Primary,Secondary, and Review Interface SRP Update Review Resposibilities

Definitions of responsibilities for the primary review branch, secondary review branch, and review interface branch are included in Chapter 4.0, Review Organization, of NRO-REG-300, *Maintaining and Updating the Standard Review Plan*. The table below reflects the current primary, secondary, and review interface responsibilities for the Standard Review Plan (SRP) update process. This table will be updated as review responsibilities change.

Current Revision Date - March 2017

SRP Section	Section Title	Primary DIV	Primary Review Branch	Secondary Review DIV	Secondary Review Branch	Secondary Reviewer Justification	Review Interface Division	Review Interface Branch	Review Interface Justification	Latest Revision	Revision Date	Currrent Revision ADAMS #
1.0	Introduction and Interfaces	DNRL	LB1, LB2, LB3, LB4	All review organizations	Multiple					Rev. 1		
2.0	Site Characteristics and Site Parameters	DNRL	LB1, LB2, LB3, LB4	All SRP Chapter 2 review organizations	Multiple		DSEA	RPAC	Site characteristics (for ESPs, COLs, CPs and OLs) or site parameters (for DCs), specifically the short-term (accident) atmospheric dispersion factors and distances to the EAB and LPZ, listed under this sectionare used as input to the DBA dose analyses reviewed under SRP 15.0.3.	Rev. 1	Oct-16	ML15279A105
2.1.1	Site Location and Description	DSEA	RPAC				DSEA	RMOT	RMOT needs to know the location of the EAB and outer boundary of the LPZ in order to perform atmospheric dispersion estimates for these locations	Rev. 3	Mar-07	ML070550023
2.1.2	Exclusion Area Authority and Control	DSEA	RPAC	NSIR-DPR						Rev. 3	Mar-07	ML070550024
2.1.3	Population Distribution	DSEA	RPAC	NSIR-DPR	LIB under EP		DSEA	RENV RPAC	RENV and RPAC are end users for information in ESRP Section 2.5.1, Demography, for socioeconomic, alternative siting, radiological health, and severe accident impacts	Rev. 3	Mar-07	ML070550028
2.2.1-2.2.2	Identification of Potential Hazards in Site Vicinity	DSEA	RPAC	DSRA	SCVB	control room habitability				Rev. 3	Mar-07	ML070460330
2.2.3	Evaluation of Potential Accidents	DSEA	RPAC	DSEA/DSRA	RMOT/SCVB	RMOT has access to onsite meteorological data that can be used to perform atmospheric dispersion estimates for potential accidents. SCVB addresses control room habitability.				Rev. 3	Mar-07	ML070460336

2.3.1	Regional Climatology	DSEA	RMOT				DEIA, DSRA, DSEA	SEB, SPSB, SCVB, SPCA, RENV	RMOT provides input to the design wind speed used in SRP 3.3.1 and the design basis tornado used in SRP 3.3.2, coordinates the review of the design-basis extreme wind parameters udes in 3.5.1.4, and provides input to the severe and extreme environmental conditions used in SRP 3.8.1, 3.8.2, 3.8.4, 9.2.5, 9.4.1, 19.0 and 19.3. RENV applies the same informtion in ESRP Section 2.7. This insures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML063600393
2.3.2	Local Meteorology	DSEA	RMOT				NRR-DE, DSEA	EEEB, RENV	RENV applies the same information in ESRP Sections 2.7 and 5.3.3.1. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML070730395
2.3.3	Onsite Meteorological Measurements Programs	DSEA	RMOT				DEIA, DSRA, DSEA	RPAC, LIB under EP, RENV	The meteorological monitoring program provides input to the Type E accident monitoring instrumentation discussed in SRP 7.5. RMOT also reviews the meteorological monitoring system described in SRP 13.3. RPAC is an end user of hourly meteorological data for off-site dose consequences in ESRP 7.2. RENV applies the same information in ESRP Sections 2.7 and 6.3. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML063600394

2.3.4	Short Term Dispersion Estimates for Accidental Atmospheric Releases	DSEA	RMOT	DSEA	RPAC		DSEA	RPAC, RENV	The short-term atmospheric dispersion factors reviewed under this SRP section are used as input to the DBA dose analyses reviewed under SRP 6.4, 15.0.1, and 15.0.3. RPAC is end user of short-term X/Q parameters for offsite gaseous effluent dose calculations in SRP 11.3. RENV applies the same information in ESRP Sections 2.7 and 5.3.3.1. RPAC applies the same information in ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML070730398
2.3.5	Long-Term Atmospheric Dispersion Estimates for Routine Releases	DSEA	RMOT	DSEA	RPAC	RPAC is end user of short/long-term X/Q and D/Q parameters for offsite gaseous effluent dose calculations in SRP 11.3.	DSEA	RENV	RENV applies the same informtion in ESRP Sections 2.7. RPAC applies the same information in ESRP Sections 4.5 and 5.4. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML070730713
2.4.1	Hydrologic Description	DSEA	RHM1, RHM2				DSEA	RENV	RENV bases ESRP Section 2.3.1 on the same information. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML070100646
2.4.2	Floods	DSEA	RHM1, RHM2	DSEA	RMOT	RMOT - Review of rainfall depth values in these analyses.	DSEA	RENV	RENV bases ESRP Section 2.3.1 on the same information. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML070100647
2.4.3	Probable Maximum Flood (PMF) on Streams and Rivers	DSEA	RHM1, RHM2	DSEA	RMOT	Review of meteorological values (rainfall and wind speeds) in these analyses.				Rev. 3	Mar-07	ML070730405
2.4.4	Potential Dam Failures	DSEA	RHM1, RHM2	DSEA	RGS	Review of seismic and geotechnical aspects of dam failures				Rev. 3	Mar-07	ML070730417
2.4.5	Probable Maximum Surge and Seiche Flooding	DSEA	RHM1, RHM2	DSEA	RMOT	Review of meteorological values (hurrican parameters, wind speeds for seiche, etc.) in these analyses.	DSEA	RENV	Sea level rise and climate change aspects is pone factor in the coastal hazards analysis.	Rev. 3	Mar-07	ML070730425
2.4.6	Probable Maximum Tsunami Hazards	DSEA	RHM1, RHM2							Rev. 3	Mar-07	ML070160659

2.4.7	Ice Effects	DSEA	RHM1, RHM2	DSEA	RMOT	Review of meteorlogical values (freezing degree days) in these analyses.				Rev. 3	Mar-07	ML070100648
2.4.8	Cooling Water Canals and Reservoirs	DSEA	RHM1, RHM2				DSEA	RENV	RENV bases ESRP Section 2.3.1 on the same information. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML070730431.
2.4.9	Channel Diversions	DSEA	RHM1, RHM2							Rev. 3	Mar-07	ML070730434
2.4.10	Flooding Protection Requirements	DSEA	RHM1, RHM2				DEIA	SEB	SEB is the end user of the subsurface information in the structural design and the scope includes the impact on structures.	Rev. 3	Mar-07	ML0707340437
2.4.11	Low Water Considerations	DSEA	RHM1, RHM2				DSEA	RENV	RENV bases ESRP Section 2.3.1 on the same information. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML070730439
2.4.12	Groundwater	DSEA	RHM1, RHM2				DEIA, DSEA	SEB, RENV, and RGS	RENV - bases ESRP Section 2.3.1 on the same information. This ensures consistency between the SER and the EIS. RGS - regarding description of the subsurface. SEB - structural design and the scope includes the impact on structures.	Rev. 3	Mar-07	ML070730443
2.4.13	Accidental Releases of Radioactive Liquid Effluents in Ground and Surface Waters	DSEA	RHM1, RHM2	DSEA	RPAC		DSEA	RENV	RENV bases ESRP Section 2.3.1 on the same information. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML070730449
2.4.14	Technical Specifications and Emergency Operation Requirements	DSEA	RHM1, RHM2							Rev. 3	Mar-07	ML070730455
2.5.1	Geologic and Seismic Information	DSEA	RGS				DSEA	RENV	RENV bases ESRP Section 2.6 on similar information. This ensures consistency between the SER and the EIS.	Rev. 5	Jul-14	ML13316C067
2.5.2	Vibratory Ground Motion	DSEA	RGS							Rev. 5	Jul-14	ML13316C066
2.5.3	Surface Faulting	DSEA	RGS							Rev. 5	Jul-14	ML13316C064

