

## **NRR-PMDAPem Resource**

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**From:** Purnell, Blake  
**Sent:** Wednesday, August 24, 2016 10:03 AM  
**To:** Lynch, Laura:(GenCo-Nuc)  
**Cc:** Miller, Ed; Barstow, James:(GenCo-Nuc)  
**Subject:** Exelon Fleet - Proposed Alternative to Use Code Case OMN-20 (CAC Nos. MF8226 - MF8237)

Ms. Lynch:

By application dated July 26, 2016 (ADAMS Accession No.ML16209A496), Exelon Generation Company, LLC (Exelon, the licensee) submitted a request in accordance with Paragraph 50.55a(z)(2) of Title 10 of the *Code of Federal Regulations* (10 CFR) for a proposed alternative to the requirements of 10 CFR 50.54(f) and the American Society of Mechanical Engineers (ASME) Operation and Maintenance Code (ASME Code) for Braidwood Station, Units 1 and 2; Clinton Power Station, Unit No. 1; LaSalle County Station, Units 1 and 2; Limerick Generating Station, Units 1 and 2; Nine Mile Point Nuclear Station, Units 1 and 2; Peach Bottom Atomic Power Station, Units 2 and 3; and R. E. Ginna Nuclear Power Plant. The proposed alternative would allow the licensee to use ASME Code Case OMN-20, "Inservice Test Frequency," as an alternative to the inservice testing frequencies for pumps and valves specified in ASME OM Code, Division 1, Section IST.

The purpose of this email is to provide the results of the U.S. Nuclear Regulatory Commission (NRC) staff's acceptance review of this proposed alternative. The acceptance review was performed to determine if there is sufficient technical information in scope and depth to allow the NRC staff to complete its detailed technical review. The acceptance review is also intended to identify whether the application has any readily apparent information insufficiencies in its characterization of the regulatory requirements or the licensing basis of the plant.

Pursuant to 10 CFR 50.55a(z), the applicant shall demonstrate that the proposed alternatives would provide an acceptable level of quality and safety, or that compliance with the specified requirements of 10 CFR 50.55a would result in hardship or unusual difficulty without a compensating increase in the level of quality or safety.

The NRC staff has reviewed your application and concluded that it provides technical information in sufficient detail to enable the staff to complete its detailed technical review and make an independent assessment regarding the acceptability of the proposed relief request in terms of regulatory requirements and the protection of public health and safety and the environment. Given the lesser scope and depth of the acceptance review as compared to the detailed technical review, there may be instances in which issues that impact the staff's ability to complete the detailed technical review are identified despite completion of an adequate acceptance review. You will be advised of any further information needed to support the staff's detailed technical review by separate correspondence.

Sincerely,

Blake Purnell, Project Manager  
Plant Licensing Branch III-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission

Docket Nos. STN 50-456, STN 50-457, 50-461,  
50-373, 50-374, 50-352, 50-353,  
50-220, 50-410, 50-277, 50-278,

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Tracking Status: None  
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