and DE-CR01-09RW9013 for Turkey Point Unit 7.

Because FPL has entered into contracts with the DOE for the disposal of spent nuclear fuel for Turkey Point Units 6 and 7, the staff considers that FP has met the applicable requirements of Section 302(b) of the Nuclear Waste Policy Act of 1982.

1.5.3 Consultation with Department of Homeland Security and Notifications

1.5.3.1 Consultation with Department of Homeland Security

In accordance with Section 657 of the EPAct, the NRC consulted with the Department of Homeland Security (DHS) with respect to the FPL COL application for Turkey Point Units 6 and 7. Between December 1, 2009, and December 3, 2009, DHS conducted a site visit. On August 2, 2010, NRC issued a DHS consultation report regarding the DHS site visit (ADAMS Accession No. ML101890846). The DHS report concludes that the applicant and the NRC staff have satisfied the requirements of Section 657 of the EPAct.

1.5.3.2 Notifications

As required by Section 182c of the AEA, as amended, and 10 CFR 50.43(a), the NRC notified the Public Service Commission of Florida of the Turkey Point COL application (ADAMS Accession No. ML16182A212). In addition, in July 2008, the NRC published notices of the application in *Total Leader and South Dade News Leader*. In accordance with Section 182c., the staff also published a notice of the application in the Federal Register on November 18, November 25, December 2, and December 9, 2011 (76 FR 71608, 72725, 75566, and 77021).

Based on the staff's completion of notifications to regulatory agencies and the public notices described above, the staff concludes that, for the purposes of issuing COLs for Turkey Point Units 6 and 7, any required notifications to other agencies or bodies have been duly made.

1.5.4 Evaluation of Departures and Exemption Associated with Numbering in the Application and Exemption Associated with Special Nuclear Material Material Control and Accounting Program

Evaluation of Departures and Exemption Associated with Numbering in the Application

In STD DEP 1.1-1, the applicant renumbered Turkey Point COL FSAR sections to include content consistent with RG 1.206 and NUREG-0800. The departure and the exemption associated with the numbering scheme of the FSAR are closely related. The departure provided in Part 7 of the COL application provides the specific sections of the Turkey Point COL FSAR that deviate from the DCD numbering scheme.

Pursuant to 10 CFR 52.7, "Specific Exemptions," and 10 CFR 52.93, "Exemptions and Variances," the applicant requested an exemption from 10 CFR Part 52, Appendix D, Section IV.A.2.a, to include "a plant-specific DCD containing the same type of information and using the same organization and numbering as the generic DCD for the AP1000 design...." In Part 7, "Departures and Exemptions," of the Turkey Point COL application, the applicant states that the exemption will not result in any significant departures from the expected organization and numbering of a typical FSAR, and the information is readily identifiable to facilitate NRC review. The applicant states that the subject deviations are considered purely administrative to support a logical construction of the document. Further, the revised organization and numbering generally follows the guidance provided in RG 1.206 and NUREG-0800.

Pursuant to 10 CFR 52.7, the Commission may, upon application by any interested person or upon its own initiative, grant exemptions from the requirements of 10 CFR Part 52. 10 CFR 52.7 further states that the Commission's consideration will be governed by

10 CFR 50.12, "Specific exemptions," which states that an exemption may be granted when: (1) the exemptions are authorized by law, will not present an undue risk to public health or safety, and are consistent with the common defense and security; and (2) special circumstances are present. Special circumstances are present whenever, according to 10 CFR 50.12(a)(2)(ii), "Application of the regulation in the particular circumstances would not serve the underlying purpose of the rule or is not necessary to achieve the underlying purpose of the rule."

Before considering whether this numbering exemption should be granted, the staff needed to address a threshold question regarding the review standard applicable to the request. Under 10 CFR 52.93(a)(1), if a request for an exemption is from any part of a DC rule, then the Commission may grant the exemption if the exemption complies with the appropriate change provision in the referenced DC rule, or if there is no applicable change provision, if the exemption complies with 10 CFR 52.63. Here, there is no applicable change provision in the referenced DC rule, so according to 10 CFR 52.93(a)(1), the exemption must meet 10 CFR 52.63. However, the standards of the appropriate provision of 10 CFR 52.63 applicable to requests for exemptions from a DC rule in 10 CFR 52.63(b)(1), by their terms, also do not apply to this change. Specifically, 10 CFR 52.63(b)(1) applies to changes to "certification information," and not administrative or procedural DC rule provisions such as this one under consideration. In the Statements of Consideration for 10 CFR 52.63, the Commission stated that it used the "phrase 'certification information' in order to distinguish the rule language in the DCRs from the DC information (e.g., Tier 1 and Tier 2) that is incorporated by reference in the DCRs," (72 FR 49444). The exemption requested from the AP1000 DCD numbering scheme is an exemption from rule language, not Tier 1 or Tier 2 information; therefore, 10 CFR 52.63 should not be used to analyze this exemption.

Because there is not an applicable change provision in the referenced DC, and because 10 CFR 52.63(b)(1) does not apply to this exemption, the exemption cannot comply with the plain language of 10 CFR 52.93(a)(1). In this situation, the language of 10 CFR 52.93(a)(1) does not appear to serve the underlying purpose of the regulation as described by the Commission in the Statements of Consideration to the rule, in which the Commission stated that only changes to certification information must meet 10 CFR 52.63. Instead, this exemption should have fallen under 10 CFR 52.93(a)(2), and, thus, be analyzed under the requirements in 10 CFR 52.7. Therefore, the staff finds that, pursuant to 10 CFR 52.7, an exemption to 10 CFR 52.93(a)(1) should be granted. This exemption is warranted because it meets the requirements in 10 CFR 50.12. First, because this is an administrative change regarding what exemption regulation applies, the exemption to 10 CFR 52.93(a)(1) is authorized by law, will not present an undue risk to public health or safety, and is consistent with the common defense and security. In addition, application of the regulation in this case is not necessary to achieve the underlying purpose of the rule. The underlying purpose of the rule is to maintain the safety benefits of standardization by requiring any exemption from certification information to meet the requirements in 10 CFR 52.63(b)(1). This underlying purpose does not apply to this exemption because the form and organization of the application does not affect the safety benefits of standardization of the certification information. Therefore, for the purpose of determining the standards applicable to the exemption related to STD DEP 1.1-1, the staff finds an exemption to 10 CFR 52.93(a)(1) to be acceptable for the review of the exemption related to STD DEP 1.1-1.

Pursuant to the exemption described above, the NRC staff has reviewed the exemption related to STD DEP 1.1-1 to determine whether it meets the requirements in 10 CFR 52.7. This exemption would allow the applicant to provide an FSAR with numbering and topics more closely related to NUREG-0800 and RG 1.206. The staff finds that this administrative change of

minor renumbering will not present an undue risk to the public health and safety and is consistent with the common defense and security. In addition, this exemption is consistent with the AEA and is authorized by law. Further, the application of the regulation in these particular circumstances is not necessary to achieve the underlying purpose of the rule. Therefore, the staff finds that the exemption to 10 CFR Part 52, Appendix D, Section IV.A.2.a is justified. Finally, for the same reasons the staff is granting the exemption request, the staff also finds the departure from the numbering scheme in the Turkey Point COL FSAR to be acceptable.

Exemption Associated with Special Nuclear Material Material Control and Accounting Program

In Part 7 of the Turkey Point COL application, the applicant requested an exemption from the requirements of 10 CFR 70.22(b), 10 CFR 70.32(c) and, in turn, 10 CFR 74.31, 10 CFR 74.41, and 10 CFR 74.51. The provision of 10 CFR 70.22(b) requires an application for a license for SNM to include a full description of the applicant's program for MC&A of SNM under 10 CFR 74.31, 10 CFR 74.33, 10 CFR 74.41, or 10 CFR 74.51.¹¹ 10 CFR 70.32(c) requires a license authorizing the use of SNM to include and be subjected to a condition requiring the licensee to maintain and follow an SNM MC&A program as required under 10 CFR Part 74 Subparts C through E and to request Commission approval prior to implementing program changes. However, 10 CFR 70.22(b), 10 CFR 70.32(c), 10 CFR 74.31, 10 CFR 74.41, and 10 CFR 74.51 include exceptions for nuclear reactors licensed under 10 CFR Part 50. The regulations applicable to the MC&A of SNM for nuclear reactors licensed under 10 CFR Part 50 are provided in 10 CFR Part 74, Subpart B and 10 CFR 74.11 through 10 CFR 74.19 (excluding 10 CFR 74.17). The applicant stated that the purpose of this exemption request is to seek a similar exception for this COL under 10 CFR Part 52, such that the same regulations will be applied to the SNM MC&A program as nuclear reactors licensed under 10 CFR Part 50. In addition, the applicant stated that the exemption request is evaluated under 10 CFR 52.7, which incorporates the requirements of 10 CFR 50.12. As stated previously, that section allows the Commission to grant an exemption under the following conditions: 1) the exemption is authorized by law, will not present an undue risk to the public health and safety, and is consistent with the common defense and security and 2) special circumstances are present as specified in 10 CFR 50.12(a)(2). The criteria in 10 CFR 50.12 encompass the criteria for an exemption in 10 CFR 70.17(a) and 10 CFR 74.7, the specific exemption requirements for 10 CFR Parts 70 and 74, respectively. Therefore, by demonstrating that the exemption criteria in 10 CFR 50.12 are satisfied, this request would also demonstrate that the exemption criteria in 10 CFR 52.7, 10 CFR 70.17(a), and 10 CFR 74.7 are satisfied.

The applicant stated that the subject exemption would allow nuclear reactors licensed under 10 CFR Part 52 to be explicitly excepted from the requirements of 10 CFR 70.22(b), 10 CFR 70.32(c), 10 CFR 74.31, 10 CFR 74.41, and 10 CFR 74.51. There is no technical or regulatory basis to treat nuclear reactors licensed under 10 CFR Part 52 differently than reactors licensed under 10 CFR Part 50 with respect to the MC&A provisions in 10 CFR Part 74. As indicated in the Statement of Considerations for 10 CFR 52.0(b) (*72 Federal Register* 49352, 49372, 49436 (August 28, 2007)), applicants and licensees under 10 CFR Part 52 are subject to all of the applicable requirements in 10 CFR Chapter I, whether or not those provisions explicitly mention a COL under 10 CFR Part 52. This regulation clearly indicates that plants licensed under 10 CFR Part 52 are to be treated no differently than plants licensed under

¹¹ While not including an explicit exception for 10 CFR Part 50 reactors, 10 CFR 74.33 applies only to uranium enrichment facilities and thus is not directly implicated in this exemption request.

10 CFR Part 50 with respect to the substantive provisions in 10 CFR Chapter I (which includes 10 CFR Parts 70 and 74). In particular, the exception for nuclear reactors licensed under 10 CFR Part 50, as in 10 CFR 70.22(b), 10 CFR 74.31, 10 CFR 74.41, or 10 CFR 74.51, should also be applied to reactors licensed under 10 CFR Part 52.

The staff agrees with the applicant's justification that nuclear reactors licensed under 10 CFR Part 52 should be treated the same as the reactors licensed under 10 CFR Part 50 regarding the MC&A for SNM.

Pursuant to 10 CFR 70.17(a), the Commission may, upon application of any interested person or upon its own initiative, grant such exemptions from the requirements of the regulations in this part as it determines are authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest.

In addition, pursuant to 10 CFR 74.7, the Commission may, upon application of any interested person or upon its own initiative, grant such exemptions from the requirements of the regulations in this part as it determines are authorized by law and will not endanger life or property or the common defense and security, and are otherwise in the public interest.

Pursuant to 10 CFR 52.7, the Commission may, upon application by any interested person or upon its own initiative, grant exemptions from the requirements of 10 CFR Part 52. 10 CFR 52.7 further states that the Commission's consideration will be governed by 10 CFR 50.12, "Specific exemptions," which states that an exemption may be granted when (1) the exemptions are authorized by law, will not present an undue risk to public health or safety, and are consistent with the common defense and security and (2) when special circumstances are present. Special circumstances are present whenever, according to 10 CFR 50.12(a)(2)(ii), "Application of the regulation in the particular circumstances would not serve the underlying purpose of the rule or is not necessary to achieve the underlying purpose of the rule."

The NRC staff reviewed the subject exemption, which will allow the applicant to have a similar exception for the COL under 10 CFR Part 52, such that the same regulations will be applied to the SNM MC&A program as nuclear reactors licensed under 10 CFR Part 50, and determined that this requested exemption will not present an undue risk to the public health and safety and is otherwise in the public interest. In addition, this exemption is consistent with the AEA and is authorized by law. Therefore, granting this exemption will not adversely affect the common defense and security. Further, the application of the regulation in these particular circumstances is not necessary to achieve the underlying purpose of the rule. Since the exemption criteria in 10 CFR 50.12 are satisfied, the staff considers that this request also demonstrates that the exemption criteria in 10 CFR 52.7, 10 CFR 70.17(a), and 10 CFR 74.7 are satisfied. Therefore, the staff finds that the exemption from 10 CFR 70.22(b), 10 CFR 70.32(c) and, in turn, 10 CFR 74.31, 10 CFR 74.41, and 10 CFR 74.51, is justified.