2.5.4	Stability of Subsurface Materials and Foundations	DSEA	RGS				DSEA, DEIA	RHM1 and 2, SEB	For hydrologist, review groundwater condition; For structural engineer, review bearing capacity, settlement, lateral earth pressure, and liquefaction potential that could have impact on structural analyses. SEB performs the structural review.	Rev. 5	Jul-14	ML13311B744
2.5.5	Stability of Slopes	DSEA	RGS				DEIA	SEB	For structural engineer, review slope stability issues that may impact structures.	Rev. 5	Jul-14	ML13316C068
3.2.1	Seismic Classification	DEIA	MEB	DCIP, DSEA	MVIB, RPAC		DSEA	RPAC	Seismic Classifications are used for evaluations of systems and components for RG 1.143.	Rev 3	Aug-16	ML16084A812
3.2.2	System Quality Group Classification	DEIA	MEB	DCIP	MVIB		DSEA	RPAC	Seismic Classifications are used for evaluations of systems and components for RG 1.143.	Rev 3	Aug-16	ML16084A884
3.3.1	Wind Loads	DEIA	SEB				DSEA	RMOT	RMOT provides input to the design wind speed used in this SRP section	Rev. 3	Mar-07	ML070570001
3.3.2	Tornado Loadings	DEIA	SEB				DSEA	RMOT	RMOT provides input to the design basis tornado used in this SRP section	Rev. 3	Mar-07	ML070570002
3.4.1	Internal Flood Protection for Onsite Equipment Failure	DSRA	SPSB	DSEA	RHM1, RHM2					Rev. 3	Mar-07	ML070550043
3.4.2	Analysis Procedures	DEIA	SEB							Rev. 3	Mar-07	ML070570003
3.5.1.1	Internally Generated Missiles (Outside Containment)	DSRA	SPSB							Rev. 3	Mar-07	ML070370569
3.5.1.2	Internally Generated Missiles (Inside Containment)	DSRA	SPSB							Rev. 3	Mar-07	ML070380167
3.5.1.3	Turbine Missiles	DEIA	MCB	DEIA	SEB	SEB is the end user of the missile information in the structural design and the scope includes the impact on structures.				Rev. 3	Mar-07	ML063600395
3.5.1.4	Missiles Generated by Extreme Winds	DSRA	SPSB				DSEA	RMOT	RMOT coordinates the review of the design-basis extreme wind parameters used in this SRP section	Rev. 4	Mar-15	ML14190A180

3.5.1.5	Site Proximity Missiles (Except Aircraft)	DSEA	RPAC	DSRA	SPSB	structural review				Rev. 4	Mar-07	ML070510635
3.5.1.6	Aircraft Hazards	DSEA	RPAC	DSRA	SPSB		DSRA	SPRA	PRA review	Rev. 4	Mar-10	ML100331298
3.5.2	Structures, Systems, and Components To Be Protected From Externally Generated Missiles	DSRA	SPSB							Rev. 3	Mar-07	ML070460362
3.5.3	Barrier Design Procedures	DEIA	SEB							Rev. 3	Mar-07	ML070570004
3.6.1	Plant Design for Protection Against Postulated Piping Failures in Fluid Systems Outside Containment	DSRA	SPSB							Rev. 3	Mar-07	ML070550032
3.6.2	Determination of Rupture Locations and Dynamic Effects Associated with the Postulated Rupture of Piping	DEIA	MEB							Rev 3	Dec-16	ML14230A035
3.6.3	Leak-Before-Break Evaluation Procedures	DEIA	MCB	DSRA	SBSP	SPSB supports MEB, when requested, by reviewing leak detection systems				Rev. 1	Mar-07	ML063600396
3.7.1	Seismic Design Parameters	DEIA	SEB	DSEA	RGS					Rev. 4	Dec-14	ML14198A460
3.7.2	Seismic System Analysis	DEIA	SEB							Rev. 4	Sep-13	ML.13198A223
3.7.3	Seismic Subsystem Analysis Review Responsibilities	DEIA	SEB							Rev. 4	Sep-13	ML13198A239
3.7.4	Seismic Instrumentation	DSEA	RGS	DSEA	RPAC					Rev .3	Jul-14	ML13324A570
3.8.1	Concrete Containment	DEIA	SEB				DSEA	RMOT	RMOT provides input to the severe and extreme environmental conditions used in this SRP section	Rev. 4	Sep-13	ML13198A245
3.8.2	Steel Containment	DEIA	SEB				DSEA	RMOT	RMOT provides input to the severe and extreme environmental conditions used in this SRP section	Rev. 3	May-10	ML100630179
3.8.3	Concrete and Steel Internal Structures of Steel or Concrete Containments	DEIA	SEB	DSEA	RPAC	Organization responsible for the review of Radiation Protection. These reviews are used to assess the adequacy of the shielding design, consistent with 10 CFR 50.34(b)(3), 10 CFR 50.49, 10 CFR 50.34(f)(2)(vii) and GDC 61.				Rev. 4	Sep-13	ML13198A250.

3.8.4	Other Seismic Category I Structures	DEIA	SEB	DSEA	RPAC	RPAC - Organization responsible for the review of Radiation Protection. These reviews are used to assess the adequacy of the shielding design, consistent with 10 CFR 50.34(b)(3), 10 CFR 50.49, 10 CFR 50.34(f)(2)(vii) and GDC 61. Organization responsible for the review of Radioactive Waste Management. Organization responsible for the review of Radiation Protection. These reviews are used to assess the adequacy of the design, consistent with the guidance of RG 1.143, for the protection of members of the public and occupational workers from the effects of damage to SSCs containing radioactive material.	DSEA	RMOT	RMOT provides inout to the severe and extreme environmental conditions used in this SRP section. These reviews are used to assess the adequacy of the design, consistent with the guidance of RG 1.143, for the protection of members of the public and occupational workers from the effects of damage to SSCs containing radioactive materials.	Rev. 4	Sep-13	ML13198A258
3.8.5	Foundations	DEIA	SEB							Rev. 4	Sep-13	ML13198A267
3.9.1	Special Topics for Mechanical Components	DEIA	MEB							Rev 4	Dec-16	ML16088A068
3.9.2	Dynamic Testing and Analysis of Systems, Components, and Equipment	DEIA	MEB							Rev.3	Mar-07	ML070230008
3.9.3	ASME Code Class 1, 2, and 3 Components, Component Supports, and Core Support Structures	DEIA	MEB							Rev. 3	Apr-14	ML14043A231
3.9.4	Control Rod Drive Systems	DEIA	MEB							Rev.3	Mar-07	ML063190004
3.9.5	Reactor Pressure Vessel Internals	DEIA	MEB							Rev.3	Mar-07	ML070230009
3.9.6	Functional Design, Qualification, and Inservice Testing Programs for Pumps, Valves, and Dynamic Restraints	DEIA	MEB							Rev.3	Mar-07	ML070720041
3.9.7	Risk-Informed Inservice Testing	DEIA	MEB	DSRA	SPRA	SPRA reviews the PRA technical adequacy for the risk-informed application and provides SE input.				Rev. 0	Aug-98	ML042880272
3.9.8	Risk-Informed Inservice Inspection of Piping	DEIA	MCB	DSRA	SPRA	SPRA reviews the PRA technical adequacy for the risk-informed application and provides SE input.				Rev. 0	Sep-03	ML032510135

3.10	Seismic and Dynamic Qualification of Mechanical and Electrical Equipment	DEIA	MEB	DEIA	EEB					Rev. 4	Dec-16	ML16088A101
3.11	Environmental Qualification of Mechanical and Electrical Equipment	DEIA	ICE	DEIA, DSEA	MEB, ICE, RPAC	**RPAC** - Organization responsible for the review of Radiation Protection. 10 CFR 50.49 explicitly identifies Radiation as one of the environmental conditions to be considered, section (e)(4) Radiation. The radiation environment must be based on the type of radiation, the total dose expected during normal operation over the installed life of the equipment, and the radiation environment associated with the most severe design basis accident during or following which the equipment is required to remain functional, including the radiation resulting from recirculating fluids for equipment located near the recirculating lines and including dose rate effects. The current SRP already discusses the need to evaluate the effects of different types of radiation on the qualification of equipment. This falls in the purview of DSEA/RPAC. For the organization responsible for the review of design basis accident radiological consequence analyses the information evaluated under the SRP 15.0.3 review of DBA dose analyses, in particular the LOCA releases, are used as input to the EQ assessment evaluated in this SRP section. The information should be coordinated between the SRP sections. **MEB**: branch contributes to the content of the SRP but is not the owner. MEB addresses qualification of mechanical equipment QME-1. **ICE** is not the lead but the scope of the review includes I&C equipment				Rev. 3	Mar-07	ML063600397
3.12	ASME Code Class 1, 2, and 3 Piping Systems and Associated Supports Design [new]	DEIA	MEB							Rev. 1	Apr-14	ML14042A513
3.13	Threaded Fasteners - ASME Code Class 1, 2, and 3	DEIA	MCB							Rev. 0	Mar-07	ML070550058
BTP 3-1	Classification of Main Steam Components Other than the Reactor Coolant Pressure Boundary for BWR Plants	DEIA	MEB							Rev. 2	Mar-07	ML070800127
BTP 3-2	Classification of BWR/6 Main Steam and Feedwater Components Other than the Reactor Coolant Pressure Boundary	DEIA	MEB							Rev. 2	Mar-07	ML070800194
BTP 3-3	Protection Against Postulated Piping Failures In Fluid Systems Outside Containment	DSRA	SPSB							Rev. 3	Mar-07	ML070800027

BTP 3-4	Postulated Rupture Locations in Fluid System Piping Inside and Outside Containment	DEIA	MEB							Rev. 3	Dec-16	ML16085A315
4.2	Fuel System Design	DSRA	SRSB				DSEA	RPAC	Information on fuel design listed in the related section of the DCD or FSAR is used in the development of the core fission product inventory which is reviewed under SRP 15.0.3. Postulated fuel failures reviewed under this SRP section are used as input to the DBA dose analyses reviewed under SRP 15.0.3. Radiation Protection reviews this section to identify the kinds and quantities of radioactive material, that may result from fuel or neutron activated components located in or near the reactor core, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5).	Rev. 3	Mar-07	ML070740002
4.3	Nuclear Design	DSRA	SRSB							Rev. 3	Mar-07	ML070740003
4.4	Thermal and Hydraulic Design	DSRA	SRSB							Rev. 2	Mar-07	ML070550060
4.5.1	Control Rod Drive Structural Materials	DEIA	MCB				DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive material, that may result from fuel or neutron activated components located in or near the reactor core, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5).	Rev. 3	Mar-07	ML070230007

