

## Regulatory Guide Periodic Review

**Regulatory Guide Number:** 1.195, Revision 0

**Title:** Methods and Assumptions for Evaluating Radiological Consequences of Design Basis Accidents at Light-Water Nuclear Power Reactors

**Office/division/branch:** NRR/DRA/ARCB  
**Technical Lead:** Elijah D. Dickson

**Staff Action Decided:** Reviewed with no issues identified

**1. What are the known technical or regulatory issues with the current version of the Regulatory Guide (RG)?**

No technical or regulatory issues were identified.

RG 1.195, Revision 0, "Methods and Assumptions for Evaluating Radiological Consequences of Design basis Accidents at Light-Water Nuclear Power Reactors," published May 2003, provides an approach acceptable to the NRC staff for licensees of operating power reactors on acceptable methods and assumptions for performing evaluations of fission product releases and radiological consequences of several postulated light-water reactor design basis accidents. This guide provides guidance on methods acceptable to the staff for determining the radiological sources; the scope, nature, and documentation of associated analyses and evaluations; consideration of impacts on analyzed risk; and the content of submittals acceptable to the NRC staff.

One editorial issue was identified. The next time regulatory guide 1.195 is revised the revision should include deleting the references to the following regulatory guides currently being withdrawn:

- RG 1.3, Rev. 2, "Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss of Coolant Accident for Boiling Water Reactors" dated June 1974;
- RG 1.4, Rev. 2, "Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss of Coolant Accident for Pressurized Water Reactors" dated June 1974;
- RG 1.5 (Safety Guide 5), "Assumptions Used for Evaluating the Potential Radiological Consequences of a Steam Line Break Accident for Boiling Water Reactors" dated March 1971;
- RG 1.25, "Assumptions used for Evaluating the Potential Radiological Consequences of a Fuel Handling Accident in the Fuel Handling and Storage Facility for Boiling and Pressurized Water Reactors" dated March 1972.

**2. What is the impact on internal and external stakeholders of not updating the RG for the known issues, in terms of anticipated numbers of licensing and inspection activities over the next several years?**

As no technical or regulatory issues were identified, there is no impact to internal or external stakeholders resulting from these activities.

- 3. What is an estimate of the level of effort needed to address identified issues in terms of full-time equivalent (FTE) and contractor resources?**

As no technical or regulatory issues were identified, no resources are required.

- 4. Based on the answers to the questions above, what is the staff action for this guide?**

Reviewed with no technical or regulatory issues identified.

- 5. Provide a conceptual plan and timeframe to address the issues identified during the review.**

N/A. No issues were identified during the review.

**NOTE: This review was conducted in July 2016, and reflects the staff's plans as of that date. These plans are tentative and are subject to change.**