1.5.5 Receipt, Possession, Use, and Transport of Source, Byproduct and Special Nuclear Material Authorized by 10 CFR Part 52 Combined Licenses

In Part 1, "General and Financial Information," of the COL application, FPL requested material licenses for receipt, possession, and use of source, byproduct, and SNM in accordance with Commission regulations in 10 CFR Parts 30, 40, and 70. The reviews that the NRC conducted for compliance with the requirements of 10 CFR Part 52 to support the issuance of the COL

encompass those reviews necessary to support granting 10 CFR Parts 30, 40, and 70 licenses. In this respect, the 10 CFR Part 52 COLs for Turkey Point will be consistent with the approach to 10 CFR Parts 30, 40, and 70 licensing followed for operating licenses for nuclear power plants licensed in accordance with 10 CFR Part 50. The staff considered the following standard license provisions for the Turkey Point COL as it relates to authorization pursuant to the regulations in 10 CFR Parts 30, 40, and 70:¹²

Subject to the conditions and requirements incorporated herein, the Commission hereby licenses FPL:

- (1) (a) pursuant to the Act and 10 CFR Part 70, to receive and possess at any time, special nuclear material as reactor fuel, in accordance with the limitations for storage and in amounts necessary for reactor operation, described in the FSAR, as supplemented and amended;

(b) pursuant to the Act and 10 CFR Part 70, to use special nuclear material as reactor fuel, after a Commission finding under 10 CFR 52.103(g) has been made, in accordance with the limitations for storage and amounts necessary for reactor operation, described in the FSAR, as supplemented and amended;
- (2) (a) pursuant to the Act and 10 CFR Parts 30 and 70, to receive, possess, and use, at any time before a Commission finding under 10 CFR 52.103(g), such byproduct and special nuclear material (but not uranium hexafluoride) as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts not exceeding those specified in 10 CFR 30.35(d) and 10 CFR 70.25(d) for establishing decommissioning financial assurance, and not exceeding those specified in 10 CFR 30.72 and 10 CFR 70.22(i)(1);

(b) pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use, after a Commission finding under 10 CFR 52.103(g), any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as necessary;
- (3) (a) pursuant to the Act and 10 CFR Parts 30 and 70, to receive, possess, and use, before a Commission finding under 10 CFR 52.103(g), any byproduct or special nuclear material (but not uranium hexafluoride) that is (1) in unsealed form; (2) on foils or plated surfaces, or (3) sealed in glass, for sample analysis or instrument calibration or other activity associated with radioactive apparatus or components; in amounts not exceeding those specified in 10 CFR 30.35(d) and 10 CFR 70.25(d) for establishing

¹² These proposed standard license conditions that the staff considered are based on similar license conditions found in the draft Combined Licenses for LNP Units 1 and 2.

decommissioning funding assurance, and not exceeding those specified in 10 CFR 30.72 and 10 CFR 70.22(i)(1);

(b) pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use, after a Commission finding under 10 CFR 52.103(g), in amounts as necessary, any byproduct, source, or special nuclear material (but not uranium hexafluoride) without restriction as to chemical or physical form, for sample analysis or instrument calibration or other activity associated with radioactive apparatus or components; and

(4) pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

The staff notes that Turkey Point COL FSAR Table 13.4-201, “Operational Programs Required by NRC Regulations,” provides milestones for the implementation of various operational programs. Important milestone dates for various operational programs that support issuance of the license and requirements relative to 10 CFR Parts 30, 40, and 70 include the following:

- Fire Protection Program—prior to initial receipt of byproduct, source, or SNM (excluding exempt quantities as described in 10 CFR 30.18, “Exempt quantities”).
- Radiation Protection Program (including as low as is reasonably achievable [ALARA] principles) - prior to initial receipt of byproduct, source, or SNM (excluding exempt quantities as described in 10 CFR 30.18).
- Non-licensed plant staff training program associated with receipt of the radioactive material – prior to initial receipt of byproduct, source, or SNM (excluding exempt quantities as described in 10 CFR 30.18).
- Security Program including physical security, safeguards contingency program, training and qualification program – prior to receipt of fuel onsite (protected area)
- Security Program including physical security, safeguards contingency, and transportation programs – prior to transport or receipt of SNM of low strategic significance
- SNM MC&A Program – prior to receipt of SNM.

As documented in Table 1.3, below, the Turkey Point applicant endorsed VEGP standard content letters related to SNM MC&A.

Table 1-3. Turkey Point Applicant Endorsements of VEGP COL Standard Content Letters

VEGP Letter Date	VEGP Letter ADAMS Accession Number	Turkey Point Endorsement Letter Date	Turkey Point Letter ADAMS Accession Number
July 29, 2009	ML092120064	February 22, 2010	ML100560114

VEGP Letter Date	VEGP Letter ADAMS Accession Number	Turkey Point Endorsement Letter Date	Turkey Point Letter ADAMS Accession Number
October 15, 2010	ML102920120	April 20, 2011	ML11111A127
November 23, 2010	ML103300034	April 19 and 20, 2011	ML11111A127
March 3, 2011	ML110660153	September 14, 2011	ML11259A054
March 16, 2011	ML110800088	September 14, 2011	ML11259A054
March 16, 2011	ML110770137	December 16, 2011	SGI
May 6, 2011	ML11129A155	September 14, 2011	ML11259A054
June 22, 2011	ML11175A169	September 14, 2011	ML11259A054

These letters identify the portions of the Turkey Point COL application that satisfy the basis for meeting the requirements of 10 CFR Parts 30, 40, 70, and 74. Section 1.5.4 of this report addresses the exemption request from the requirements of 10 CFR 70.22(b), 10 CFR 70.32(c) and, in turn, 10 CFR 74.31, 10 CFR 74.41, and 10 CFR 74.51.

Section 1.2.3 of this SER provides a discussion of the strategy used by the NRC to perform one technical review for each standard issue outside the scope of the DC and use this review in evaluating subsequent COL applications. To ensure that the staff's findings on standard content that were documented in the SER for the reference COL application (VEGP Units 3 and 4) were equally applicable to the Turkey Point Units 6 and 7 COL application, the staff undertook the following reviews:

- The staff compared the VEGP COL FSAR, Revision 5 to the Turkey Point COL FSAR. In performing this comparison, the staff considered changes made to the Turkey Point COL FSAR (and other parts of the COL application, as applicable) resulting from RAIs.
- The staff confirmed that all responses to RAIs identified in the corresponding standard content evaluation were endorsed.
- The staff confirmed that the Turkey Point Units 6 and 7 COL application, Part 11, contains the same SNM MC&A Program description as that submitted by Southern Nuclear Operating Company (SNC) for VEGP Units 3 and 4 in a letter dated November 23, 2010.
- The staff confirmed that the Turkey Point Units 6 and 7 COL application, Part 11, Enclosure 6 contains the same New Fuel Shipping Plan as that submitted by SNC in the VEGP Units 3 and 4 COL application.
- The staff confirmed that the supplemental information in support of 10 CFR Part 70 SNM license application found in Part 11 of the VEGP COL application is identical to the material found in Part 11 of the Turkey Point Units 6 and 7 COL application.
- The staff verified that site-specific differences were not relevant and, where the staff identified relevant differences, the staff performed additional review to determine the acceptability of the differences.

The staff has completed its review and found the evaluation performed for the standard content to be directly applicable to the Turkey Point COL application, with the site-specific exceptions noted. This standard content material is identified in this SER by use of italicized, double-indented formatting. Section 1.2.3 of this SER provides an explanation of why the standard content material from the SER for the reference COL application (VEGP) includes evaluation material from the SER for the BLN Units 3 and 4 COL application.

The following portion of this technical evaluation section is reproduced from Section 1.5.5 of the VEGP SER:

In addition to the evaluation of the implementation milestones noted above, the staff's evaluation of the radiation protection program that supports the issuance of the 10 CFR Parts 30, 40, and 70 licenses is addressed in Chapter 12 of this SER. Additional staff evaluations that support the issuance of the 10 CFR Part 70 license are addressed in Chapter 9 of this SER (i.e., new fuel storage, spent fuel storage, and fire protection programs) and in the staff's evaluation of TVA's security program. The staff finds that the information in the Bellefonte COL application to support granting of the 10 CFR Part 70 license mentioned as part of the license above is sufficient, pending resolution of the open items in this report related to new and spent fuel, fire protection program, security program, and the implementation of the fire protection and security programs. However, TVA needs to provide a discussion of which parts of its COL application other than the reference to the radiation protection program provide sufficient information to support compliance with the applicable portions of 10 CFR Part 30 and 40, prior to the 10 CFR 52.103(g) finding. This is Open Item 1.5-1.

Resolution of Standard Content Open Item 1.5-1

In letters dated July 29, 2009, July 9, 2010, and October 15, 2010, the applicant provided additional information related to source, byproduct and SNM and its purposes, radiation safety personnel, personnel training, facilities and equipment, waste management, and the radiation safety program in general.

Subsequent to the issuance of the SER with open items for the BLN application, the staff performed an additional review associated with granting the 10 CFR Parts 30, 40 and 70 licenses. For the 10 CFR Part 70 license, the staff considered SNM associated with the fuel (including security requirements) and SNM associated with non-fuel material (i.e., fission chambers). The staff also considered emergency plan requirements associated with SNM (fuel and non-fuel material). Based on these reviews, standard content Open Item 1.5-1 is resolved. These reviews are described below.

Review of Parts 30 and 40 Materials

In a letter dated March 3, 2011, the applicant provided information regarding specific types of sources and byproduct material, the chemical or physical form, and the maximum amount at any time for the requested material licenses under 10 CFR Parts 30 and 40. The applicant also stated that SNM shall be in the form of reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the VEGP COL FSAR. Byproduct material and source material shall be in the form of sealed neutron sources for reactor startup and sealed sources for reactor instrumentation, radiation monitoring equipment, calibration, and fission detectors in amounts as required. The applicant also committed that no 10 CFR Part 40 specifically licensed source material, including natural uranium, depleted uranium and uranium hexafluoride will be received, possessed, or used during the period between issuance of the COL and the Commission's 10 CFR 52.103(g) finding for each of the VEGP Units 3 and 4. The applicant also stated that the quantity of any byproduct material with atomic numbers 1 through 93 would not exceed 100 millicuries for a single source and 5 Curies total. The maximum quantity for Americium-241 would not exceed 300 millicuries for single source and 500 millicuries total. Following the 10 CFR 52.103(g) finding for each of the VEGP Units 3 and 4, byproduct material, source material, and SNM in amounts as required, without restriction to chemical forms or physical form, would be used for the following:

- *Sample analysis,*
- *Instrument and equipment calibration, and*
- *Associated with radioactive apparatus or components.*

With respect to the requirements of 10 CFR Parts 30, 40, and 70 that are related to radiation protection (including administrative controls), the applicant provided information (in letters dated July 9, and November 23, 2010) on the purpose, storage and security of sources in VEGP COL FSAR Sections 12.2 and 12.5. Information related to the radiation protection program itself, including procedures for the use of these sources, is also described in VEGP COL FSAR Chapter 12. In addition, VEGP COL FSAR Section 13.4 states that the radiation protection program will be implemented according to the milestones listed in VEGP COL FSAR Table 13.4-201, Item 10. These milestones ensure that those portions of the program necessary to comply with the requirements of 10 CFR Parts 20, 30, 40, and 70, are implemented prior to the receipt of byproduct, source, SNM, or fuel, onsite.

The staff finds that the information provided by the applicant that describes the radiation protection measures (Chapter 12 of the VEGP COL FSAR) that will be implemented prior to receipt of byproduct, source or SNM, conforms to the applicable guidance in NUREG-1556, "Consolidated Guidance about Materials Licenses," and is therefore acceptable. The radiation protection program milestones included in the VEGP COL FSAR Table 13.4-201 are evaluated in Section 12.5 of this SER.

In a letter dated July 9, 2010, the applicant provided supplemental information relative to Item 14, Emergency Planning, in VEGP COL FSAR Table 13.4-201.

In addition, the applicant proposed to revise the term 'portions applicable to SNM' to 'portions applicable to radioactive materials' for Item 14; Item 8, Fire Protection Program; Item 11, Non-Licensed Plant Staff Training Program; and Item 15, Physical Security Program. In addition, the applicant proposed to correct the references to regulatory citations of 10 CFR 30.32, "Application for specific licenses"; 10 CFR 40.31, "Application for specific licenses"; and 10 CFR 70.22, "Contents of applications." It also proposed to revise the "Requirements" column for Item 14 of the VEGP COL FSAR Table 13.4-201 to reference 10 CFR 30.32(i)(1), 10 CFR 40.31(j)(1), and 10 CFR 70.22(i)(1). It also proposed to revise Part 10 of the VEGP COL application, Proposed License Condition 3, "Operational Program Implementation," Section C, "Receipt of Materials," to include implementation of the portions of the emergency planning program applicable to SNM. In addition to the evaluation of the implementation milestones noted above, the staff's evaluation that supports the issuance of the 10 CFR Parts 30 and 40 licenses is addressed in Chapter 9 (the fire protection program).

The operational programs are specific programs that are required by regulations. VEGP COL FSAR Table 13.4-201 lists each operational program, the regulatory source for the program, the section of the FSAR in which the operational program is described, and the associated implementation milestone(s). The applicant proposed a license condition in Part 10, License Condition 3, Item C.3 of the VEGP COL application, which provides the milestones for implementing the portions of the non-licensed plant staff training program applicable to receipt of the radioactive material. However, Table 13.4-201 specifies implementation requirements (10 CFR 30.32(a), 10 CFR 40.31(a), and 10 CFR 70.22(a)) for the non-licensed plant staff training program associated with receipt of the radioactive material. Therefore, the staff determined that Item C.3 of proposed License Condition 3 is not needed because the implementation milestones for the non-licensed plant staff training program associated with receipt of radioactive material are governed by the applicable regulations.

The applicant proposed a license condition in Part 10 of the VEGP COL application to provide a schedule to support the NRC's inspection of operational programs, including the non-licensed plant staff training program applicable to receipt of the radioactive material. The proposed license condition is consistent with the policy established in SECY-05-0197, "Review of Operational Programs in a Combined License Application and Generic Emergency Planning Inspections, Tests, Analyses, and Acceptance Criteria," for operational programs and is acceptable.