4.5.2	Reactor Internal and Core Support Structure Materials	DEIA	MCB				DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive material, that may result from fuel or neutron activated components located in or near the reactor core, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5)	Rev. 3	Mar-07	ML063190005
4.6	Functional Design of Control Rod Drive System	DSRA	SRSB							Rev. 2	Mar-07	ML070540139
BTP 4.1	Westinghouse Constant Axial Offset Control (CAOC)	DSRA	SRSB							Rev. 3	Mar-07	ML070790015.
5.2.1.1	Compliance With American Society of Mechanical Engieers Code Requirements in the Codes and Standards Rule, 10 CFR 50.55a	DEIA	MEB	DEIA	MCB	MCB shares responsibility with MEB regardging application of ASME Code requirements				Rev 4	Dec-16	ML14227A623.
5.2.1.2	Applicable Code Cases	DEIA	MEB	DEIA	MCB	MCB shares responsibility with MEB regardging application of ASME Code requirements				Rev 4	Dec-16	ML16088A219
5.2.2	Overpressure Protection	DSRA	SRSB							Rev. 3	Mar-07	ML070540076.
5.2.3	Reactor Coolant Pressure Boundary Materials	DEIA	MCB				DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive material, that may result from neutron activation of corrosion and wear products, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5).	Rev. 3	Mar-07	ML063190006
5.2.4	Reactor Coolant Pressure Boundary Inservice Inspection and Testing	DEIA	MCB							Rev. 2	Mar-07	ML070550066

5.2.5	Reactor Coolant Pressure Boundary Leakage Detection	DSRA	SPSB				DSEA	RPAC	Radiation Protection reviews this section to identify the type and capability of radiation monitoring equipment provided for RCS leakage detection, in accordance with GDC 14 and 10 CFR 55a. RPAC: RPAC is end user of RCPB leakage detection information in SRP 11.2, 11.3, and 11.5.	Rev. 2	Mar-07	ML070610277
5.3.1	Reactor Vessel Materials	DEIA	MCB				DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive material, that may result from neutron activation of corrosion and wear products, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5).	Rev. 2	Mar-07	ML063190007
5.3.2	Pressure-Temperature Limits, Upper-shelf Energy, and Pressurized Thermal Shock	DEIA	MCB							Rev. 2	Mar-07	ML070380185
5.3.3	Reactor Vessel Integrity	DEIA	MCB							Rev. 2	Mar-07	ML063190008
5.4	Reactor Coolant Systems Components and Subsystem Design	DSRA	SRSB	Multiple as defined in the SRP			DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive fission, activation and corrosion products contained in the tanks, heat exchangers and related components which interface with the RCS, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5).	Rev. 2	Mar-07	ML070610167
5.4.1.1	Pump Flywheel Integrity (PWR)	DEIA	MCB							Rev. 3	May-10	ML100680157

5.4.2.1	Steam Generator Materials	DEIA	MCB				DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive material, that may result from neutron activation of corrosion and wear products, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5).	Rev. 4	Jul-16	ML16147A289
5.4.2.2	Steam Generator Program	DEIA	MCB				DSEA	RPAC	Radiation Protection reviews this section to identify radiation protection elements, such as radiation monitoring equipment described in Chapter 12 that is used to detect primary to secondary leakage, of NEI 97-06 and tne required EPRI implementing documents, consistent with the Steam Generator Program described in Technical Specifications.	Rev. 2	Mar-07	ML070380194
5.4.6	Reactor Core Isolation Cooling System (BWR)	DSRA	SRSB				DSEA	RPAC	RPAC is end user of reactor core isolation cooling system (BWR) in SRP 11.2, 11.3, and 11.5.	Rev. 4	Mar-07	ML070540102
5.4.7	Residual Heat Removal (RHR) System	DSRA	SRSB				DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive fission, activation and corrosion products contained in the pumps, heat exchangers and related components which interface with the RCS, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5).	Rev. 5	May-10	ML100680577

5.4.8	Reactor Water Cleanup System (BWR)	DEIA	MCB				DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive fission, activation and corrosion products contained in the pumps, heat exchangers and related components which interface with the RCS, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5). RPAC is end user of reactor cleanup water system (BWR) information in SRP 11.2, 11.3, and 11.5.	Rev. 3	Mar-07	ML063190009
5.4.11	Pressurizer Relief Tank	DSRA	SRSB				DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive fission, activation and corrosion products contained in the tanks and related components which interface with the RCS, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5).	Rev. 4	May-10	ML100700178
5.4.12	Reactor Coolant System High Point Vents	DSRA	SRSB							Rev. 1	Mar-07	ML070770005

5.4.13	Isolation Condenser System (BWR)	DSRA	SRSB				DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive fission, activation RPAC is end user of isolation condenser system (BWR) information in SRP 11.2, 11.3, and 11.5. and corrosion products contained in the tanks, heat exchangers and related components which interface with the RCS, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5). 10 CFR 20.1406, as it relates to minimizing contamination of the facility and the environment, resulting from the operation of the system. RPAC is end user of isolation condenser system (BWR) information in SRP 11.2, 11.3, and	Rev. 0	Mar-07	ML070810517
BTP 5-1	Monitoring of Secondary Side Water Chemistry in PWR Steam Generators	DEIA	MCB				DSEA	RPAC	RPAC is end user of monitoring of secondary side water chemistry in PWR steam generators information as it relates to sensitivity of installed radioactivity detectors in SRP 11.2, 11.3, and 11.5.	Rev. 3	Mar-07	ML070850019
BTP 5-2	Overpressure Protection of Pressurized-Water Reactors While Operating at Low Temperatures	DSRA	SRSB							Rev. 3	Mar-07	ML070850008
BTP 5-3	Fracture Toughness Requirements	DEIA	MCB							Rev. 2	Mar-07	ML070850035.
BTP 5-4	Design Requirements of the Residual Heat Removal System	DSRA	SRSB							Rev. 4	Mar-07	ML070850123

6.1.1	Engineered Safety Features Materials	DEIA	MCB				DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive material, that may result from neutron activation of corrosion and wear products, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5).	Rev. 2	Mar-07	ML063190010
6.1.2	Protective Coating Systems (Paints)□Organic Materials	DEIA	MCB				DSEA	RPAC	RPAC is end user of epoxy coating information in SRP 11.2 and 11.4.	Rev. 3	Mar-07	ML063600399
6.2.1	Containment Functional Design	DSRA	SCVB				DSEA	RPAC	Radiation Protection reviews this section to identify the design features provided to minimize ORE due to operation, inspection and maintenance of containment SSCs, consistent with 10 CFR 20.1101.	Rev. 3	Mar-07	ML070220505
6.2.1.1.A	PWR Dry Containments, Including Subatmospheric Containments	DSRA	SCVB							Rev. 3	Mar-15	ML063600402
6.2.1.1.B	Ice Condenser Containments	DSRA	SCVB							draft Rev 3	Jun-96	ML052070482
6.2.1.1.C	Pressure-Suppression Type BWR Containments	DSRA	SCVB							Rev. 7	Mar-15	ML063600403.
6.2.1.2	Subcompartment Analysis	DSRA	SCVB							Rev. 3	Mar-07	ML070620009
6.2.1.3	Mass and Energy Release Analysis for Postulated Loss of Coolant Accidents	DSRA	SCVB							Rev. 3	Mar-07	ML053560191
6.2.1.4	Mass and Energy Release Analysis for Postulated Secondary System Pipe Ruptures	DSRA	SCVB							Rev. 2	Mar-07	ML070620010.
6.2.1.5	Minimum Containment Pressure Analysis for Emergency Core Cooling System Performance Capability Studies	DSRA	SCVB							Rev. 3	Mar-07	ML063600405.

6.2.2	Containment Heat Removal Systems	DSRA	SCVB	DEIA	MCBMEB	**MCB** provides support regarding water chemisty/chemical engineering and containment coatings aspects. **MEB** reviews strainers, pipe break assumption for GSI-191.	DSEA	RPAC	Radiation Protection reviews this section to identify the design features provided to minimize occupational radiation exposure (ORE) due to operation, inspection and maintenance of SSCs, consistent with 10 CFR 20.1101.	Rev. 5	Mar-07	ML070160661
6.2.3	Secondary Containment Functional Design	DSRA	SCVB							Rev. 3	Mar-07	ML063600406
6.2.4	Containment Isolation System	DSRA	SCVB				DSEA	RPAC	RPAC is end user of containment isolation system information as it relates to containment purging/venting design features minimizing purging time consistent with ALARA for occupational exposure, and review of the radiological dose consequence analysis for the release of containment atmosphere prior to closure of containment isolation valves in lines providing a direct path to the environs.	Rev. 3	Mar-07	ML070380197
6.2.5	Combustible Gas Control in Containment	DSRA	SCVB							Rev. 3	Mar-07	ML070620006
6.2.6	Containment Leakage Testing	DSRA	SCVB							Rev. 3	Mar-07	ML070620007
6.2.7	Fracture Prevention of Containment Pressure Boundary	DEIA	MCB							Rev. 1	Mar-07	ML063600407
6.3	Emergency Core Cooling System	DSRA	SRSB	misc. as stated in SRP						Rev. 3	Mar-07	ML070550068
6.4	Control Room Habitability System	DSRA	SCVB	DSEA, DEIA	RPAC, MCB	RPAC reviews Control Room radiological habitability.	DSEA	RMOT	RMOT provides the atmospheric discpersion factors used in this SRP section.	Rev. 3	Mar-07	ML070550069

