

## **NRR-PMDAPEm Resource**

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**From:** Sreenivas, V  
**Sent:** Tuesday, August 30, 2016 1:45 PM  
**To:** Sreenivas, V  
**Subject:** Temporary Verbal Authorization: McGuire Nuclear Station, Unit 1: Relief Request 16-MN-003 Alternative for the Temporary Repair of degraded Nuclear Service Water System Piping.

**Date of Temporary Verbal Authorization: August 29, 2016 at 11 am**  
**Project Manager: V. Sreenivas, (301) 415-2597**

VERBAL AUTHORIZATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELIEF REQUEST 16-MN-003  
ALTERNATIVE REPAIR FOR NUCLEAR SERVICE WATER SYSTEM PIPING  
MCGUIRE NUCLEAR STATION UNIT 1  
DUKE ENERGY  
DOCKET NO. 50-369

**Technical Evaluation read by David Alley, Chief of the Component Performance, Non-Destructive Examination, and Testing Branch, Office of Nuclear Reactor Regulation**

By letters dated August 10 and 18, 2016, Duke Energy (the licensee) requested relief from the requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI, IWA-4420, at McGuire Nuclear Station Unit 1. Pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR) 50.55a(z)(2), the licensee requested to use the alternative in Relief Request 16-MN-003 on the basis that compliance with the specified ASME requirements would result in hardship or unusual difficulty without a compensating increase in the level of quality and safety. Relief Request 16-MN-003 provides an alternative for the temporary repair of degraded nuclear service water system piping.

The licensee proposed to use sleeves as described in ASME Code Case N-786-2 to repair the degraded 3-inch drain line of the nuclear service water system piping. The temporary repair will be effective until next refueling outage. The proposed alternative took exceptions and modifications to certain requirements of Code Case N-786-2. The NRC staff has not approved Code Case N-786-2. However, the NRC staff finds that the proposed alternative with exceptions and modifications to the code case are acceptable. The NRC staff finds the licensee's hardship justification is acceptable.

The NRC staff has determined that the proposed alternative will provide reasonable assurance that the structural integrity and leak tightness of the degraded nuclear service water system piping until next refueling outage.

**Authorization read by Michael Markley, Chief of the Plant Licensing Branch II-1, Office of Nuclear Reactor Regulation**

As Chief of the Plant Licensing Branch II-1, Office of Nuclear Reactor Regulation, I agree with the evaluation made by the Component Performance, Non-Destructive Examination, and Testing Branch.

The NRC staff has determined that complying with the ASME Code requirement would result in a hardship or unusual difficulty without a compensating increase in the level of quality and safety. Furthermore, the NRC staff has determined that the proposed Relief Request 16-MN-003 provides reasonable assurance of structural integrity and leak tightness of the repaired piping. Accordingly, the NRC staff concludes that the licensee has adequately addressed all of the regulatory requirements set forth in 10 CFR 50.55a(z)(2) and, therefore,

authorizes the proposed alternative for the McGuire Nuclear Station Unit 1 until next refueling outage which is scheduled for Fall 2017.

This authorization is for the requested relief and repair at McGuire Nuclear Station Unit 1 and does not imply or infer the NRC's approval of ASME Code Case N-786-2.

All other requirements of ASME Code, Section XI, for which relief was not specifically requested and authorized by the NRC staff remain applicable, including the third party review by the Authorized Nuclear In-service Inspector.

This verbal authorization does not preclude the NRC staff from asking additional questions regarding Relief Request 16-MN-003, while preparing the subsequent written safety evaluation.

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**Mail Envelope Properties** (V.Sreenivas@nrc.gov20160830134400)

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**From:** Sreenivas, V

**Created By:** V.Sreenivas@nrc.gov

**Recipients:**  
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