In response to RAI 1.5-1, the applicant stated, in a letter dated October 15, 2010, that no byproduct material will be received, possessed, or used at AP1000 units of a physical form that is in unsealed form, on foils or plated sources, or sealed in glass, that exceeds the quantities in Schedule C of 10 CFR 30.72. Since the quantities do not exceed Schedule C, an emergency plan that meets the requirements of 10 CFR 30.32(i)(3) is not required. As such, the implementation of the emergency plan prior to the receipt of byproduct material will be removed from VEGP COL FSAR Table 13.4-201 and from Part 10 proposed License Condition 3, Item C.4. The request for a 10 CFR Part 40 license does not involve

authorization to receive, possess, or use uranium hexafluoride in excess of 50 kilograms in a single container or 1000 kilograms total. However, in a letter dated March 3, 2011, the applicant revised the request for a 10 CFR Part 40 license to state that no 10 CFR Part 40 specifically-licensed source material, including natural uranium, depleted uranium and uranium hexafluoride (UF_6), will be received, possessed, and used during the period between issuance of the COL and the Commission's 10 CFR 52.103(g) finding for each of the VEGP Units 3 and 4. Since the above quantities are not exceeded, an emergency plan for responding to the radiological hazards of an accidental release of source material and to any associated chemical hazards related to the material is not required. As such, the implementation of the emergency plan prior to the receipt of source material will be removed from VEGP COL FSAR Table 13.4-201. This applicant's proposal meets the requirements of 10 CFR 30.32 and 10 CFR 40.31 and is, therefore, acceptable. The incorporation of changes into a future revision of the VEGP COL FSAR is **Confirmatory Item 1.5-1**.

Resolution of Standard Content Confirmatory Item 1.5-1

Confirmatory Item 1.5-1 is an applicant commitment to revise FSAR Table 13.4-201. The staff verified that the VEGP COL FSAR Table 13.4-201 was appropriately revised. As a result, Confirmatory Item 1.5-1 is now closed.

The applicant also proposed an FSAR commitment to address the limitations during the period prior to the implementation of the emergency plan. In a letter dated March 16, 2011, the applicant stated that it has no plans to process UF_6 at the plant site at any time following the Commission's 10 CFR 52.103(g) finding and consequently does not expect the requested 10 CFR Part 40 license to include receipt, storage, or use of UF_6 at the plant site. However, using the guidance of DC/COL-ISG-15, "Post-Combined License Commitments," the staff has determined that the commitment is not sufficient and instead the staff is proposing to add a restriction in the license condition related to 10 CFR Parts 30 and 40. (See License Condition 1-1,c(ii)).

Review of Part 70 Materials

The staff reviewed information related to nuclear fuel as SNM included in the VEGP COL application including the AP1000 DCD against 10 CFR Part 70 requirements. Specifically, the staff's review included:

- *General information—financial qualification, site description, hydrology, geology, meteorology, the nearby population, and potential effects of natural phenomena (Part 1 of the application, FSAR Section 1.1 and Chapter 2, Section 4.1 and Table 4.1-1 of the AP1000 DCD against the requirements of 10 CFR 70.22(a)(1) through (a)(4));*
- *Organization and Administration—the responsibilities and associated resources for the receipt, possession, inspection, and storage of the SNM in the form of fresh fuel assemblies (Part 1 of the application, Quality Assurance Program included in Part 11 (Enclosure 11A) of the*

application, VEGP COL FSAR Section 13.1 for organization against the requirements of 10 CFR 70.22(a)(6) and (a)(8));

- *Radiation Protection—Radiation protection program implementation, organization and personnel qualification, written procedures, ALARA, radiation survey and monitoring (AP1000 DCD Section 9.1 and Chapter 12 of VEGP COL FSAR against the requirements of 10 CFR 70.22(a)(6) through (a)(8));*
- *Nuclear Criticality Safety—use of area radiation monitors in lieu of criticality accident alarms (AP1000 DCD Sections 9.1.1.3 and 11.5.6 against the requirements of 10 CFR 70.22(a)(6) through (a)(8) and 10 CFR 50.68(b));*
- *Fire safety—fire protection program (VEGP COL FSAR Section 9.5.1 and Table 13.4-201 against the requirements of 10 CFR 70.22(a)(6) through (a)(8));*
- *Emergency Preparedness—emergency preparedness program for the VEGP site (VEGP COL FSAR Section 13.3 and Table 13.4-201 and the Emergency Plan against the requirements of 10 CFR 70.22(i));*
- *Environmental Protection—organization, procedures and controls that ensures that the environment is protected during the conduct of activities (i.e., receipt, possession, inspection, and storage of SNM (VEGP COL FSAR Section 11.5 and AP1000 DCD Sections 9.1.1 and 11.5 against the requirements of 10 CFR 70.22(a)(7) and (a)(8)); and*
- *MC&A Program and Security (MC&A program included in the application against requirements of 10 CFR 70.22(b) and 10 CFR Part 74 and the Physical Security Plan (PSP) against the requirements of 10 CFR 73.67, “Licensee fixed site and in-transit requirements for the physical protection of special nuclear material of moderate and low strategic significance”).*

As indicated above, the applicant’s compliance with several applicable 10 CFR Part 70 requirements regarding radiation protection, nuclear criticality safety, and environmental protection is already encompassed by the design information incorporated by reference from the AP1000 DCD and evaluated by the staff as part of the design certification proceeding. As explained further below, with respect to other applicable 10 CFR Part 70 requirements to be addressed by the COL applicant, the staff finds that the information provided regarding general information, organization and administration, radiation protection, nuclear criticality safety, fire safety, emergency preparedness, and environmental protection to support receipt, storage, and possession of SNM, conforms to the applicable guidance in NUREG-1520 and NUREG-0800 and, therefore, is acceptable. First, however, the staff’s review of information regarding the MC&A program (10 CFR 70.22(b) and 10 CFR Part 74) and the PSP (10 CFR 73.67) is provided below.

MC&A Program for SNM (Fuel)

In RAI 1.5-3, the staff requested the applicant to review the requirements of 10 CFR 70.22(b) for the program addressing the control and accounting of SNM and provide descriptions of how the applicable requirements for material accounting and controls under 10 CFR Part 74 will be met for the possession and storage of SNM during construction and prior to the operation of the nuclear power plant. In addition, the staff requested the applicant to provide a proposed license condition to clearly establish full implementation of the MC&A program meeting the applicable requirements of 10 CFR Part 74 prior to receipt of SNM, consistent and concurrent with the proposed license condition for implementing the applicable security (i.e., physical protection) requirements of 10 CFR Part 73.

In response to RAI 1.5-3, the applicant, in a letter dated November 23, 2010, stated that all non-irradiated SNM for the AP1000 units is identified as Category III, SNM of low strategic significance, as defined in 10 CFR 74.4, "Definitions." No SNM at an AP1000 nuclear facility will exceed an uranium-235 isotope enrichment of 10 percent. The quantity of SNM will be documented, controlled, and communicated to the NRC as required in 10 CFR 74.13, "Material status reports"; 10 CFR 74.15, "Nuclear material transaction reports"; and 10 CFR 74.19, "Recordkeeping."

Turkey Point Units 6 and 7 COL application includes Part 11F, "Supplemental Information of 10 CFR Part 70 Special Nuclear Material License Application," acknowledging that Turkey Point Units 6 and 7 would possess uranium sources containing uranium enriched to 93 percent uranium-235 in a quantity meeting the criteria of SNM of low strategic significance.

The following portion of this technical evaluation section is reproduced from Section 1.5.5 of the VEGP SER:

In its response to RAI 1.5-3, the applicant also described the SNM MC&A program and stated that this program will be provided as an enclosure in the VEGP COL application, Part 11. The SNM MC&A program will be developed for control and accounting of SNM in accordance with the applicable requirements of 10 CFR Part 74, Subparts A and B. This program will be consistent with guidance of American National Standards Institute (ANSI) 15.8-2009, "Material Control Systems – Special Nuclear Material Control and Accounting Systems for Nuclear Power Plants." The SNM MC&A program will be implemented prior to receipt of SNM at the plant site and will remain in effect until the SNM is shipped from the plant site. The procedures constituting the SNM MC&A program will delineate the requirements, responsibilities, and methods of SNM control necessary to address the following programmatic elements:

- 1. Establish, maintain, and follow written MC&A procedures to account for SNM.*
- 2. Maintain adequate records of the initial receipt or current inventory of SNM, including records of isotopic content, material received, material shipped, and material lost (material balance reports and physical inventory listing reports).*

3. *Develop adequate inventory procedures and maintain adequate perpetual inventory records.*
4. *Inventory SNM within the 12-month prescribed frequency.*
5. *Report SNM inventories on the applicable forms.*
6. *Establish an individual responsible for the control and accountability of SNM.*
7. *Report the loss of or inability to find SNM items in a timely manner.*
8. *Control access to SNM.*
9. *Control the shipping and transfer of SNM.*

The applicant proposed to add a new FSAR Section 13.5.2.2.9, which will summarize the use of plant procedures to address MC&A of SNM. The applicant also stated that VEGP COL FSAR Table 13.4-201 will be revised to provide information related to implementation of the SNM MC&A program.

In order to address the applicable 10 CFR Part 74 MC&A requirements prior to power operation, the applicant proposed a license condition that will require implementation of a MC&A program prior to receipt of SNM on site. Implementation of the SNM MC&A program prior to SNM receipt will also address the SNM possession and storage requirements during construction and prior to operation of the nuclear power plant.

*The applicant's MC&A program for SNM is consistent with ANSI 15.8 and meets reporting and recordkeeping requirements of 10 CFR 74.11, "Reports of loss or theft or attempted theft or unauthorized production of special nuclear material"; 10 CFR 74.13; 10 CFR 74.15; and 10 CFR 74.19. The documentation, submitted by the applicant, for a program addressing the control and accounting of SNM provided descriptions of how the applicable requirements for material accounting and controls under 10 CFR Part 74 are met and, therefore, is acceptable, subject to the proposed revision to the VEGP COL application and the VEGP COL FSAR (this has been tracked as **Confirmatory Item 1.5-2**). In addition, the proposed license condition includes a provision to provide a schedule to support the NRC's inspection of the MC&A program for the SNM. This is consistent with the policy established in SECY-05-0197 and is thus acceptable.*

Resolution of Standard Content Confirmatory Item 1.5-2

Confirmatory Item 1.5-2 is an applicant commitment to revise FSAR Sections 13.4, 13.5 and Parts 7 and 11 (Enclosure 11D of its application to address the SNM MC&A program. The staff verified that the VEGP COL FSAR and Parts 7 and 11 (Enclosure D) [Part 11 of the LNP COL application] of its application were appropriately revised. As a result, Confirmatory Item 1.5-2 is now closed.

The following portion of this technical evaluation section is reproduced from Section 1.5.5 of the VEGP SER. Portions of the standard content review addressing SNM physical protection superseded by the staff's review of additional site-specific information have been deleted from the standard content review appearing below. The staff review of the additional site-specific information, including a revised Special Nuclear Material Physical Protection Program Description (SNMPPPD), appears below, following the review of the standard content material.

Security Review for 10 CFR Part 70 Materials

[Standard content deleted as noted above]

In a letter dated March 15, 2011, the NRC staff asked the applicant to provide its plan regarding the protection of new fuel as SNM at the VEGP Units 3 and 4 plant site prior to declaration of an operational protected area (PA) and implementation of the requirements of 10 CFR 73.55, as described in the SNM MC&A Program description. In addition, the staff also requested that the applicant consider the applicability of the substantive provisions of interim compensatory orders (ICMO) that were issued to Category III Fuel Cycle Facilities to ensure adequate protection when SNM is on site prior to the activation of the PA.

[Standard content deleted as noted above]

The staff raised a question regarding the licensee's ability to receive new fuel and return new fuel rods/assemblies to the fuel manufacturer. In a letter dated May 6, 2011, the applicant proposed to revise its FSAR Section 13.5.2.2.8 to include the New Fuel Shipping Plan that addresses the applicable 10 CFR 73.67 requirements in the event that unirradiated new fuel assemblies or components are returned to the supplying fuel manufacturer(s) facility. The New Fuel Shipping Plan summarizes the procedures and the written agreement that the applicant will have in place prior to shipment of new fuel back to the fuel manufacturer and this plan will be included in Part 11, Enclosures of its application. The staff finds this New Fuel Shipping Plan acceptable because it meets the applicable requirements of 10 CFR 73.67(g). The staff verified that the VEGP FSAR Section 13.5 and Part 11 (Enclosure E) are appropriately updated.

[Standard content deleted as noted above]

In addition, the applicant has adequately addressed security issues related to; security response procedures, coordination with local law enforcement for

response support, storage of hazardous materials on-site, review of emergency shutdown/cool down procedures, supplementing of the Emergency Actions Levels, site accountability and evacuation strategies, emergency communications, evaluation of computer and communications networks for vulnerabilities, capabilities to provide fire suppression, evaluation of the need for offsite medical support, emergency support, and access to Federal support, and limiting public access to sensitive plant information.

[Standard content deleted as noted above]

Non-Fuel SNM

In a letter dated, June 22, 2011, the applicant provided information regarding the name, amount, and specifications (including the chemical and physical form and, where applicable, isotopic content) of the non-fuel SNM (Fission Chambers) the applicant proposes to use (10 CFR 70.22(a)(4)). The letter also provided information to confirm that the applicable design and programmatic elements provided in the licensing basis will satisfy the requirements in 10 CFR 70.22(a)(6) through (8) prior to receipt of non-fuel SNM.