6.5.1	ESF Atmosphere Cleanup Systems	DSRA	SCVB	DSEA	RPAC		DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive fission, activation and corrosion products contained in the filters and related components which may collect radioactive material, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5).	Rev. 4	May-10	ML100700256
6.5.2	Containment Spray as a Fission Product Cleanup System	DEIA	MCB	DSEA, DSRA	RPAC, SCVB	SCVB reviews and provides inputs to the staff SE on sections relating to the system operation design features, and performance with respect to spray distribution, droplet size and containment mixing. RPAC provides input on the review of the fission product removal capability of these systems and the effect on the DBA dose analyses.				Rev. 4	Mar-07	ML070190178
6.5.3	Fission Product Control Systems and Structures	DSRA	SCVB	DSEA	RPAC	RPAC as the organization responsible for review of design basis accident radiolgical consequences provides input on the review of the fission product removal capability of these systems and the effect on the DBA dose analyses.				Rev. 3	Mar-07	ML063600408
6.5.4	Ice Condenser as a Fission Product Cleanup System	DSEA	RPAC				DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive material, that may accumulate on the SSC, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5), 10 CFR 50.34(f)(xxviii) GDC 19, and 10 CFR 20.1101	Draft Rev 4	Apr-96	ML052070476

6.5.5	Pressure Suppression Pool as a Fission Product Cleanup System	DSEA	RPAC	DSRA, DEIA	SCVB, MCB	**SCVB** reviews and provides inputs to the staff SE on sections relating to system operation, design features and performance related to containment integrity and containment atmosphere decontamination functions. **MCB** provides support regarding water chemisty/chemical engineering aspects	DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive material, that may accumulate on the SSC, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5), 10 CFR 50.34(f)(xxviii) GDC 19, and 10 CFR 20.1101	Rev. 1	Mar-07	ML063600409
6.6	Inservice Inspection of Class 2 and 3 Components	DEIA	MCB							Rev. 2	Mar-07	ML070550071
6.7	Main Steam Isolation Valve Leakage Control System (BWR)	DSRA	SPSB							draft Rev 3	Apr-96	ML052070451
BTP 6-1	pH for Emergency Coolant Water for Pressurized Water Reactors	DEIA	MCB							Rev. 3	Mar-07	ML063190011
BTP 6-2	Minimum Containment Pressure Model for PWR ECCS Performance Evaluation	DSRA	SCVB							Rev. 3	Mar-07	ML070740442
BTP 6-3	Determination of Bypass Leakage Paths in Dual Containment Plants	DSRA	SCVB	DSEA	RPAC	RPAC as the organization responsible for review of design basis accident radiological consequence analyses is confirmed to have secondary review responsibility. RPAC is not a secondary review branch for the SER. The results of the review under this SRP section are used as inputs to the SRP 15.0.3 review of DBA dose analyses.				Rev. 3	Mar-07	ML070740004

BTP 6-4	Containment Purging During Normal Plant Operations	DSRA	SCVB				DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive material, that may accumulate on the SSC, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5), 10 CFR 50.34(f)(xxviii) GDC 19, and 10 CFR 20.1101. Also, how the design and operation parameter of the ventilation system effects airborne radioactivity concentrations, consistent with 10 CFR 20 Subpart H	Rev. 3	Mar-07	ML070740319
BTP 6-5	Currently the Responsibility of Reactor Systems Piping from the RWST (or BWST) and Containment Sump(s)to the Safety Injection Pumps	DSRA	SCVB							Rev. 3	Mar-07	ML070740429
7.0	Instrumentation and Controls - Overview of Review Process	DEIA	ICE	DCIP	HOIB	for inventory of controls and alarms.	DSEA	RPAC	RPAC is end user of instrumentation, components, and controls information in SRP 11.2, 11.3, 11.4, and 11.5.	Rev. 7	Aug-16	ML16020A049
Appendix 7.0-A	Review Process for Digital Instrumentation and Control Systems	DEIA	ICE							Rev. 6	Aug-16	ML16019A085
7.1	Instrumentation and Controls - Introduction	DEIA	ICE				DSEA	RPAC	Radiation Protection reviews this section to identify the instrumentation and contros associated with radiation monitoring equipment, and radiation protection controls described in Chapter 12, in accordance with 10 CFR 20 and 10 CFR 50.34. The staff reviews design features provided to minimize occupational radiaiton exposure resulting from maintenance and calibration, consitent with 10 CFR 20.1101.	Rev. 6	Aug-16	ML16020A050

Appendix 7.1-A	Acceptance Criteria and Guidelines for Instrumentation and Control Systems Important to Safety	DEIA	ICE							Rev. 6	Aug-16	ML16019A058
Appendix 7.1-B	Guidance for Evaluation of Conformance to IEEE Std. 279	DEIA	ICE							Rev. 6	Aug-16	ML16019A091
Appendix 7.1-C	Guidance for Evaluation of Conformance to IEEE Std. 603	DEIA	ICE							Rev. 6	Aug-16	ML16019A107
Appendix 7.1-D	Guidance for Evaluation of the Application of IEEE Std. 7-4.3.2	DEIA	ICE							Rev. 1	Aug-16	ML16019A114
Table 7.1	Regulatory Requirements, Acceptance Criteria, and Guidelines For Instrumentation and Control Systems Important to Safety	DEIA	ICE							Rev. 6	Aug-16	ML16020A103
7.2	Reactor Trip System	DEIA	ICE							Rev. 6	Aug-16	ML16020A059
7.3	Engineered Safety Features Systems	DEIA	ICE				DSEA	RPAC	Radiation Protection reviews this section to identify the instrumentation and contros associated with radiation monitoring equipment, and radiation protection controls described in Chapter 12, in accordance with 10 CFR 20 and 10 CFR 50.34. The staff reviews design features provided to minimize occupational radiaiton exposure resulting from maintenance and calibration, consitent with 10 CFR 20.1101.	Rev. 6	Aug-16	ML16020A082
7.4	Safe Shutdown Systems	DEIA	ICE				DSEA	RPAC	Radiation Protection reviews this section to ensure that areas identified as needed to ensure safe shut down of the plant, are accessible during the anticipated conditions, consistent with 10 CFR 50.34 and 10 CFR 20.	Rev. 6	Aug-16	ML16020A086

7.5	Information Systems Important to Safety	DEIA	ICE				DSEA	RPAC, RMOT	RPAC is end user of information systems important to safety in SRP 11.2, 11.3, 11.4, and 11.5. The meteorological monitoring program provides input to the Type E accident monitoring instrumentation.	Rev. 6	Aug-16	ML16020A088
7.6	Interlock Systems Important to Safety	DEIA	ICE							Rev. 6	Aug-16	ML16020A092
7.7	Control Systems	DEIA	ICE	DSRA	SRSB	SRSB reviews portions related to the protection system	DSEA	RPAC	Radiation Protection reviews this section to identify the instrumentation and contros associated with radiation monitoring equipment, and radiation protection controls described in Chapter 12, in accordance with 10 CFR 20 and 10 CFR 50.34. The staff reviews design features provided to minimize occupational radiaiton exposure resulting from maintenance and calibration, consitent with 10 CFR 20.1101.	Rev. 6	Aug-16	ML16020A095
7.8	Diverse Instrumentation and Control Systems	DEIA	ICE	DSRA, DCIP	SRSB, HOIB	SRSB reviews portions related to the protection system. HOIB - As required in NUREG-0711, "Important Human Actions," Human Factors Engineering reviews the manual actions associated with the Diverse Actuation System. Historically, this review has been documented in Section 7.8.				Rev. 6	Aug-16	ML16020A096
7.9	Data Communication Systems	DEIA	ICE							Rev. 6	Aug-16	ML16020A097
Appendix 7-A	General Agenda, Station Site visits	DEIA	ICE							Rev. 5	Mar-07	ML070660412
Appendix 7-B	Acronyms, Abbreviations, and Glossary	DEIA	ICE							Rev. 4	Jun-97	ML070660430
BTP 7-1	Guidance on Isolation of Low-Pressure Systems from the High-Pressure Reactor Coolant System	DEIA	ICE							Rev. 6	Aug-16	ML16019A127

BTP 7-2	Guidance on Requirements of Motor-Operated Valves in the Emergency Core Cooling System Accumulator Lines	DEIA	ICE							Rev. 6	Aug-16	ML16019A299
BTP 7-3	Guidance on Protection System Trip Point Changes for Operation with Reactor Coolant Pumps out of Service	DEIA	ICE							Rev. 6	Aug-16	ML16019A358
BTP 7-4	Guidance on Design Criteria for Auxiliary Feedwater Systems	DEIA	ICE							Rev. 6	Aug-16	ML16020A028
BTP 7-5	Guidance on Spurious Withdrawals of Single Control Rods in Pressurized Water Reactors	DEIA	ICE							Rev. 6	Aug-16	ML16020A033
BTP 7-6	Guidance on Design of Instrumentation and Controls Provided to Accomplish Changeover from Injection to Recirculation Mode	DEIA	ICE							Rev. 6	Aug-16	ML16011A106
BTP 7-8	Guidance for Application of Regulatory Guide 1.22	DEIA	ICE							Rev. 6	Aug-16	ML16020A044
BTP 7-9	Guidance on Requirements for Reactor Protection System Anticipatory Trips	DEIA	ICE							Rev. 6	Aug-16	ML16011A062