10 CFR Part 70 Requirements - Other than MC&A (10 CFR 70.22(b) and 10 CFR Part 74) and Security (10 CFR 73.67) - for Fuel and Non-Fuel Material

As noted above, in addition to MC&A and security, the staff also examined the applicant's compliance with 10 CFR Part 70 requirements regarding general information, organization and administration, radiation protection, nuclear criticality safety, fire safety, emergency preparedness, and environmental protection to support receipt, storage, and possession of SNM.

The staff's analysis follows with respect to those other requirements not already resolved via the applicant's incorporation of the AP1000 DCD. For the reasons described in Section 1.4.4 of this FSER the staff agrees that the applicant is technically qualified to engage in the proposed activities associated with this license, based on the applicant's ongoing experience in the safe operation of nuclear power plants, as presented in Section 1.4.1 of the VEGP COL FSAR. Likewise, the applicant's financial qualifications and ownership structure meet the requirements of 10 CFR 70.22 for the same reasons described above in Section 1.5.1.

Note: Turkey Point COL FSAR Section 1.4.1 has a similar discussion regarding the applicant's operation of its other nuclear power plants. The staff also concludes the applicant is technically qualified to engage in the proposed activities associated with this license based on the applicant's ongoing experience with the safe operation of its other nuclear power plants. In addition, Section 1.5.1 of this report finds that the financial qualifications for the Turkey Point COL application are acceptable.

The following portion of this technical evaluation section is reproduced from Section 1.5.5 of the VEGP SER:

Similarly, the applicant has explained the anticipated amounts, types, and uses of 10 CFR Part 70 materials at the site are consistent with the provisions of 10 CFR 70.22. The VEGP COL FSAR and Part 1 of the application provide adequate description of the VEGP Units 3 and 4 facility and the proposed activities related to 10 CFR Parts 30, 40 and 70 material. In addition the VEGP COL FSAR provides information regarding regional hydrology, geology, meteorology, the nearby population, and potential effects of natural phenomena that could occur at the facility. The applicant has described the responsibilities and associated resources (see Part 1, "General and Administration Information," and Enclosure 11A, "Nuclear Development Quality Assurance Manual" of the application) for the receipt, possession, inspection, and storage of the 10 CFR Part 70 material (fuel and non-fuel). Therefore, it meets the requirements of 10 CFR 70.22(a)(1). Furthermore, as indicated in VEGP COL FSAR Table 13.4-201, applicable portions of the Radiation Protection Program will be implemented prior to initial receipt of byproduct, source, or SNMs. In accordance with VEGP COL FSAR Table 13.4-201, Item 10, Implementation Milestone #1, and the NRC-approved template, Nuclear Energy Institute (NEI) 07-03A, "Generic FSAR Template Guidance for Radiation Protection Program Description," which is incorporated by reference into VEGP COL FSAR Appendix 12AA (see SER Section 12.5), the appropriate radiation protection program elements associated with organization, facilities, instrumentation and equipment, procedures (e.g., procurement, receipt, inventory, labeling, leak testing, surveillance, control, transfer, disposal, storage, issuance, and use of radioactive sources), and training will be in place prior to initial receipt of byproduct, source, or special nuclear materials, thereby satisfying the requirements of 10 CFR 70.22(a)(4), (6), (7), and (8). VEGP COL FSAR Section 12.2 includes the requirements for written procedures that address leak-testing of radioactive sources. The leak-test will be consistent with 10 CFR 20.1501, "General," survey and monitoring requirements for evaluating the quantities of radioactive material and the potential radiological hazard of the radioactive source.

The fission chambers will be disposed of consistent with the operating procedures that specify the processes to be followed to ship waste that complies with the waste acceptance criteria (WAC) of the disposal site, the waste classification and characteristics requirements of 10 CFR 61.55, "Waste classification," and 10 CFR 61.56, "Waste characteristics," and the requirements of third party waste processors as applicable. This process is identified in VEGP COL FSAR Section 11.4.6.1.

With respect to fire safety, prior to installation, the new fission chambers (along with the new fuel) will be stored in the Auxiliary Building fuel handling area, which is an area protected by the fire protection program and fire protection system, as discussed in the AP1000 DCD Section 9A.3.1.3.1.2. Temporary storage of these non-combustible sealed sources is not specifically addressed in the AP1000 fire protection analysis in DCD Appendix 9A; however, the approach to extinguishing fires and containing material releases associated with the fission chambers would be similar to, and bounded by, the approach considered for the fuel handling area in general. The fuel handling area has been evaluated and determined acceptable for the storage of SNM in a full core load of new fuel. The

hazards imposed by the relatively small quantity of SNM associated with the fission chambers (less than 100 grams), is not expected to be a challenge to the existing fire protection analysis for the new fuel storage (see Section 9.5.1 of this SER). The VEGP COL FSAR Section 12.2 includes the requirements for written procedures that address leak testing of radioactive sources (byproduct, source, and devices that contain SNM, as appropriate). Further, the fission chambers that contain the non-fuel SNM are sealed sources that are tested periodically to confirm their leak-tightness. Therefore, it is expected that the capabilities of the fire protection program and the fire protection equipment servicing this area are sufficient to meet the requirements of 10 CFR 70.22(a)(7) and 10 CFR 70.22(a)(8).

Emergency Plan (SNM, Fuel, and Non-Fuel)

The applicant will be storing the new fuel in the new fuel racks (stored dry) or in the spent fuel racks prior to loading into the reactor. The safety analysis included in AP1000 DCD Sections 9.1.1.3 and 9.1.2.3 provides safety analysis that indicates that: (1) the design of new fuel rack is such that K_{eff} remains less than or equal to 0.95 with full density unborated water and less than equal to 0.98 with optimum moderation and full reflection conditions; and (2) the design of spent fuel rack is such that K_{eff} remains less than or equal to 0.95 under design basis conditions. This criticality evaluation meets requirements of 10 CFR 50.68(b). Therefore, a criticality accident alarm system to meet the requirements of 10 CFR 70.24, "Criticality accident requirements," is not required. As a result, an emergency plan (to receive and possess) pursuant to 10 CFR 70.22(i) is also not required. In addition, an emergency plan for the fission chambers (to receive and possess) pursuant to 10 CFR 70.22(i) is not required due to the small quantity of SNM (less than 100 grams) associated with the fission chambers.

1.5.5.1 Physical Protection of Special Nuclear Material

1.5.5.1.1 Introduction

This section addresses the physical protection of SNM while possessed, used, and transported by the applicant, including during the period prior to implementation of the site PSP.

1.5.5.1.2 Summary of Application

The applicant submitted a SNMPPPD, Revision 7, dated September 24, 2015 (Safeguards LAN Electronic Safe (SLES) Accession No. NS113247).

1.5.5.1.3 Regulatory Basis

The regulatory requirements and guidance applicable to fixed site and in-transit physical protection are as follows:

- 10 CFR 73.67, "Licensee fixed site and in-transit requirements for the physical protection of special nuclear material of moderate and low strategic significance."
- RG 5.66, "Access Authorization Program for Nuclear Power Plants," Revision 1, July 2009.

- Post-September 11, 2001, Security Orders for SNM of Low Strategic Significance.
- RG 5.59, "Standard Format and Content for a Licensee Physical Security Plan for the Protection of Special Nuclear Material of Moderate or Low Strategic Significance," Revision 1, 1983.
- RIS 2005-22, "Requirements for the Physical Protection During Transportation of Special Nuclear Material of Moderate and Low Strategic Significance: 10 CFR Part 73 vs. Regulatory Guide 5.59 (1983)."

1.5.5.1.4 Technical Evaluation

The staff performed a technical evaluation of the Turkey Point Units 6 and 7 COL application against applicable 10 CFR 73.67 fixed-site and in-transit general performance objectives, general requirements, and physical protection requirements for SNM of low strategic significance. In addition, the staff requested information related to how the applicant addressed the post September 11, 2001, security order measures for SNM of low strategic significance. The staff also conducted a technical evaluation of how the order measures were addressed.

1.5.5.1.4.1 Fixed-Site General Performance Objectives

The applicable physical protection requirements specified in 10 CFR 73.67 include the following general performance objectives for fixed sites as stated in 10 CFR 73.67(a)(1):

- (1) Each licensee who possesses, uses or transports special nuclear material of moderate or low strategic significance shall establish and maintain a physical protection system that will achieve the following objectives:
 - (i) Minimize the possibilities for unauthorized removal of special nuclear material consistent with the potential consequences of such actions; and
 - (ii) Facilitate the location and recovery of missing special nuclear material.
- (2) To achieve these objectives, the physical protection system shall provide:
 - (i) Early detection and assessment of unauthorized access or activities by an external adversary within the controlled access area containing special nuclear material;
 - (ii) Early detection of removal of special nuclear material by an external adversary from a controlled access area;
 - (iii) Assure proper placement and transfer of custody of special nuclear material; and

- (iv) Response to indications of an unauthorized removal of SNM and then notify the appropriate response forces of its removal in order to facilitate its recovery.

Therefore the fixed-site, physical protection requirements of 10 CFR 73.67(a)(1) are applicable because of the manner in which the Turkey Point Units 6 and 7 COL application described SNM of low strategic significance.

Applicable Requirement: 10 CFR 73.67(a)(1), “General performance objectives. (1) Each licensee who possesses, uses or transports special nuclear material of moderate or low strategic significance shall establish and maintain a physical protection system that will achieve the following objectives:...”

The applicant states in the “Implementation Milestone” column of FSAR Table 13.4-201, under Item 15, its commitment to meet the requirements of 10 CFR 73.67, “180 days prior to initial receipt of new fuel or non-fuel special nuclear material.” Establishment of the physical protection system is outlined in the SNMPPPD, Revision 7, dated September 24, 2015 (Safeguards LAN Electronic Safe (SLES) Accession No. NS113247). Specifically, Section 4.4.1, “Establishment of the Physical Protection System,” describes six establishment elements that pertain to the following: lighting, detection, alarm station status, communications, access control, and physical barriers of the controlled access area. In addition, Section 4.4.2, “Maintenance of the Physical Protection System,” of the SNMPPPD contains an explanation of the maintenance that will be applied to the physical protection system.

The FPL application states that FPL will fully implement the requirements of 10 CFR 73.67 before SNM is on site. Also, the application outlined establishment and maintenance elements for the physical protection system. The establishment of physical protection elements is sufficient because before the physical protection infrastructure will be considered operational: 1) the lighting necessary for human detection through visual observation will be tested and confirmed as adequate, 2) visual assessment systems will be tested as functioning as necessary to support security operations, 3) alarm stations will be validated as having the ability to adequately support physical security activities for the protection of the SNM of low strategic significance, 4) communication technologies that are to be relied upon to enable the physical security strategy to operate effectively will be tested and confirmed to allow for intelligible voice interfaces, 5) the means of access control will be tested for its performance to support the physical security strategy, and 6) the physical barriers that provide containment of the SNM of low strategic significance will be inspected to ensure a comprehensive impediment to personnel entry is in place. The development of a maintenance program for the six physical protection elements established is committed to in the application. In addition, the application states that the maintenance program will have periodicity of maintenance configured for each of the six physical protection system elements that is commensurate with each of the elements' intended function. Therefore, the staff finds that the requirement of 10 CFR 73.67(a)(1), to have a physical protection system established and maintained, would be met.

Applicable Requirement: 10 CFR 73.67(a)(1)(i), “General performance objectives. Each licensee who possesses, uses or transports special nuclear material of moderate or low strategic significance shall establish and maintain a physical protection system that will achieve the following objectives: (i) Minimize the possibilities for unauthorized removal of special nuclear material consistent with the potential consequences of such actions:...”

The applicant states in the “Implementation Milestones” column of FSAR Table 13.4-201, under Item 15, its commitment to meet the requirements of 10 CFR 73.67, “180 days prior to initial receipt of new fuel or non-fuel special nuclear material.” In addition, the SNMPPPD describes in Section 5.3.1, “Monitoring SNM (Non-Fuel SNM - HEU Neutron Sources),” how this general performance objective will be met for the highly enriched uranium (HEU) sources by detailing adversary scenarios and explaining how the physical protection system will work to meet the requirement. In addition, the SNMPPP describes in Section 5.3.2, “Monitoring SNM (New Fuel Assemblies),” adversary scenarios applied to SNM reactor fuel and explains how the physical protection system will work to meet this requirement as well.

The FPL application states that FPL will fully implement the requirements of 10 CFR 73.67 before SNM is delivered. In addition, its SNMPPPD describes how the possibilities for unauthorized removal are minimized consistent with the consequences of such actions. The application describes potential adversarial scenarios for all activities involving SNM of low strategic significance and highlights how the six physical protection system elements work in a coordinated fashion to adequately minimize the risk of theft of the materials. Therefore, the staff finds that the requirement of 10 CFR 73.67(a)(1)(i), to have a physical protection system established and maintained that has the objective to minimize the possibilities for unauthorized removal of SNM consistent with the potential consequences of such actions, would be met.

Applicable Requirement: 10 CFR 73.67(a)(1)(ii), “General performance objectives. Each licensee who possesses uses or transports special nuclear material of moderate or low strategic significance shall establish and maintain a physical protection system that will achieve the following objectives: “...(ii) Facilitate the location and recovery of missing special nuclear material.”

The applicant states in the “Implementation Milestone” column of FSAR Table 13.4-201, under Item 15, its commitment to meet the requirements of 10 CFR 73.67, “180 days prior to initial receipt of new fuel or non-fuel special nuclear material.” In addition, its SNMPPPD describes in Section 5.10, “Contingency Response,” the detection, assessment, and response strategies of the physical protection system that would facilitate the location and recovery of missing SNM.