BTP 7-10	Guidance on Application of Regulatory Guide 1.97	DEIA	ICE				DSEA	RPAC	Radiation Protection reviews this section to identify the instrumentation and contros associated with radiation monitoring equipment provided for post accident conditions are appropriately identified and characterized consistent with 10 CFR 50.34 and NUREG-0737. RPAC is end user on the performance, design, qualification, display, QA, and selection of monitoring variables of radiation monitoring equipment required for accident monitoring and sampling in SRP 11.2	Rev. 6	Aug-16	ML16019A169
BTP 7-11	Guidance on Application and Qualification of Isolation Devices	DEIA	ICE							Rev. 6	Aug-16	ML16019A184
BTP 7-12	Guidance on Establishing and Maintaining Instrument Setpoints	DEIA	ICE				DSEA	RPAC	RPAC is end user for any portion of the LWMS post-accident subsystems that supports safety-related functions in SRP 11.2	Rev. 6	Aug-16	ML16019A200
BTP 7-13	Guidance on Cross-Calibration of Protection System Resistance Temperature Detectors	DEIA	ICE							Rev. 6	Aug-16	ML16019A240
BTP 7-14	Guidance on Software Reviews for Digital Computer-Based Instrumentaiton and Control Systems	DEIA	ICE							Rev. 6	Aug-16	ML16019A308
BTP 7-17	Guidance on Self-Test and Surveillance Test Provisions	DEIA	ICE							Rev. 6	Aug-16	ML16019A316
BTP 7-18	Guidance on the Use of Programmable Logic Controllers in Digital Computer-Based Instrumentation and Control Systems	DEIA	ICE							Rev. 6	Aug-16	ML16019A327

BTP 7-19	Guidance for Evaluation of Diversity and Defense-in-Depth and Diversity Computer-Based Instrumentation and Control Systems	DEIA	ICE	DSRA, DCIP	SRSB, HOIB	SRSB reviews portions related to the protection system. HOIB - BTP specifically calls for Human Factors Engineering as the secondary reviewer.				Rev. 7	Aug-16	ML16019A344
BTP 7-21	Guidance on Digital Computer Real-Time Performance	DEIA	ICE							Rev 6	Aug-16	ML16020A036
8.1	Electric Power / Introduction	NRR/DE	EEEB							Draft Rev 5	May-14	ML113640121.
8.2	Offsite Power System	NRR/DE	EEEB				DSEA	RMOT	RMOT provides input to the severe environmental conditions (including salt deposition from cooling tower plumes) used in this SRP section	Rev. 5	May-10	ML100740246.
8.3.1	A C Power Systems (Onsite)	NRR/DE	EEEB							Rev. 4	May-10	ML100740289
8.3.2	D C Power Systems (Onsite)	NRR/DE	EEEB							Rev. 4	May-10	ML100740391
8.4	Station Blackout	NRR/DE	EEEB							Rev. 1	May-10	ML100740424
Appendix 8-A	General Agenda, Station Site Visits	NRR/DE	EEEB							Rev. 1	Mar-07	ML063600411.
BTP 8-1	Requirements on Motor-operated Valves in the ECCS Accumulator Lines	NRR/DE	EEEB							Rev. 3	Mar-07	ML070710423
BTP 8-2	Use of Diesel-Generator Sets for Peaking	NRR/DE	EEEB							Rev. 3	Mar-07	ML070710434
BTP 8-3	Stability of Offsite Power Systems,	NRR/DE	EEEB							Rev. 3	Mar-07	ML070710446
BTP 8-4	Application of the Single Failure Criterion to Manually Controlled Electrically Operated Valves	NRR/DE	EEEB							Rev. 3	Mar-07	ML070710452
BTP 8-5	Supplemental Guidance for Bypass and Inoperable Status Indication for Engineered Safety Features Systems	NRR/DE	EEEB							Rev. 3	Mar-07	ML070710466
BTP 8-6	Adequacy of Station Electric Distribution System Voltages	NRR/DE	EEEB							Rev. 3	Mar-07	ML070710478

BTP 8-7	Criteria for Alarms and Indications Associated with Diesel-Generator Unit Bypassed and Inoperable Status	NRR/DE	EEEB							Rev. 3	Mar-07	ML070710497
BTP 8-8	Onsite (Emergency Diesel Generators) and Offsite Power Sources Allowed Outage Time Extensions.	NRR/DE	EEEB							Rev. 0	Feb-12	ML113640138
BTP 8-9	Open Phase Conditions In Electric Power System	NRR/DE	EEEB							Rev. 0	Jul-15	ML15057A085
9.1.1	Criticality Safety of Fresh and Spent Fuel Storage and Handling	DSRA	SRSB	DEIA	MCB	MCB provides support regarding neutron absorber selection, monitoring, etc.	DSEA	RPAC	Radiation Protection reviews the kinds and quantities of radioactive material, and the means for controlling and limiting radiation exposures, consistent with 52.47(a)(5), 10 CFR 52.79(a)(3) and 10 CFR 52.157(e). In addition, RP reviews the type and location of radiation monitoring equipment provided to satisfy 10 CFR 70.24 or 10 CFR 50.68(b)(6)	Rev. 3	Mar-07	ML070570006
9.1.2	New and Spent Fuel Storage	DSRA	SPSB	DEIA	MCB, SEB,	The scope inclues the assessment of structures which is addressed by **SEB**. **MCB** provides support regarding neutron absorber selection, monitoring, etc.	DSEA	RPAC	Radiation Protection reviews the kinds and quantities of radioactive material, and the means for controlling and limiting radiation exposures, consistent with 52.47(a)(5), 10 CFR 52.79(a)(3) and 10 CFR 52.157(e). In addition, RP reviews the type and location of radiation monitoring equipment provided to satisfy 10 CFR 70.24 or 10 CFR 50.68(b)(6)	Rev. 4	Mar-07	ML070550057

9.1.3	Spent Fuel Pool Cooling and Cleanup System	DSRA	SPSB	DEIA	MCB	MCB provides support regarding water chemisty/chemical engineering aspects	DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive fission, activation and corrosion products contained in the tanks, heat exchangers, filters, demineralizers, pool water and related components which interface with the RCS, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5). 10 CFR 20.1406, as it relates to minimizing contamination of the facility and the environment. RPAC is end user of SFP cooling and cleanup system information in SRP 11.2 and 11.3.	Rev. 2	Mar-07	ML063190013
9.1.4	Light Load Handling System and Related Cavity Design	DSRA	SPSB				DSEA	RPAC	Radiation Protection reviews the means for controlling and limiting radiation exposures from spent fuel and irradiated components, consistent with 10 CFR 20.	Rev. 4	Mar-07	ML13085A145
9.1.5	Overhead Heavy Load Handling Systems	DSRA	SPSB	DEIA	SEB	SEB uses the information on heavy load drops in the design and analysis of structures and supports with information on the impact on structures				Rev. 5	Mar-07	ML070550056
9.2.1	Station Service Water System	DSRA	SPSB							Rev. 4	Mar-07	ML070550053
9.2.2	Reactor Auxiliary Cooling Water Systems			DEIA	MCB	MCB provides support regarding water chemisty/chemical engineering aspects	DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive fission, activation and corrosion products contained in the tanks, heat exchangers, and related components which interface or serve as boundaries with the RCS, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5). 10 CFR 20.1406, as it relates to minimizing contamination of the facility and the environment.	Rev. 4	Mar-07	ML070550053

9.2.4	Potable and Sanitary Water Systems	DSRA	SPSB	DSEA	RPAC	10 CFR 20 provides requirements for the control and monitoring of potential exposure from ingestion of radioactive material. 10 CFR 20.1406 describes the requirements for minimizing contamination. RG 4.21, "Minimization of Contamination and Radioactive Waste Generation: Life-Cycle Planning," and IE Bulletin 80-10, "Contamination of Nonradioactive System and Resulting Potential for Unmonitored, Uncontrolled Release of Radioactivity to Environment," provide additional guidance on how to minimize contamination of systems and actions to be taken in the event of system contamination. Radiation Protection personnel have the most knowledge about the industry events that have resulted in inadvertent internal exposures to site personnel from these systems, or inadvertent contamination of these systems.				Rev. 3	Mar-07	ML070550051
9.2.5	Ultimate Heat Sink	DSRA	SPSB				DSEA	RMOT	RMOT provides input to the adverse environmental conditions used in this SRP section	Rev. 3	Mar-07	ML070550048
9.2.6	Condensate Storage Facilities	DSRA	SPSB				DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive fission, activation and corrosion products contained in the tanks, filters and demineralizers, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5). 10 CFR 20.1406, as it relates to minimizing contamination of the facility and the environment. 10 CFR 20.1301(e) as it relates to limiting direct radiation exposure to members of the public. RPAC is end user of condensate storage facilities information in SRP 11.2 and 11.3.	Rev. 3	Mar-07	ML070550046
9.2.7	Chilled Water	DSRA	SPSB							Rev 0	Sep-15	ML15103A559