The FPL application states that FPL will fully implement the requirements of 10 CFR 73.67 before SNM is delivered. In addition, its SNMPPP describes the detection, assessment and response attributes of the physical protection system that would facilitate the location and recovery of missing SNM. The application explicitly describes how the planned-for detection and assessment physical protection system elements function to provide adequate detection and assessment of malevolent activities in order to initiate a specific response that would enable the location and recovery of SNM of low strategic significance. Scenarios that depict adversary actions, operation of physical security system elements, and security force response activities provide assurance that the requirement of 10 CFR 73.67(a)(1)(ii) would be met. Therefore, the staff finds that the requirement of 10 CFR 73.67(a)(1)(ii), to have a physical protection system established and maintained that has the objective to facilitate the location and recovery of missing SNM, would be met.

Applicable Requirement: 10 CFR 73.67(a), “General performance objectives. (2) To achieve these objectives, the physical protection system shall provide: (i) Early detection and assessment of unauthorized access or activities by an external adversary within the controlled access area containing special nuclear material...”

The applicant states in the "Implementation Milestone" column of FSAR Table 13.4-201 under Item 15 its commitment to meet the requirements of 10 CFR 73.67, "180 days prior to initial receipt of new fuel or non-fuel special nuclear material." In addition, its SNMPPPD Sections 5.3.1 and 5.3.2 "Monitoring SNM (New Fuel Assemblies)" describes how the physical protection system provides for early detection and assessment of unauthorized access or activities by an external adversary within the controlled access area containing SNM.

The FPL application states that FPL will fully implement the requirements of 10 CFR 73.67 before SNM is delivered. In addition, its SNMPPP describes the physical protection strategies for early detection and assessment to address unauthorized access or activities by an external adversary within the controlled access area containing SNM. These physical protection strategies are consistent with staff guidance in RG 5.59. Therefore, the staff finds that the requirement of 10 CFR 73.67(a)(2)(i), to have a physical protection system that provides early detection and assessment of unauthorized access or activities by an external adversary within the controlled access area containing SNM, would be met.

Applicable Requirement: 10 CFR 73.67(a)(2)(ii), "General performance objectives. To achieve these objectives, the physical protection system shall provide: ... (ii) Early detection of removal of special nuclear material by an external adversary from a controlled access area..."

The applicant states in the "Implementation Milestone" column of FSAR Table 13.4-201, under Item 15, its commitment to meet the requirements of 10 CFR 73.67, "180 days prior to initial receipt of new fuel or non-fuel special nuclear material." In addition, its SNMPPPD Sections 5.3.1 and 5.3.2 describe how the physical protection system provides for early detection of removal of SNM by an external adversary from a controlled access area.

The FPL application states that FPL will fully implement 10 CFR 73.67 before SNM is delivered. In addition, its SNMPPPD describes the physical protection strategies for early detection and assessment to address the potential for the removal of SNM by an external adversary from a controlled access area. These physical protection strategies are consistent with staff guidance in RG 5.59. Therefore, the staff finds that the requirement of 10 CFR 73.67(a)(2)(ii), to have a physical protection system that provides early detection of removal of SNM by an external adversary from a controlled access area, would be met.

Applicable Requirement: 10 CFR 73.67(a)(2)(iii), "General performance objectives. To achieve these objectives, the physical protection system shall: ... (iii) Assure proper placement and transfer of custody of special nuclear material; and..."

The applicant states in the "Implementation Milestone" column of FSAR Table 13.4-201, under Item 22, its commitment to meet the applicable requirements of 10 CFR 74, "Material Control and Accounting of Special Nuclear Material," "[p]rior to receipt of special nuclear material" as a license condition. Also, the applicant states in Enclosure 5, "Special Nuclear Material (SNM) Material Control and Accounting Program Description," of Part 11 of the COL application that the applicant will establish a criteria for the "SNM control and accounting system, including criteria for the receipt, internal control, physical inventory, and shipment of SNM."

In addition, the applicant describes in its SNMPPPD, in Section 5.1.1, "Receipt of non-fuel SNM"; Section 5.1.2, "Receipt of SNM - Fuel Assemblies/Fuel Components"; and Section 5.8, "Internal Transfers," the MC&A measures specific to the non-fuel and fuel SNM, respectively.

The FPL application states that FPL will fully implement the appropriate provisions of 10 CFR 74 before SNM is received. The application also states that: 1) notification will be made to the shipper upon receipt of the SNM of low strategic significance; 2) an investigation will be initiated as required per 10 CFR 73.67 and 10 CFR 74.11 if the shipment is not received as scheduled; 3) the NRC Operations Center will be notified within an hour after assessing that a shipment has not arrived and/or within an hour of SNM of low strategic significance recovery; 4) the licensee will conduct an inspection of tamper seal devices on containers of SNM of low strategic significance after accessing the shipment conveyance that has been received at the nuclear reactor facility; and 5) the licensee will verify that the shipment is consistent with the shipment's manifest in regard to identification markings and numbers of SNM containers.

In addition, the applicant has described in its SNMPPPD, how specific MC&A measures apply to meet this general performance objective. Therefore, the staff finds that the requirement of 10 CFR 73.67(a)(2)(iii), to assure proper placement and transfer of custody of SNM, would be met.

Applicable Requirement: 10 CFR 73.67(a)(2)(iv), "General performance objectives. To achieve these objectives, the physical protection system shall: ... (iv) Respond to indications of an unauthorized removal of special nuclear material and then notify the appropriate response forces of its removal in order to facilitate its recovery."

The applicant states in the "Implementation Milestone" column of FSAR Table 13.4-201, under Item 15, its commitment to meet the requirements of 10 CFR 73.67, "180 days prior to initial receipt of new fuel or non-fuel special nuclear material." In addition, its SNMPPPD Section 5.10 describes the detection, assessment, and response strategies that would provide indications of missing or stolen SNM and subsequently facilitate the recovery thereof. The appropriate response from offsite (i.e., the particular coordinated response with a local law enforcement agency, etc.), was identified in the SNMPPPD by discussion of the offsite response plans and by referencing Section 8, "Local Law Enforcement Liaison," of the Turkey Point Units 6 and 7 PSP, and Section 5.6, "Local Law Enforcement (LLEA)"; Section 5.7, "State Response Organizations"; and Section 5.8, "Federal Response Organizations," of the Turkey Point Units 6 and 7 Contingency Plan, Revision 2, dated November 9, 2010 (SLES Accession No. NS10824).

The FPL application states that FPL will fully implement 10 CFR 73.67 before SNM is delivered. In addition, its SNMPPPD describes the early detection, assessment and response physical protection strategies that would facilitate recovery of missing or stolen SNM. Specifically, the applicant described in the SNMPPP detection, assessment, communication, and response scenarios associated with all locations of SNM of low strategic significance. In addition, the response protocols described are consistent with both RG 5.59 and the response measure criteria in the post-September 11, 2001, SNM of low strategic significance security order. Therefore, the staff finds that the requirement of 10 CFR 73.67(a)(2)(iv), to have a physical protection system that shall respond to indications of an unauthorized removal of SNM and then notify the appropriate response forces of its removal in order to facilitate its recovery, would be met.

1.5.5.1.4.2. *Fixed Site General Requirements*

The applicable requirements specified in 10 CFR 73.67, "Licensee fixed site and in-transit requirements for the physical protection of special nuclear material of moderate and low strategic significance," include the following general requirements for fixed sites.

"(c) Each licensee who possesses, uses, transports, or delivers to a carrier for transport special nuclear material of moderate strategic significance, or 10 kg or more of special nuclear material of low strategic significance shall:

- (1) Submit a security plan or an amended security plan describing how the licensee will comply with all the requirements of paragraphs (d), (e), (f), and (g) of this section, as appropriate, including schedules of implementation. The licensee shall retain a copy of the effective security plan as a record for three years after the close of period for which the licensee possesses the special nuclear material under each license for which the original plan was submitted. Copies of superseded material must be retained for three years after each change.
- (2) Within 30 days after the plan submitted pursuant to paragraph (c)(1) of this section is approved, or when specified by the NRC in writing, implement the approved security plan."

Applicable Requirement: 10 CFR 73.67(c)(1), "Submit a security plan...including schedules for implementation...shall retain a copy ...for three years..." ... "Copies of the superseded material must be retained for three years after each change."

The applicant submitted a security plan termed the SNMPPPD that covered the applicable fixed site provisions of 10 CFR 73.67 (i.e., 10 CFR 73.67(f)). The applicant stated in Section 5.7, "Audits and Records," of its SNMPPPD that the security plan (i.e., the SNMPPPD) would be retained for three years and that copies of superseded material will be retained for three years after each change.

The SNMPPPD describes the required retention parameters for the SNMPPPD and changes to it. Therefore, the staff finds that the requirement of 10 CFR 73.67(c)(1); to submit a security plan, retain the security plan for three years after the specific type of SNM has been removed from the site, and to retain superseded security plan change(s) for three years after each change, would be met.

Applicable Requirement: 10 CFR 73.67(c)(2), "Within 30 days after the plan submitted pursuant to paragraph (c)(1) of this section is approved, or when specified by the NRC in writing, implement the approved security plan."

The applicant states in the "Implementation Milestone" column of FSAR Table 13.4-201, under Item 15, its commitment to meet the requirements of 10 CFR 73.67, "180 days prior to initial receipt of new fuel or non-fuel special nuclear material."

Additionally, the staff proposes to impose the following license condition, based on License Condition 6, as listed in Part 10 of the COL application:

No later than 12 months after issuance of the COL, the licensee shall submit to the Director of NRO, or the Director's designee, a schedule for implementation of

the operational programs listed in FSAR Table 13.4-201, including the associated estimated date for initial loading of fuel. The schedule shall be updated every 6 months until 12 months before scheduled fuel loading, and every month thereafter until all the operational programs listed in FSAR Table 13.4-201 have been fully implemented.

In the application, the applicant has stated that it will implement the requirements of 10 CFR 73.67 before SNM is received. Also, a license condition has been applied to ensure the NRC staff is aware of the scheduled date for implementation of the requirements of 10 CFR 73.67. Therefore, the requirement to either implement the SNMPPPD within 30 days after NRC approval, or as designated by the NRC in writing, will be met by the required schedule for implementation of operational programs.

1.5.5.1.4.3. Fixed Site Physical Protection Requirements

The applicable requirements specified in 10 CFR 73.67 include fixed-site, physical protection requirements for SNM of low strategic significance.

The regulations of 10 CFR 73.67(f) state that, "Each licensee who possesses, stores, or uses special nuclear material of low strategic significance at a fixed site or contiguous sites, except those who are licensed to operate a nuclear power reactor pursuant to Part 50, shall:

- (1) Store or use the material only within a controlled access area,
- (2) Monitor with an intrusion alarm or other device or procedures the controlled access areas to detect unauthorized penetrations or activities,
- (3) Assure that a watchman or offsite response force will respond to all unauthorized penetrations or activities, and
- (4) Establish and maintain response procedures for dealing with threats of thefts or thefts of this material. The licensee shall retain a copy of the current response procedures as a record for three years after the close of period for which the licensee possesses the special nuclear material under each license for which the procedures were established. Copies of superseded material must be retained for three years after each change."

The fixed-site physical protection requirements of 10 CFR 73.67(f) are applicable because of the manner in which SNM of low strategic significance was described in the Turkey Point Units 6 and 7 COL application.

Applicable Requirement: 10 CFR 73.67(f)(1), "Fixed site requirements for special nuclear material of low strategic significance. Each licensee who possesses, stores, or uses special nuclear material of low strategic significance at a fixed site or contiguous sites, except those who are licensed to operate a nuclear power reactor pursuant to Part 50, shall: (1) Store or use the material only within a controlled access area..."

The applicant states in the "Implementation Milestone" column of FSAR Table 13.4-201, under Item 15, its commitment to meet the requirements of 10 CFR 73.67, "180 days prior to initial receipt of new fuel or non-fuel special nuclear material." In addition, the SNMPPPD Section 5.2

“Storage”; Section 5.8 “Internal Transfers”; and Figures 1 through 13, describe the physical characteristics of the controlled access area. The description of the controlled access area depicted in the SNMPPPD includes temporary and permanent controlled access areas to enable protection during receipt and long-term storage of SNM, respectively. In addition, the described physical characteristics of the controlled access area are consistent with the recommended penetration resistance features explained in RG 5.59. Furthermore, as described in the application, both the fuel SNM and non-fuel SNM of low strategic significance will always be protected within a controlled access area. The non-fuel SNM is described as only being removed from its controlled access area and into its functioning location after the protected area of the nuclear reactor has been established per 10 CFR 73.55(e)(8), which is an acceptable practice because when the SNM is located inside a protected area, it is provided adequate protection.

The FPL application states that FPL will fully implement 10 CFR 73.67 before SNM is delivered. In addition, the SNMPPPD describes the characteristics of their planned-for controlled access area; therefore, the staff finds that the requirement of 10 CFR 73.67(f)(1), to store or use the material only within a controlled access area, would be met.

Applicable Requirement: 10 CFR 73.67(f)(2) “Fixed site requirements for special nuclear material of low strategic significance. Each licensee who possesses, stores, or uses special nuclear material of low strategic significance at a fixed site or contiguous sites, except those who are licensed to operate a nuclear power reactor pursuant to Part 50, shall: (2) Monitor with an intrusion alarm or other device or procedures the controlled access areas to detect unauthorized penetrations or activities.”