9.3.1	Compressed Air System	DSRA	SPSB	DSEA	RPAC	For a number of designs, portions of the station air or instrument air, or other compressed gas systems are utilized to provide air for breathing. 10 CFR 20 provides requirements for the control and monitoring of potential exposure from ingestion of radioactive material. RG 8.15 "Acceptable Programs for Respiratory Protection," provides guidance on acceptable methods for maintaining breathable air quality. 10 CFR 20.1406 describes the requirements for minimizing contamination. RG 4.21, "Minimization of Contamination and Radioactive Waste Generation: Life-Cycle Planning," and IE Bulletin 80-10, "Contamination of Nonradioactive System and Resulting Potential for Unmonitored, Uncontrolled Release of Radioactivity to Environment," provide additional guidance on how to minimize contamination of systems and actions to be taken in the event of system contamination. Radiation Protection personnel have the most knowledge about the industry events that have resulted in inadvertent internal exposures to site personnel from these systems, or inadvertent contamination of these systems.				Rev 0	Mar-07	ML070550044
9.3.2	Process and Post Accident Sampling Systems	DEIA	MCB	Multiple as defined in the SRP, DSEA	RPAC					Rev. 2	Mar-07	ML070680137
9.3.3	Equipment and Floor Drainage System	DSRA	SPSB				DSEA	RPAC	products contained within the system, consistent with 10 CFR 20.1204 and 10 CFR 20.1406, as they to limiting internal exposure to radionuclides, and minimizing contamination of the facility and the environment. RPAC is end user of equipment and floor drainage system information in SRP 11.2 and 11.3.	Rev. 3	Mar-07	ML070550042

9.3.4	Chemical and Volume Control System (PWR) (Including Boron Recovery System)	DSRA	SRSB	DEIA	MCB	MCB provides support regarding water chemisty/chemical engineering aspects	DSEA	RPAC	Radiation Protection reviews the kinds and quantities of radioactive material, and the means for controlling and limiting radiation exposures, consistent with 52.47(a)(5), 10 CFR 52.79(a)(3) and 10 CFR 52.157(e). RPAC is end user of equipment and floor drainage system information in SRP 11.2 and 11.3.	Rev. 3	Mar-07	ML070160660
9.3.5	Standby Liquid Control System (BWR)	DSRA	SRSB							Rev. 3	Mar-07	ML070680186
9.4.1	Control Room Area Ventilation System	DSRA	SCVB	DSEA, DCIP	RPAC, RMOT, HOIB	RPAC as the organization responsible for review of design basis accident radiological consequence analyses is confirmed to have secondary review responsibility. RPAC is not a secondary review branch for the SER. The results of the review under this SRP section are used as inputs to the SRP 15.0.3 review of DBA dose analyses and in the control room habitability review under SRP 6.4 where RPAC is a secondary review branch. RMOT provides the historic ambient outdoor air temperatures used in this SRP section				Rev. 3	Mar-07	ML070550045
9.4.2	Spent Fuel Pool Area Ventilation System	DSRA	SCVB				DSEA	RPAC	Information on this system listed in the related section of the DCD or FSAR may be used as input to the DBA dose analyses which is reviewed under SRP 15.0.3. Radiation Protection reviews the kinds and quantities of radioactive material, and the means for controlling and limiting radiation exposures, consistent with 52.47(a)(5), 10 CFR 52.79(a)(3) and 10 CFR 52.157(e).	Rev. 3	Mar-07	ML070550038

9.4.3	Auxiliary and Radwaste Area Ventilation System	DSRA	SCVB				DSEA	RPAC	Information on this system listed in the related section of the DCD or FSAR may be used as input to the DBA dose analyses which is reviewed under SRP 15.0.3. Radiation Protection reviews the kinds and quantities of radioactive material, and the means for controlling and limiting radiation exposures, consistent with 52.47(a)(5), 10 CFR 52.79(a)(3) and 10 CFR 52.157(e).	Rev. 3	Mar-07	ML070550039
9.4.4	Turbine Area Ventilation System	DSRA	SCVB				DSEA	RPAC	Information on this system listed in the related section of the DCD or FSAR may be used as input to the DBA dose analyses which is reviewed under SRP 15.0.3. Information in the related section of the DCD or FSAR is used to evaluate the effectiveness of radiation protection shielding, under SRP sections 12.2 and 12.3. Information in the related section of the DCD or FSAR is used to evaluate the effectiveness of radiological effluents under SRP Chapter 11.	Rev. 3	Mar-07	ML070550040
9.4.5	Engineered Safety Feature Ventilation System	DSRA	SCVB	DSEA	RPAC	RPAC as the organization responsible for review of design basis accident radiological consequence analyses is confirmed to have secondary review responsibility. RPAC is not a secondary review branch for the SER. The results of the review under this SRP section are used as inputs to the Chapter 15 review of DBA dose analyses when fission product removal by this system is credited in the dose analysis.	DSEA	RPAC	Information in the related section of the DCD or FSAR is used to evaluate the effectiveness of radiation protection shielding, under SRP sections 12.2 and 12.3.	Rev. 3	Mar-07	ML070550041

9.5.1.1	Fire Protection Program	DSRA	SPSB	DSEA	RPAC	GDC 3 requires fire protection design features to minimize the probability and effect of fires. 10 CFR Part 50.48 also includes fire protection SSCs needed to minimize the release of radioactive material to the environment - i.e., equipment important to safety. Fire protection design features include those provided to; (1) prevent fires in contaminated charcoal adsorption media, HEPA filters used in ventilation systems; (2) detect, control and minimize the generation of explosive H2 and O2 gas mixtures in systems and where high activity radioactive wastes are stored, and (3) act as barriers to prevent the release of radioactive materials. These design features help ensure that any releases of radioactive material due to a fire will remain within the effluent limits of 10 CFR 20.				Rev 0	Feb-09	ML090510170
9.5.1.2	Risk Informed (RI) and Performance Based (PB) Fire Protection Program (FPP)	DSRA	SPRA	DSEA	RPAC		DSEA	RPAC	RPAC: GDC 3 requires fire protection design features to minimize the probability and effect of fires.	Rev 0	Dec-09	ML092590527
9.5.2	Communications Systems	DEIA	ICE				DSEA	RPAC	Information in the related section of the DCD or FSAR is used to evaluate the effectiveness of design features provided to facilitate work, minimize radiation exposure, and implement Technical Specifications requirements specified for implementing high radiation area access controls per 10 CFR 20.1601(b) and 10 CFR 20.1601(c). 10 CFR 20.1101(b).	Rev. 3	7-Mar	ML070550037
9.5.3	Lighting Systems	DEIA	ICE				DSEA	RPAC	Information in the related section of the DCD or FSAR is used to evaluate the effectiveness of design features provided to minimize radiation exposure per 10 CFR 20.1101(b) and Regulatory Guide 8.8.	Rev. 3	Mar-07	ML070550036
9.5.4	Emergency Diesel Engine Fuel Oil Storage and Transfer System	DSRA	SPSB	DEIA	MCB	MCB provides support regarding fuel oil chemisty, monitoring, etc. aspects				Rev. 3	Mar-07	ML070680388

9.5.5	Emergency Diesel Engine Cooling Water System	DSRA	SPSB							Rev. 3	Mar-07	ML070550035
9.5.6	Emergency Diesel Engine Starting System	DSRA	SPSB							Rev. 3	Mar-07	ML070550034
9.5.7	Emergency Diesel Engine Lubrication System	DSRA	SPSB							Rev. 3	Mar-07	ML070460354
9.5.8	Emergency Diesel Engine Combustion Air Intake and Exhaust System	DSRA	SPSB							Rev. 3	Mar-07	ML070460354
10.2	Turbine Generator (Obsolete??)	DSRA	SPSB							Rev. 2	Mar-07	ML070380204
10.2.3	Turbine Rotor Integrity	DEIA	MCB							Rev. 3	Mar-07	ML063190015
10.3	Main Steam Supply System	DSRA	SPSB							Rev. 3	Mar-07	ML070380206
10.3.6	Steam and Feedwater System Materials	DEIA	MCB							Rev. 3	Mar-07	ML063190016
10.4.1	Main Condensers	DSRA	SPSB				DSEA	RPAC	RPAC is end user of main condensers information in SRP 11.2.	Rev. 3	Mar-07	ML070510077
10.4.2	Main Condenser Evacuation System	DSRA	SPSB							Rev. 3	Mar-07	ML070510078
10.4.3	Turbine Gland Sealing System	DSRA	SPSB				DSEA	RPAC	RPAC is end user of turbine gland sealing system information in SRP 11.2.	Rev. 3	Mar-07	ML070510079
10.4.4	Turbine Bypass System	DSRA	SPSB							Rev. 3	Mar-07	ML070380209
10.4.5	Circulating Water System	DSRA	SPSB							Rev. 3	Mar-07	ML070380215

10.4.6	Condensate Cleanup System	DEIA	MCB				DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive fission, activation and corrosion products contained in the tanks, filters and demineralizers, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5). 10 CFR 20.1406, as it relates to minimizing contamination of the facility and the environment. 10 CFR 20.1301(e) as it relates to limiting direct radiation exposure to members of the public. RPAC is end user of condensate cleanup system information to treat regenerant solutions in SRP 11.2.	Rev. 4	Mar-07	ML070710340
10.4.7	Condensate and Feedwater System	DSRA	SPSB							Rev. 3	Mar-07	ML070380221
10.4.8	Steam Generator Blowdown System (PWR)	DEIA	MCB	DE	ICE	ICE is not the lead but the scope of the review includes I&C systems	DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive fission, activation and corrosion products contained in the tanks, filters and demineralizers, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5). 10 CFR 20.1406, as it relates to minimizing contamination of the facility and the environment. 10 CFR 20.1301(e) as it relates to limiting direct radiation exposure to members of the public. RPAC is end user of SGBS (PWR) information in SRP 11.2	Rev. 3	Mar-07	ML070550004
10.4.9	Auxiliary Feedwater System (PWR)	DSRA	SPSB							Rev. 3	Mar-07	ML070570007

BTP 10-1	Design Guidelines for Auxiliary Feedwater System Pump Drive and Power Supply Diversity for Pressurized Water Reactor Plants	DSRA	SPSB							Rev. 4	Mar-07	ML07850410
BTP 10-2	Design Guidelines for Avoiding Water Hammers in Steam Generators	DSRA	SPSB							Rev.4	Mar-15	ML070850324
11.1	Coolant Source Terms	DSEA	RPAC				DSEA	RPAC, RENV	Coolant activity concentations reviewed under this section are used in DBA dose analyses reviewed under SRP 15.0.3. Radiation Protection reviews this section to identify the kinds and quantities of radioactive material, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5), 10 CFR 20 Subpart F, 10 CFR 20.1602 and 10 CFR 20.1101, 10 CFR 20.1301(e) and 10 CFR 20.1406. RPAC applies the same information to ESRP Sections 4.5 and 5.4. This ensures consistency between the SER and the EIS.	Rev. 4	Jan-16	ML15029A022
11.2	Liquid Waste Management Systems	DSEA	RPAC	DSRA	SPSB	SPSB is responsible for reviewing the systems portion while RPAC looks at the waste portion	DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive material, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5), 10 CFR 20 Subpart F, 10 CFR 20.1602 and 10 CFR 20.1101, 10 CFR 20.1301(e) and 10 CFR 20.1406. RPAC applies the same information to ESRP Sections 4.5 and 5.4. This ensures consistency between the SER and the EIS.	Rev. 5	Jan-16	ML15029A032

11.3	Gaseous Waste Management Systems	DSEA	RPAC	DSRA, DSEA	SPSB	SPSB is responsible for reviewing the systems portion while RPAC looks at the waste portion.	DSEA	RPAC	RMOT provides the atmospheric dispersion factors used in this section.	Rev. 4	Jan-16	ML15029A039
11.4	Solid Waste Management Systems	DSEA	RPAC	DSRA	SPSB	SPSB is responsible for reviewing the systems portion while RPAC looks at the waste portion	DSEA	RPAC, RENV	Radiation Protection reviews this section to identify the kinds and quantities of radioactive material, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5), 10 CFR 20 Subpart F, 10 CFR 20.1602 and 10 CFR 20.1101, 10 CFR 20.1301(e) and 10 CFR 20.1406. RPAC applies the same information to ESRP Sections 3.5, 4.5, and 5.4. 5.5.2. RENV applies the information to ESRP Section 5.5.2. This ensures consistency between the SER and the EIS	Rev. 4	Jan-16	ML15029A174
11.5	Process and Effluent Radiological Monitoring Instrumentation and Sampling Systems	DSEA	RPAC	DEIA, DSEA	ICE,	ICE: the scope of the review includes I&C systems	DSEA	RMOT	RMOT - This SRP section includes inspecting elements of the ODCM which includes atmospheric dispersion modeling.	Rev. 6	Jan-16	ML15029A182
BTP 11-3	Design Guidance for Solid Radioactive Waste Management Systems Installed in Light-Water -Cooled Nuclear Power Reactor Plants	DSEA	RPAC			Delete this item. The BTP is already part of SRP 11.4	DSEA	RPAC	Radiation Protection reviews this section to identify the kinds and quantities of radioactive material, and the means for controlling and limiting radiation exposures, in accordance with 10 CFR 52.47(a)(5), 10 CFR 20 Subpart F, 10 CFR 20.1602 and 10 CFR 20.1101, 10 CFR 20.1301(e) and 10 CFR 20.1406. RPAC applies the same information to ESRP Section 3.5. This ensures consistency between the SER and the EIS	Rev. 4	Jan-16	ML15027A198
BTP 11-5	Postulated Radioactive Releases Due to a Waste Gas System Leak or Failure	DSEA	RPAC							Rev 4	Jan-16	ML15027A302

BTP 11-6	Postulated Radioactive Releases Due to Liquid-Containing Tank Failures	DSEA	RPAC							Rev. 4	Jan-16	ML15027A401
12.1	Assuring that Occupational Radiation Exposures Are As Low As Is Reasonably Achievable	DSEA	RPAC				DSEA	RPAC	RPAC applies the same information to ESRP Sections 4.5. This ensures consistency between the SER and the EIS.	Rev. 4	Sep-13	ML13151A061
12.2	Radiation Sources	DSEA	RPAC				DSEA	RPAC	Information from this SRP section is used to estimate the direct dose contribution for the control room habitability analysis, which is reviewed under SRP 6.4 and 15.0.3. RPAC applies the same information to ESRP Sections 4.5. This ensures consistency between the SER and the EIS.	Rev. 5	Sep-13	ML13151A413
12.3 - 12.4	Radiation Protection Design Features	DSEA	RPAC				DSEA	RPAC	Information from this SRP section is used to estimate the direct dose contribution for the control room habitability analysis, which is reviewed under SRP 6.4 and 15.0.3	Rev. 5	Sep-13	ML13151A475
12.5	Operational Radiation Protection Program	DSEA	RPAC							Rev. 5	Aug-14	ML13155A232
13.1.1	Management and Technical Support Organization	DCIP	HOIB	DRA	APHB		DSEA	RPAC	Radiation Protection reviews this section to identify the management and technical support organizations provided for radiation protection and the levels of organizational freedom provided, consistent with the guidance of RG 8.8 and 8.10.	Draft 6	Aug-16	ML15005A449
13.1.2 - 13.1.3	Operating Organization	DCIP	HOIB	DSEA, DRA	RPAC, APHB					Rev. 7	Aug-16	ML15007A296
13.2.1	Reactor Operator Requalification Program; Reactor Operator Training	DCIP	HOIB	DIRS	IOLB					Rev. 4	Aug-16	ML15006A035
13.2.2	Non- Licensed Plant Staff Training	DCIP	HOIB	DSEA, DIRS	RPAC, IOLB					Rev. 4	Aug-16	ML15006A129

13.3	Emergency Planning	NSIR-DPR	LIB under EP	DSEA, DCIP	RMOT, HOIB	RMOT reviews the meteorlogical monitoring system that is described in this SRP section. HOIB for human factors at EOF facilities.	DSEA	RPAC	The technical support center habitabilty is reviewed as part of the review of DBA dose analyses under SRP 15.0.3. RPAC is end user for any portion of the post-accident subsystems that supports safety-related functions in SRP 11.2, 11.3, andf 11.5.	Rev. 3	Mar-07	ML063410307
13.4	Operational Program	DNRL	LB1, LB2, LB3, LB4				DSEA	RPAC	RPAC is end user of operational program information in SRP 11.2, 11.3.			ML070470463
13.5.1.1	Adminsitrative Procedures - General	DCIP	HOIB	Multi as defined by SRP, DSEA, DIRS	RPAC, IOLB	10 CFR 50.40(a) "Standard for Licenses, Certifications, and Regulatory Approvals," specifically identifies compliance with the regulations in 10 CFR 20.				Rev. 0	Aug-14	ML13115A067
13.5.1.2	Administrative Procedures - Initial Test Program	deleted	deleted							Draft Rev 0	Apr-96	ML052070642
13.5.2.1	Operating and Emergency Operating Procedures	DCIP	HOIB	DIRS	IOLB					Draft Rev. 3	Aug-14	ML13311B514
13.5.2.2	Maintenance and Other Operating Procedures	deleted	deleted							Draft Rev. 0	Jun-96	ML052070648
13.6.	Physical Security	NSIR-DSP	RSB							Rev. 1	Oct-10	ML102230082.
13.6.1	Physical Security - Combined License and Operating Reactors	NSIR-DSP	RSB							Rev. 0	Mar-07	ML070720094
13.6.2	Physical Security - Review of Physical Security Design - Standard Design Certification and Operating Reactor Licensing Applications	NSIR-DSP	RSB							Rev 2	Jun-15	ML14140A210
13.6.3	Physical Security - Early Site Permit and Reactor Siting Criteria	NSIR-DSP	RSB							Rev 2	Oct-16	ML15061A471
13.6.4	Access Authorization - Operational Program	NSIR-DSP	RSB							Rev. 0	Oct-16	ML15226A009
13.6.6	Cyber Security Program	NSIR-DSP	RSB							Rev 0	Nov-10	ML102630477
13.7	Fitness for Duty - Introduction	NSIR-DSP	SPSB							rev 0	Oct-16	ML15111A091
13.7.1	Fitness for Duty - Operational	NSIR-DSP	SPSB							rev 0	Oct-16	ML15111A036
13.7.2	Fitness for Duty - Construction	NSIR-DSP	SPSB							rev 0	Oct-16	ML15111A034