The applicant states in the “Implementation Milestone” column of FSAR Table 13.4-201, under Item 15, its commitment to meet the requirements of 10 CFR 73.67, “180 days prior to initial receipt of new fuel or non-fuel special nuclear material.” In addition, their SNMPPPD Section 5.3, “Monitoring”; Section 5.3.1, “Monitoring SNM (Non fuel SNM...)”; and Section 5.3.2, “Monitoring SNM (New Fuel Assemblies),” describe the detection processes that would result in recognition of unauthorized penetrations or activities in the locations of SNM of low strategic significance and the controlled access area.

The FPL application states that FPL will fully implement the requirements of 10 CFR 73.67 before SNM is received. In addition, its SNMPPPD describes the detection processes that would result in recognition of unauthorized penetrations or activities in the locations of SNM and the controlled access area. Specifically, the applicant describes in its SNMPPPD the detection techniques and assessment methods that would result in a high probability of detection and accurate assessment of malevolent acts or potentially malevolent indications. In addition, in the SNMPPPD, the applicant described administrative controls that would reduce the risk of not detecting a malevolent act or indications of potential malevolent acts to an acceptable level. Therefore, the staff finds that the requirement of 10 CFR 73.67(f)(2), to monitor with an intrusion alarm or other device or procedures the controlled access areas to detect unauthorized penetrations or activities, would be met.

Applicable Requirement: 10 CFR 73.67(f)(3), “Fixed site requirements for special nuclear material of low strategic significance. Each licensee who possesses, stores, or uses special nuclear material of low strategic significance at a fixed site or contiguous sites, except those who are licensed to operate a nuclear power reactor pursuant to

part 50, shall: (3) Assure that a watchman or offsite response force will respond to all unauthorized penetrations or activities...”

The applicant states in the “Implementation Milestone” column of FSAR Table 13.4-201, under Item 15, its commitment to meet the requirements of 10 CFR 73.67, “180 days prior to initial receipt of new fuel or non-fuel special nuclear material.” In addition, their SNMPPPD Section 5.3.1, Section 5.3.2, and Section 5.10 describe the detection, assessment, and response measures for the physical protection of the material. Furthermore, the appropriate response from offsite (e.g., the specifically coordinated response with a local law enforcement agencies), was pointed out in the SNMPPPD by discussion of the offsite response plans and by referencing Section 8 of the Turkey Point Units 6 and 7 reactor PSP and Sections 5.6, 5.7, and 5.8 of the Turkey Point Units 6 and 7 reactor Contingency Plan, Revision 2, dated November 9, 2010 (SLES Accession No. NS108024).

The FPL application states that FPL will fully implement the requirements of 10 CFR 73.67 before SNM is received. In addition, their SNMPPPD and other information referenced in the SNMPPPD describe the detection, assessment, and response measures for the physical protection of the material. The applicant provided details in the SNMPPPD of the protocols of detection, assessment, communications, and response that would work to adequately protect the SNM. In addition, those protocols both onsite and offsite response actions were committed to be developed and implemented via written procedures. Therefore, the staff finds that the requirement of 10 CFR 73.67(f)(3), to assure that a watchman or offsite response force will respond to all unauthorized penetrations or activities, would be met.

Applicable Requirement: 10 CFR 73.67(f)(4), “Fixed site requirements for special nuclear material of low strategic significance. Each licensee who possesses, stores, or uses special nuclear material of low strategic significance at a fixed site or contiguous sites, except those who are licensed to operate a nuclear power reactor pursuant to Part 50, shall: (4) Establish and maintain response procedures for dealing with threats of thefts or thefts of this material. The licensee shall retain a copy of the current response procedures as a record for three years after the close of period for which the licensee possesses the special nuclear material under each license for which the procedures were established. Copies of superseded material must be retained for three years after each change.”

The applicant states in the “Implementation Milestone” column of FSAR Table 13.4-201, under Item 15, its commitment to meet the requirements of 10 CFR 73.67, “180 days prior to initial receipt of new fuel or non-fuel special nuclear material.” In addition, their SNMPPPD Section 4.1, “Procedures”; Section 5.3.1; Section 5.3.2; Section 5.7, and Section 5.10 describe the framework of and details to the development of response procedures. In addition, Section 5.7 of the SNMPPPD notes the retention provision of three years for response procedures and the changes thereof.

The FPL application states that FPL will fully implement the requirements of 10 CFR 73.67 before SNM is delivered. In addition, their SNMPPPD describes: 1) the framework of the response procedures, 2) the details on the development of response procedures, 3) and the retention actions of three years for the response procedures. Therefore, the staff finds that the requirement of 10 CFR 73.67(f)(4), to establish and maintain response procedures, would be met.

1.5.5.1.4.4. *In-Transit General Performance Objectives*

The applicable requirements specified in 10 CFR 73.67 include general performance objectives.

The physical protection requirements of 10 CFR 73.67(a), states, "General performance objectives.

- (1) Each licensee who possesses, uses, or transports special nuclear material of moderate or low strategic significance shall establish and maintain a physical protection system that will achieve the following objectives:
 - (i) Minimize the possibilities for unauthorized removal of special nuclear material consistent with the potential consequences of such actions; and
 - (ii) Facilitate the location and recovery of missing special nuclear material.
- (2) To achieve these objectives, the physical protection system shall provide:
 - (i) Early detection and assessment of unauthorized access or activities by an external adversary within the controlled access area containing special nuclear material;
 - (ii) Early detection of removal of special nuclear material by an external adversary from a controlled access area;
 - (iii) Assure proper placement and transfer of custody of special nuclear material; and
 - (iv) Respond to indications of an unauthorized removal of special nuclear material and then notify the appropriate response forces of its removal in order to facilitate its recovery."

The in-transit physical protection requirements of 10 CFR 73.67(a) are applicable because of the manner in which SNM of low strategic significance was described in the Turkey Point Units 6 and 7 COL application.

Applicable Requirement: 10 CFR 73.67(a), "General performance objectives. (1) Each licensee who possesses, uses or transports special nuclear material of moderate or low strategic significance shall establish and maintain a physical protection system that will achieve the following objectives:..."

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g) in Section 6, "Shipment," of its SNMPPPD. The applicant also submitted a New Fuel Shipping Plan, in Enclosure 6 of Part 11 of the COL application. The SNMPPPD states that an SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has "plans and procedures" that are developed and implemented in such a manner that each general performance objective of 10 CFR 73.67 will be met. Because FPL will be using a SNM-qualified licensee to perform the shipment of SNM of low strategic significance and will

confirm that such a licensee has the physical protection measures in place to meet each general performance objective, subsequently that SNM-qualified licensee will have the ability to meet the requirement to establish and maintain a physical protection system.

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has physical protection measures in place to meet each general performance objective. Therefore, the staff finds that the requirement of 10 CFR 73.67(a)(1), to establish and maintain a physical protection system, would be met.

Applicable Requirement: 10 CFR 73.67(a)(1)(i), “General performance objectives. Each licensee who possesses, uses or transports special nuclear material of moderate or low strategic significance shall establish and maintain a physical protection system that will achieve the following objectives: (i) Minimize the possibilities for unauthorized removal of special nuclear material consistent with the potential consequences of such actions;...”

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g) in Section 6 of its SNMPPPD. The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that each general performance objective of 10 CFR 73.67 will be met. Because FPL will be using a SNM-qualified licensee to perform the shipment of SNM of low strategic significance and will confirm that such a licensee has the physical protection measures in place to meet each general performance objective, subsequently that SNM-qualified licensee will have the ability to meet the requirement to establish and maintain a physical protection system that has the capability to minimize the possibilities for unauthorized removal of SNM consistent with the potential consequences of such actions.

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has physical protection measures in place to meet each general performance objective. Therefore, the staff finds that the requirement of 10 CFR 73.67(a)(1)(i), to minimize the possibilities for unauthorized removal of SNM consistent with the potential consequences of such actions, would be met.

Applicable Requirement: 10 CFR 73.67(a)(1)(ii), “General performance objectives. Each licensee who possesses uses or transports special nuclear material of moderate or low strategic significance shall establish and maintain a physical protection system that will achieve the following objectives: “...(ii) Facilitate the location and recovery of missing special nuclear material.”

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g) in Section 6 of their SNMPPPD. The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that each general performance objective

of 10 CFR 73.67 will be met. Because FPL will be using a SNM-qualified licensee to perform the shipment of SNM of low strategic significance and will confirm that such a licensee has the physical protection measures in place to meet each general performance objective, subsequently that SNM-qualified licensee will have the ability to meet the requirement of establishing and maintaining a physical protection system that has the capability to facilitate the location and recovery of missing SNM.

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has physical protection measures in place to meet each general performance objective. Therefore, the staff finds that the requirement of 10 CFR 73.67(a)(1)(ii), to facilitate the location and recovery of missing SNM, would be met.

Applicable Requirement: 10 CFR 73.67(a), “General performance objectives. (2) To achieve these objectives, the physical protection system shall provide: (i) Early detection and assessment of unauthorized access or activities by an external adversary within the controlled access area containing special nuclear material...”

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g) in Section 6 of their SNMPPPD. The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that each general performance objective of 10 CFR 73.67 will be met. Because FPL will be using a SNM-qualified licensee to perform the shipment of SNM of low strategic significance and will confirm that such a licensee has the physical protection measures in place to meet each general performance objective, subsequently that SNM-qualified licensee will have the ability to meet the requirement of establishing and maintaining a physical protection system that has the capability to provide for early detection and assessment of unauthorized access or activities by an external adversary within the controlled access area containing SNM.

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has physical protection measures in place to meet each general performance objective. Therefore, the staff finds that the requirement of 10 CFR 73.67(2)(i), to provide early detection and assessment of unauthorized access or activities by an external adversary within the controlled access area containing SNM, would be met.

Applicable Requirement: 10 CFR 73.67(a)(2)(ii), “General performance objectives. To achieve these objectives, the physical protection system shall provide: ... (ii) Early detection of removal of special nuclear material by an external adversary from a controlled access area...”

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g) in Section 6 of their SNMPPPD. The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that each general performance objective

of 10 CFR 73.67 will be met. Because FPL will be using a SNM-qualified licensee to perform the shipment of SNM of low strategic significance and will confirm that such a licensee has the physical protection measures in place to meet each general performance objective, subsequently that SNM-qualified licensee will have the ability to meet the requirement of establishing and maintaining a physical protection system that has the capability to provide for early detection of removal of SNM by an external adversary from a controlled access area.

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has physical protection measures in place to meet each general performance objective. Therefore, the staff finds that the requirement of 10 CFR 73.67(2)(ii) to provide early detection of removal of SNM by an external adversary from a controlled access area, would be met.

Applicable Requirement: 10 CFR 73.67(a)(2)(iii), “General performance objectives. To achieve these objectives, the physical protection system shall: ... (iii) Assure proper placement and transfer of custody of special nuclear material.”

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g) in Section 6 of their SNMPPPD. The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that each general performance objective of 10 CFR 73.67 will be met. Also, FPL has described the process for receiving and placing SNM in Sections 5.1.1 (for non-fuel SNM) and 5.1.2 (for fuel SNM) of its SNMPPP. Furthermore, SNM to be transported from the site or received at the site will have an MC&A program applied to it as described in Part 11 of the application. Because FPL will be using a SNM-qualified licensee to perform the shipment of SNM of low strategic significance and will confirm that such a licensee has the physical protection measures in place to meet each general performance objective, has procedures for receipt/placement of SNM, and has an MC&A program that will apply to SNM, subsequently, that SNM-qualified licensed shipper and FPL will have the ability to meet the requirement of establishing and maintaining a physical protection system that has the capability to assure proper placement and transfer of custody of SNM.

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has physical protection measures in place to meet each general performance objective. In addition, FPL has a described process for receiving and placing SNM and will have a material control and accounting program applied to SNM to be shipped or received. Therefore, the staff finds that the requirement of 10 CFR 73.67(2)(iii), to assure proper placement and transfer of custody of SNM, would be met.

Applicable Requirement: 10 CFR 73.67(a)(2)(iv), “General performance objectives. To achieve these objectives, the physical protection system shall: ... (iv) Respond to indications of an unauthorized removal of special nuclear material and then notify the appropriate response forces of its removal in order to facilitate its recovery.”

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g) in Section 6 of their SNMPPPD. The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that each general performance objective of 10 CFR 73.67 will be met. Because FPL will be using a SNM-qualified licensee to perform the shipment of SNM of low strategic significance and will confirm that such a licensee has the physical protection measures in place to meet each general performance objective, subsequently that SNM-qualified licensee will have the ability to meet the requirement of responding to indications of an unauthorized removal of SNM and then notify the appropriate response forces of its removal in order to facilitate its recovery.

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has physical protection measures in place to meet each general performance objective. Therefore, the staff finds that the requirement of 10 CFR 73.67(a)(2)(iv), to respond to indications of an unauthorized removal of SNM and then notify the appropriate response forces of its removal in order to facilitate its recovery, would be met.

1.5.5.1.4.5. *In-Transit General Requirements*

The applicable requirements specified in 10 CFR 73.67 include the following general requirements.

- “(c) Each licensee who possesses, uses, transports, or delivers to a carrier for transport special nuclear material of moderate strategic significance, or 10 kg or more of special nuclear material of low strategic significance shall:
- (1) Submit a security plan or an amended security plan describing how the licensee will comply with all the requirements of paragraphs (d), (e), (f), and (g) of this section, as appropriate, including schedules of implementation. The licensee shall retain a copy of the effective security plan as a record for three years after the close of period for which the licensee possesses the special nuclear material under each license for which the original plan was submitted. Copies of superseded material must be retained for three years after each change,
 - (2) Within 30 days after the plan submitted pursuant to paragraph (c)(1) of this section is approved, or when specified by the NRC in writing, implement the approved security plan.”

The in-transit physical protection requirements of 10 CFR 73.67(a) are applicable because of the manner in which SNM of low strategic significance was described in the Turkey Point Units 6 and 7 COL application.

Applicable Requirement: 10 CFR 73.67(c)(1), “Submit a security plan including schedules for implementation...shall retain a copy ...for three years...” ... “Copies of the superseded material must be retained for three years after each change.”

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67 (g) in Section 6 of its SNMPPPD. The SNMPPPD Section 5.7 states that the SNMPPPD would be retained for three years and that copies of superseded material will be retained for three years after each change.

The FPL application states that FPL will fully implement the requirements of 10 CFR 73.67 before SNM is received. In addition, its SNMPPPD describes the required retention parameters for the SNMPPPD and changes to it. Therefore, the staff finds that the requirement of 10 CFR 73.67(c)(1), to submit a security plan including schedules for implementation, to retain the security plan for three years after the specific type of SNM has been removed from the site, and to retain copies of the superseded material for three years after each change, would be met.

Applicable Requirement: 10 CFR 73.67(c)(2), “Within 30 days after the plan submitted pursuant to paragraph (c)(1) of this section is approved, or when specified by the NRC in writing, implement the approved security plan.”

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g) in Section 6 of its SNMPPPD. The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that 10 CFR 73.67(c)(2) would be met. Furthermore, the applicant stated in “Table 13.4-201,” (ADAMS Accession No. ML15301A332), under Item 15, titled “Implementation Milestone,” their commitment to meet the requirements of 10 CFR 73.67, “180 days prior to initial receipt of new fuel or non-fuel special nuclear material.” Therefore, the staff finds the requirement, of 10 CFR 73.67(c)(2) to “Within 30 days after the plan submitted pursuant to paragraph (c)(1) of this section is approved, or when specified by the NRC in writing, implement the approved security plan,” would be met.

1.5.5.1.4.6. *In-Transit Physical Protection Requirements*

The applicable requirements specified in 10 CFR 73.67, “Licensee fixed site and in-transit requirements for the physical protection of special nuclear material of moderate and low strategic significance,” had in-transit physical protection requirements described.

The physical protection requirements of 10 CFR 73.67(g), “In-transit requirements for special nuclear material of low strategic significance,” states that,

- (1) Each licensee who transports or who delivers to a carrier for transport special nuclear material of low strategic significance shall:
 - (i) Provide advance notification to the receiver of any planned shipments specifying the mode of transport, estimated time of arrival, location of the nuclear material transfer point, name of carrier and transport identification,
 - (ii) Receive confirmation from the receiver prior to commencement of the planned shipment that the receiver will be ready to accept the shipment at the planned time and location and acknowledges the specified mode of transport,

- (iii) Transport the material in a tamper indicating sealed container,
 - (iv) Check the integrity of the containers and seals prior to shipment, and
 - (v) Arrange for the in-transit physical protection of the material in accordance with the requirements of Section 73.67(g)(3) of this part, unless the receiver is a licensee and has agreed in writing to arrange for the in-transit physical protection.
- (2) Each licensee who receives quantities and types of special nuclear material of low strategic significance shall:
- (i) Check the integrity of the containers and seals upon receipt of the shipment,
 - (ii) Notify the shipper of receipt of the material as required in Section 74.15 of this chapter, and
 - (iii) Arrange for the in-transit physical protection of the material in accordance with the requirements of Section 73.67(g)(3) of this part, unless the shipper is a licensee and has agreed in writing to arrange for the in-transit physical protection.
- (3) Each licensee, either shipper or receiver, who arranges for the physical protection of special nuclear material of low strategic significance while in transit or who takes delivery of such material free on board (f.o.b.) the point at which it is delivered to a carrier for transport shall:
- (i) Establish and maintain response procedures for dealing with threats or thefts of this material. The licensee shall retain a copy of the current response procedures as a record for three years after the close of period for which the licensee possesses the special nuclear material under each license for which the procedures were established. Copies of superseded material must be retained for three years after each change.
 - (ii) Make arrangements to be notified immediately of the arrival of the shipment at its destination, or of any such shipment that is lost or unaccounted for after the estimated time of arrival at its destination, and
 - (iii) Conduct immediately a trace investigation of any shipment that is lost or unaccounted for after the estimated arrival time and notify the NRC Operations Center within one hour after the discovery of the loss of the shipment and within one hour after recovery of or accounting for such lost shipment in accordance with the provisions of Section 73.71 of this part.”

The in-transit physical protection requirements of 10 CFR 73.67(g) are applicable because of the manner in which SNM of low strategic significance was described in the Turkey Point Units 6 and 7 COL application.

Applicable Requirement: 10 CFR 73.67(g) “In-transit requirements for special nuclear material of low strategic significance. (1) Each licensee who transports or who delivers to a carrier for transport special nuclear material of low strategic significance shall: (i)

Provide advance notification to the receiver of any planned shipments specifying the mode of transport, estimated time of arrival, location of the nuclear material transfer point, name of carrier and transport identification.”

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g) in Section 6 of its SNMPPPD. The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that 10 CFR 73.67(g)(1)(i) will be met. FPL will be using a SNM-qualified licensee to perform the shipment of SNM of low strategic significance and will confirm that such a licensee has the physical protection measures in place to meet 10 CFR 73.67(g)(1)(i). Therefore, that SNM-qualified licensee will have the ability to meet the requirement of providing advance notification to the receiver of any planned shipments specifying the mode of transport, estimated time of arrival, location of the nuclear material transfer point, name of carrier, and transport identification.

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has physical protection measures in place to meet 10 CFR 73.67(g)(1)(i). Therefore, the staff finds the requirement of 10 CFR 73.67(g)(1)(i) would be met.

Applicable Requirement: 10 CFR 73.67(g)(1)(ii) “In-transit requirements for special nuclear material of low strategic significance. (1) Each licensee who transports or who delivers to a carrier for transport special nuclear material of low strategic significance shall: ... (ii) Receive confirmation from the receiver prior to commencement of the planned shipment that the receiver will be ready to accept the shipment at the planned time and location and acknowledges the specified mode of transport.”

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g) in Section 6 of its SNMPPPD. The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used to transport SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that 10 CFR 73.67(g)(1)(ii) will be met. FPL will be using a SNM qualified licensee to perform the shipment of SNM of low strategic significance and will confirm that such a licensee has the physical protection measures in place to meet the requirements of 10 CFR 73.67(g)(1)(ii). Therefore, that SNM-qualified licensee will have the ability to meet the requirement of receiving confirmation from the receiver prior to commencement of the planned shipment that the receiver will be ready to accept the shipment at the planned time and location and acknowledges the specified mode of transport.

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has physical protection measures in place to meet 10 CFR 73.67(g)(1)(ii). Therefore, the staff finds that the requirement of 10 CFR 73.67(g)(1)(ii), to receive confirmation from the receiver prior to commencement of the planned shipment that the receiver will be ready to accept the shipment at the planned time and location and acknowledges the specified mode of transport, would be met.

Applicable Requirement: 10 CFR 73.67(g)(1)(iii) “In-transit requirements for special nuclear material of low strategic significance. (1) Each licensee who transports or who delivers to a carrier for transport special nuclear material of low strategic significance shall: ... (iii) Transport the material in a tamper indicating sealed container.”

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g) in Section 6 of its SNMPPPD. The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that 10 CFR 73.67(g)(1)(iii) will be met.

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has physical protection measures in place to meet 10 CFR 73.67(g)(1)(iii). Therefore, the staff finds that the requirement of 10 CFR 73.67(g)(1)(iii), to transport the material in a tamper indicating sealed container, would be met.

Applicable Requirement: 10 CFR 73.67(g)(2)(i) “In-transit requirements for special nuclear material of low strategic significance. (2) Each licensee who receives quantities and types of special nuclear material of low strategic significance shall: (i) Check the integrity of the containers and seals upon receipt of the shipment.”

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g) in Section 6 of its SNMPPPD. Specifically, Section 3.3 of the SNMPPPD states that for both non-fuel and fuel SNM, that the integrity of both shipping containers and tamper-seals will be checked.

The FPL application states that shipment containers and tamper-seals applied to those containers would be checked upon receipt. Therefore, the staff finds the requirement of 10 CFR 73.67(g)(2)(i), to check the integrity of the containers and seals upon receipt of the shipment, would be met.

Applicable Requirement: 10 CFR 73.67(g)(2)(ii) “In-transit requirements for special nuclear material of low strategic significance. (2) Each licensee who receives quantities and types of special nuclear material of low strategic significance shall: ... (ii) Notify the shipper of receipt of the material as required in Section 74.15 of this chapter.”

The FPL SNMPPPD, Sections 5.1.1 (for non-fuel SNM) and 5.1.2 (for fuel SNM), states that the shipper would be notified in accordance with 10 CFR 74.15. In addition, Section 4.1, “Procedures,” of the SNMPPPD states that the development of procedures for “Receiving and shipping SNM” was described.

The FPL application states that the shipper would be notified in accordance with 10 CFR 74.15 for both non-fuel and fuel SNM. Therefore, the staff finds that the requirement of 10 CFR 73.67(g)(2)(ii), to notify the shipper of receipt of SNM, as required per 10 CFR 74.15, would be met.

Applicable Requirement: 10 CFR 73.67(g)(2)(iii) “Arrange for the in- transit physical protection of the material in accordance with the requirements of Section 73.67(g)(3) of this part, unless the shipper is a licensee and has agreed in writing to arrange for the in-transit physical protection.”

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g) in Section 6 of its SNMPPPD. The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that 10 CFR 73.67(g)(2)(iii) will be met. Specifically, Section 6.1 “In-Transit Requirements for SNM” of the SNMPPPD states that FPL would ensure that the “SNM qualified licensee... would arrange for in-transit physical protection...in accordance with 10 CFR 73.67(g)(3), unless the shipper is a licensee and has agreed in writing to arrange for the in-transit physical protection.”

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has provisions in place to meet 10 CFR 73.67(g)(2)(iii). Therefore, the staff finds that the requirement of 10 CFR 73.67(g)(2)(iii), to arrange for the in-transit physical protection of the material in accordance with the requirements of Section 73.67(g)(3) of this part, unless the shipper is a licensee and has agreed in writing to arrange for the in-transit physical protection, would be met.

Applicable Requirement: 10 CFR 73.67(g)(3), “Each licensee, either shipper or receiver, who arranges for the physical protection of special nuclear material of low strategic significance while in transit or who takes delivery of such material f.o.b. the point at which it is delivered to a carrier for transport shall: (i) Establish and maintain response procedures for dealing with threats or thefts of this material. The licensee shall retain a copy of the current response procedures as a record for three years after the close of period for which the licensee possesses the special nuclear material under each license for which the procedures were established. Copies of superseded material must be retained for three years after each change.”

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g) in Section 6 of its SNMPPPD. The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that 10 CFR 73.67(g)(3)(i) will be met.

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has provisions in place to meet 10 CFR 73.67(g)(3)(i). Therefore, the staff finds the requirement, of 10 CFR 73.67(g)(3)(i) to, “[e]stablish and maintain response procedures ...,” would be met.

Applicable Requirement: 10 CFR 73.67(g)(3), “Each licensee, either shipper or receiver, who arranges for the physical protection of special nuclear material of low strategic significance while in transit or who takes delivery of such material f.o.b. the point at

which it is delivered to a carrier for transport shall: ... (ii) Make arrangements to be notified immediately of the arrival of the shipment at its destination point, or of any shipment that is lost or unaccounted for after the estimated time of arrival at its destination.”

The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g)(ii) in Section 6 of its SNMPPPD. The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that 10 CFR 73.67(g)(3)(ii) will be met. Also, Sections 5.1.1 and 5.1.2 of the SNMPPPD state that the shipping vendor will be notified in accordance with 10 CFR 73.67(g)(3)(ii), for non-fuel and fuel SNM, respectively.

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has provisions in place to meet 10 CFR 73.67(g)(3)(ii). In addition, the applicant states that notifications for both no-fuel and fuel SNM will be made in accordance with 10 CFR 73.67(g)(3)(ii). Therefore, the staff finds that the requirement of 10 CFR 73.67(g)(3)(ii), to make arrangements to be notified immediately of the arrival of the shipment at its destination point, or of any shipment that is lost or unaccounted for after the estimated time of arrival at its destination, would be met.

Applicable Requirement: 10 CFR 73.67(g)(3), “Each licensee, either shipper or receiver, who arranges for the physical protection of special nuclear material of low strategic significance while in transit or who takes delivery of such material f.o.b. the point at which it is delivered to a carrier for transport shall: ... (iii) Conduct immediately a trace investigation of any shipment that is lost or unaccounted for after the estimated arrival time and notify the NRC Operations Center within one hour after the discovery of the loss of the shipment and within one hour after recovery of or accounting for such lost shipment in accordance with the provisions of Section 73.71 of this part.”

In Section 6 of its SNMPPPD, the applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g). The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that 10 CFR 73.67(g)(3)(iii) will be met. FPL has committed to meeting the requirement in its SNMPPPD in Sections 5.1.1 and 5.1.2. Also, in Section 4.1 of the SNMPPPD, FPL noted that a procedure would be developed for notification processes.

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has provisions in place to meet 10 CFR 73.67(g)(3)(iii). In addition, FPL has committed to meeting the 10 CFR 73.67(g)(3)(iii) trace investigation/notification requirement. Therefore, the staff finds that the requirement of 10 CFR 73.67(g)(3)(iii), to conduct immediately a trace investigation of any shipment that is lost or unaccounted for after the estimated arrival time, to notify the NRC Operations Center within one hour after the discovery of the loss of the

shipment, to notify the NRC and within one hour after recovery of or accounting for such lost shipment in accordance with the provisions of 10 CFR Section 73.71, would be met.

Applicable Requirement: 10 CFR 73.67(g)(4), “Each licensee who exports special nuclear material of low strategic significance shall comply with the appropriate requirements specified in paragraphs (c) and (g) (1) and (3) of this section. The licensee shall retain each record required by these sections for three years after the close of period for which the licensee possesses the special nuclear material under each license that authorizes the licensee to export this material. Copies of superseded material must be retained for three years after each change.”

Section 1.5.5.1.4.2 of this SER describes how the applicant would meet the requirements of 10 CFR 73.67(c). The applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g) in Section 6 of its SNMPPPD. The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that 10 CFR 73.67(g)(4) will be met.

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has provisions in place to meet 10 CFR 73.67(c) requirements, as specified in Section 6 of the SNMPPPD. Section 1.5.5.1.4.6 of this SER details how the requirements of 10 CFR 73.67(g)(1) and (3) would be met. Therefore, the staff finds that the requirement of 10 CFR 73.67(g)(4), as described above, would be met.

Applicable Requirement: 10 CFR 73.67(g)(5)(i) “Each licensee who imports special nuclear material of low strategic significance shall: (i) Comply with the requirements specified in paragraphs (c) and (g) (2) and (3) of this section and retain each record required by these paragraphs for three years after the close of period for which the licensee possesses the special nuclear material under each license that authorizes the licensee to import this material. Copies of superseded material must be retained for three years after each change.”

In Section 6 of its SNMPPPD, the applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g). The SNMPPPD states that a SNM-qualified licensed shipper, other than FPL, will be used for transport of SNM of low strategic significance both to and from the site. In addition, Section 6 of the SNMPPPD states that FPL will confirm that the licensee used for transport of SNM has “plans and procedures” that are developed and implemented in such a manner that 10 CFR 73.67(g)(5) will be met.

The FPL application states that arrangements with a SNM-qualified licensed shipper would be made for the transport of SNM of low strategic significance, and that FPL will confirm that the licensed shipper has provisions in place to meet 10 CFR 73.67(c) requirements, as specified in SNMPPPD Section 6. How the requirements of 10 CFR 73.67(c) would be met by the applicant are described above in this SER. Therefore, the staff finds that the requirement of 10 CFR 73.67(g)(5), as described above, would be met.

Applicable Requirement: 10 CFR 73.67(g)(5)(ii) “Each licensee who imports special nuclear material of low strategic significance shall: (ii) Notify the person who delivered the material to a carrier for transport of the arrival of such material.”

In Section 6 of its SNMPPPD, the applicant included a description of how it intends to meet the in-transit physical protection requirements of 10 CFR 73.67(g). Specifically, Sections 5.1.1 and 5.1.2 state that the shipper would be notified upon receipt of SNM. In addition, the applicant described the development of procedures for receiving and shipping SNM in Section 4.1 of the SNMPPPD.

The staff finds that because FPL has described 1) notification actions to be made upon the receipt of SNM in their SNMPPPD, and 2) the development of procedures that would pertain to receiving and shipping SNM, the requirement of 10 CFR 73.67(g)(5)(ii), to notify the person who delivered the material to a carrier for transport of the arrival of such material, would be met.

1.5.5.1.4.7 Turkey Point Units 6 and 7 COL FSAR Section 13.5.2.2.8

The applicant included in Turkey Point Units 6 and 7 COL FSAR Section 13.5.2.2.8, “Security Procedures,” in general terms, the correct manner in which the requirements of 10 CFR 73.67 must be applied to the non-fuel HEU sources that are SNM of low strategic significance, that the applicant proposes to possess, transport, and use at the Turkey Point site. Therefore, the staff finds that the application of the correct physical protection measures, as stated in 10 CFR 73.67, to all types of SNM of low strategic significance, would be met.

1.5.5.1.4.8 Post September 11, 2001, Security Order for SNM of Low Strategic Significance

**Applicable Requirement: “General Performance Objectives and Requirements,”
Analysis per the order.**

The applicant considered the order and assessed that only Parts C and D of the order should be addressed. Section 1 of the SNMPPPD discusses the analysis that justified only Parts C and D of the order needed to be addressed. The analysis provided by the applicant describes the details of the assessment as to whether or not the nuclear reactor would have a critical target area, as defined in the security order text. Therefore, the staff finds that the analysis requirement presented in the beginning of the order would be met

Part C of the order “Response”

Applicable Requirement: Part C.1. of the order “Develop security response procedures...”

In Section 4.1 of the SNMPPPD, the applicant described the procedures that it would develop. Among these were the response procedures.

The staff finds that, because the applicant committed to develop response implementing procedures that would be subject to NRC inspection, the order requirement of Part C.1. would be met.

Applicable Requirement: Part C.2. of the order (Part C.2. of the order contains safeguards information and is not described here).

The applicant addressed Part C.2. of the order in Section 5.10 of the SNMPPPD.

The staff finds that, because the applicant described the response attributes that aligned with Part C.2. of the order, the order requirement of Part C.2. would be met.

Part D of the Order “General”

Applicable Requirement: Part D.1. of the order “hexafluoride”

The applicant addressed this order requirement in Section 1 of the SNMPPPD. The applicant stated that uranium hexafluoride would not be brought on the nuclear power reactor site and was not at all associated with the license application.

The staff finds that, because the applicant described the conditions associated with uranium hexafluoride in regard to the Turkey Point Units 6 and 7 site, the order requirement of Part D.1. would be met.

Applicable Requirement: Part D.2. of the order “Hazardous Material”

The applicant addressed this order requirement in Section 5.9, “Chemicals and Hazardous Materials,” of the SNMPPPD. In addition, a procedure to implement the strategy outlined in Section 5.9 of the SNMPPPD was committed to be developed in Section 4.1 of the SNMPPPD.

The applicant described an acceptable means to reduce storage of hazardous material on-site to the minimal necessary in order to avoid disrupting operations. Therefore, the staff finds that, because the applicant described a strategy to address Part D.2. of the order and committed to development of a procedure to implement that strategy; the order requirement of Part D. 2. would be met.

Applicable Requirement: Part D.3. of the order “Supplement the Emergency Action Levels”

The applicant addressed Part D.3. of the order in Section 5.11, “Emergency Response,” of the SNMPPPD. The applicant committed to supplementing the Emergency Action Levels and their thresholds in response to a range of credible or imminent threats. The staff reviewed the applicant's description of the Emergency Action Level actions to be accomplished and found that the order measure was addressed in an acceptable manner.

The staff finds that, because the applicant described how the requirement of Part D.3. of the order would be addressed, the order requirement of Part D.3. would be met.

Applicable Requirement: Part D.4. of the order “Evaluate computer and communications”

The applicant addressed Part D.4. of the order in Section 5.11 of the SNMPPPD. Specifically, the applicant committed to the evaluation of computer and communication networks for vulnerabilities, including modem access vulnerabilities, and to address them as necessary.

The staff finds that, because the applicant described how the requirement of Part D.4. of the order would be addressed, the order requirement of Part D.4. would be met.

Applicable Requirement: Part D.5. of the order “Evaluate capabilities...fire suppression”

The applicant addressed Part D.5. of the order in Section 5.12, “Fire Response,” of the SNMPPPD. Specifically, the applicant coordinated with off-site fire departments and developed a response plan to notify those departments, if and when necessary, to facilitate fire suppression efforts.

The staff finds that, because the applicant described how the requirement of Part D.5. of the order would be addressed, the order requirement of Part D.5. would be met.

Applicable Requirement: Part D.6. of the order “Evaluate...medical”

The applicant addressed Part D.6. of the order in Section 5.13, “Medical Response,” of the SNMPPPD. Specifically, the applicant identified two local medical care facilities available for utilization given such a need was requested.

The staff finds that, because the applicant described how the requirement of Part D.6. of the order would be addressed, the order requirement of Part D.6. would be met.

Applicable Requirement: Part D.7. of the order “Limit...access...”

The applicant discussed in Section 5.7 how the order requirement D.7 would be addressed. This included a discussion of the restriction of public access to sensitive documents.

The staff finds that, because the applicant described how the requirement of Part D.7. of the order would be addressed, the order requirement of Part D.7. would be met.

Part 3 of the Order “Access Control and Badging”

The applicant stated in Section 5.4, “Access Control and Badging,” of the SNMPPPD that those persons afforded access to the controlled access area would be under the access authorization program as presented in Section 14.1 of their power reactor PSP. Section 14.1 of the Turkey Point Units 6 and 7 power reactor PSP describes an access authorization program in compliance with RG 5.66. The access authorization program as described in RG 5.66 includes fingerprinting and an overall more-stringent access authorization program than that described in Part 3 of the order. In addition, as stated in the SNMPPPD Section 5.4 individuals not under the subject access authorization program would be escorted into, out of, and within the controlled access area in accordance with Section 14.4.6 of the power reactor PSP which describes the escort methodologies developed for the Turkey Point Units 6 and 7 site.

The applicant described that RG 5.66 would be applied to meet Part 3 of the order and is the staff recognizes that in doing so a more stringent access authorization process would be utilized than that described in Part 3 of the order. Therefore, the staff finds that the order requirements of Part 3, which include fingerprinting and other access authorization provisions, would be met.

1.5.5.2 Conclusion and Post Combined License Activities

Based on the above, the NRC staff finds that the information regarding general information, organization and administration, radiation protection, nuclear criticality safety, fire safety, emergency preparedness, and environmental protection to support receipt, storage, and possession of fuel and non-fuel SNM (Fission Chambers), conforms to the applicable guidance in NUREG-1520 and NUREG-0800 and, therefore, is acceptable.

With respect to the applicable physical protection requirements specified in 10 CFR 73.67 and the post-September 11, 2001, security order for the possession, use, and transport of SNM of low strategic significance, the NRC staff reviewed the application and concludes that the relevant information in the application is acceptable because it meets the applicable requirements and the guidance in RG 5.59.

The license condition language in this section has been modified, per a letter from the applicant dated April 8, 2016 (ADAMS Accession No. ML16103A507), confirming the acceptability of the following license conditions proposed by the staff. These changes do not affect the staff's above analysis of the conditions, and therefore, for the reasons discussed in the technical evaluation section above, the staff finds the following license conditions acceptable:

License Condition (1-3) - Subject to the conditions and requirements incorporated herein, the Commission hereby licenses FPL:

- (1) (a) pursuant to the Act and 10 CFR Part 70, to receive and possess at any time, special nuclear material as reactor fuel, in accordance with the limitations for storage and in amounts necessary for reactor operation, described in the FSAR, as supplemented and amended;
- (b) pursuant to the Act and 10 CFR Part 70, to use special nuclear material as reactor fuel, after a Commission finding under 10 CFR 52.103(g) has been made, in accordance with the limitations for storage and amounts necessary for reactor operation, described in the FSAR, as supplemented and amended;
- (2) (a) pursuant to the Act and 10 CFR Parts 30 and 70, to receive, possess, and use, at any time before a Commission finding under 10 CFR 52.103(g), such byproduct and special nuclear material (but not uranium hexafluoride) as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts not exceeding those specified in 10 CFR 30.35(d) and 10 CFR 70.25(d) for establishing decommissioning financial assurance, and not exceeding those specified in 10 CFR 30.72 and 10 CFR 70.22(i)(1);
- (b) pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use, after a Commission finding under 10 CFR 52.103(g), any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as necessary;

(3) (a) pursuant to the Act and 10 CFR Parts 30 and 70, to receive, possess, and use, before a Commission finding under 10 CFR 52.103(g), any byproduct or special nuclear material (but not uranium hexafluoride) that is (1) in unsealed form; (2) on foils or plated surfaces, or (3) sealed in glass, for sample analysis or instrument calibration or other activity associated with radioactive apparatus or components, in amounts not exceeding those specified in 10 CFR 30.35(d) and 10 CFR 70.25(d) for establishing decommissioning financial assurance, and not exceeding those specified in 10 CFR 30.72 and 10 CFR 70.22(i)(1);

(b) pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use, after a Commission finding under 10 CFR 52.103(g), in amounts as necessary, any byproduct, source, or special nuclear material (but not uranium hexafluoride) without restriction as to chemical or physical form, for sample analysis or instrument calibration or other activity associated with radioactive apparatus or components but not uranium hexafluoride; and

(4) pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

- License Condition (1-4) - Prior to initial receipt of special nuclear materials onsite, the licensee shall implement the Special Nuclear Material Control and Accounting Program. No later than 12 months after issuance of the COL, the licensee shall submit to the Director of the Office of New Reactors a schedule that supports planning for and conduct of NRC inspections of the Special Nuclear Material Control and Accounting Program. The schedule shall be updated every 6 months until 12 months before scheduled fuel loading, and every month thereafter until the Special Nuclear Material Control and Accounting Program has been fully implemented.
- License Condition (1-5) – No later than 12 months after issuance of the COL, the licensee shall submit to the Director of the Office of New Reactors a schedule that supports planning for and conduct of NRC inspection of the non-licensed plant staff training program. The schedule shall be updated every 6 months until 12 months before scheduled fuel loading, and every month thereafter until the non-licensed plant staff training program has been fully implemented.
- License Condition (1-6) – Prior to initial receipt of special nuclear material on site, the licensee shall implement the Special Nuclear Material Physical Protection Program. No later than 12 months after issuance of the COL, the licensee shall submit to the Director of the Office of New Reactors a schedule that supports planning for and conduct of NRC inspection of the Special Nuclear Material Physical Protection Program. The schedule shall be updated every 6 months until 12 months before scheduled fuel loading, and every month thereafter until the Special Nuclear Material Physical Protection Program has been fully implemented.