14.2	Initial Plant Test Program - Design Certification and New License Applicants	DCIP	QVIB	MULTIPLE, DEIA	MULTIPLE, ICE	ICE is not the lead but the scope of the review includes I&C systems	DSEA	RPAC	The initial test program checks the operability of numerous systems used to demonstrate compliance with 10 CFR 20 effluent controls, GDC 14, GDC 30, GDC 44, GDC 60, GDC 61, GDC 63 GDC 64, 10 CFR 50 Appendix I, and 10 CFR 20.1406, to name a few.	Rev. 3	Mar-07	ML070550027
14.2.1	Generic Guidelines for Extended Power Uprate Testing Programs	DCIP	QVIB	~~DSRA~~	~~SRSB, SBPA/B, SFPT~~	The SRP Section is not Applicable to NRO. Power Uprates would be NRR				Rev. 0	Mar-06	ML062210398
14.3	Inspections, Tests, Analyses, and Acceptance Criteria	DNRL	LB1, LB2, LB3, LB4	MULTIPLE	MULTIPLE					Rev. 0	Mar-07	ML070660618
14.3.1	Reserved											ML070520589
14.3.2	Structural and Systems Engineering - Inspections, Tests, Analyses, and Acceptance Criteria	DEIA	SEB	MULTIPLE	MULTIPLE		DSEA	RPAC	The checks the installation of a number SSCs used to demonstrate compliance with 10 CFR 20 effluent controls, GDC 14, GDC 30, GDC 44, GDC 60, GDC 61, GDC 63 GDC 64, and 10 CFR 50 Appendix I, to name a few.	Rev. 0	Mar-07	ML070660522
14.3.3	Piping Systems and Components Inspections, Tests, Analyses, and Acceptance Criteria	DEIA	MEB							Rev. 0	Mar-07	ML070660622
14.3.4	Reactor Systems - Inspections, Tests, Analyses, and Acceptance Criteria	DSRA	SRSB							Rev. 0	Mar-07	ML070660623
14.3.5	Instrumentation and Controls - Inspections, Tests, Analyses, and Acceptance Criteria	DEIA	ICE				DSEA	RPAC	Radiation Protection and Radioactive Waste reviews this section to identify the instrumentation and controls provided to meet 10 CFR 52.47(a)(5), 10 CFR 50 Appendix I, 10 CFR 20 effluent release limits, GDC 60, GDC 61, GDC 63 and GDC 64.	Rev. 0	Mar-07	ML070660624
14.3.6	Electrical Systems - Inspections, Tests, Analyses, and Acceptance Criteria	NRR/DE	EEEB							Rev. 0	Mar-07	ML070660625

14.3.7	Plant Systems - Inspections, Tests, Analyses, and Acceptance Criteria	DSRA	SPSB	DSEA	RPAC		DSEA	RPAC	Radiation Protection and Radioactive Waste reviews this section to identify the instrumentation and controls provided to meet 10 CFR 52.47(a)(5), 10 CFR 50 Appendix I, 10 CFR 20 effluent release limits, GDC 60, GDC 61, GDC 63 and GDC 64.	Rev. 0	Mar-07	ML070550025.
14.3.8	Radiation Protection - Inspections, Tests, Analyses, and Acceptance Criteria	DSEA	RPAC				DSEA	RPAC	RPAC: Reviewers responsible for trhe review of the post accident and sampling systems	Rev. 0	Mar-07	ML070550022.
14.3.9	Human Factors Engineering - Inspections, Tests, Analyses, and Acceptance Criteria	DCIP	HOIB	DRA	APHB					Rev. 0	Mar-07	ML070550021.
14.3.10	Emergency Planning - Inspections, Tests, Analyses, and Acceptance Criteria	NSIR	NLRB							Rev. 0	Mar-07	ML070730206
14.3.11	Containment Systems - Inspections, Tests, Analyses, and Acceptance Criteria	DSRA	SCVB							Rev. 0	Mar-07	ML070550011
14.3.12	Physical Security Hardware - Inspections, Tests, Analyses, and Acceptance Criteria	NSIR	RSB							Draft Rev. 2	Jun-16	ML16032A050
15.0	Introduction—Transient and Accident Analyses	DSRA	SRSB	DCIP	HOIB	Manual actions, procedures	DSEA	RPAC	The classification of events evaluated under this SRP section is information that is required to assess whether the appropriate DBAs are evaluated for radiological consequences under the SRP 15.0.3 review. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML070710376

15.0.1	Radiological Consequence Analyses Using Alternate Source Terms	N/A for new reactors					DSEA	RPAC, RMOT	RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS. RMOT provides the atmospheric dispersion facots used in this SRP section.	Rev. 0	Jul-10	ML003734190
15.0.2	Review of Transient and Accident Analysis Methods	DSRA	SRSB				DSEA	RPAC	RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML070820123
15.0.3	Design Basis Accident Radiological Consequence Analyses for Advanced Light Water Reactors	DSEA	RPAC	DSEA, DEIA, as necessary, DSRA	RHM/RMOT, MCB, as necessary: SCVB, SRSB, SPRA	**RMOT** provides the atmospheric dispersion factors used in this SRP section. **MCB** provides support regarding water chemisty/chemical engineering aspects	Multiple, DSEA	Multiple, RPAC	Information in other SRP reviews are used as input to the DBA dose analyses reviewed under this section. SRP sections included are 2.0, 2.3.4, 4.2, 6.4, 6.5.1, 6.5.2, 6.5.3, 9.4.1, 9.4.2, 9.4.3, 9.4.4, 9.4.5, 11.1, 12.2, 13.3, 15.0, 15.1.5, 15.2.8, 15.3.3-15.3.4, 15.4.8, 15.4.9, 15.6.5, and 15.7.5. For ENVIRONMENTAL REVIEW, ESRP section 7.1 verifies that information on DBA releases in the environmental report is the same as used in FSAR Chapter 15, as reviewed under this SRP section.	Rev. 0	Mar-07	ML070230012
15.1.1 - 15.1.4	Decrease in Feedwater Temperature, Increase in Feedwater Flow, Increase in Steam Flow, and Inadvertent Opening of a Steam Generator Relief or Safety Valve	DSRA	SRSB				DSEA	RPAC	RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 2	Mar-07	ML070550005

15.1.5	Steam System Piping Failures Inside and Outside of Containment (PWR)	DSRA	SRSB	DSEA	RPAC	RPAC as the organization responsible for review of design basis accident radiological consequence analyses is confirmed to have secondary review responsibility. RPAC is not a secondary review branch for the SER. The results of the review under this SRP section are used as inputs to the SRP 15.0.3 review of DBA dose analyses. Information on the accident initiation and progression should be ensured to be consistent with the assumptions underlying the guidance in SRP 15.0.3 and the technical information in RG 1.183, as referenced in SRP 15.0.3.	DCIP, DSEA	HIOB, RPAC	The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML070550006
15.1.5.A	Radiological Consequences of Main Steam Line Failures Outside Containment of a PWR	DSEA *N/A for new reactors - use 15.0.3	RPAC	DSRA	SRSB		DSEA	RPAC	RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 2	Jul-81	ML052350118
15.2.1 - 15.2.5	Loss of External Load; Turbine Trip; Loss of Condenser Vacuum; Closure of Main Steam Isolation Valve (BWR); and Steam Pressure Regulator Failure (Closed)	DSRA	SRSB	DSEA	RPAC	RPAC did not claim this as a secondary review responsibility with respect to the SER. RPAC as the organization responsible for review of design basis accident radiological consequence analyses is confirmed to have secondary review responsibility with respect to the development and maintenance of the SRP. RPAC is not a secondary review branch for the SER. • In general these events have not resulted in radiological release and are not considered in the DBA dose analyses reviewed under SRP 15.0.3. However, the Information on event initiation and progression should be ensured to be consistent with the assumptions underlying the guidance in SRP 15.0.3 and the technical information in RG 1.183, as referenced in SRP 15.0.3.	DSEA	RPAC	RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 2	Mar-07	ML070300702

15.2.6	Loss of Nonemergency AC Power to the Station Auxiliaries	DSRA	SRSB				DCIP, DSEA	HOIB, RPAC	The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 2	Mar-07	ML070550008
15.2.7	Loss of Normal Feedwater Flow	DSRA	SRSB				DCIP, DSEA	HOIB, RPAC	The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 2	Mar-07	ML070300709
15.2.8	Feedwater System Pipe Breaks Inside and Outside Containment (PWR)	DSRA	SRSB				DCIP, DSEA	HOIB, RPAC	HOIB- The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC - Accident modeling, including estimates of fuel damage, steam and coolant mass release, thermal hydraulics, etc., evaluated under this SRP section are used to develop the DBA dose analyses reviewed under SRP 15.0.3. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 2	Mar-07	ML070550009

15.3.1 - 15.3.2	Loss of Forced Reactor Coolant Flow Including Trip of Pump Motor and Flow Controller Malfunctions	DSRA	SRSB				DCIP, DSEA	HOIB, RPAC	The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 2	Mar-07	ML070550010
15.3.3 - 15.3.4	Reactor Coolant Pump Rotor Seizure and Reactor Coolant Pump Shaft Break	DSRA	SRSB				DCIP, DSEA	HOIB, RPAC	HOIB - The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC - Accident modeling, including estimates of fuel damage, steam and coolant mass release, thermal hydraulics, etc., evaluated under this SRP section are used to develop the DBA dose analyses reviewed under SRP 15.0.3. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML070550012
15.4.1	Uncontrolled Control Rod Assembly Withdrawal from a Subcritical or Low Power Startup Condition	DSRA	SRSB				DCIP	HOIB, RPAC	The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML063600413

15.4.2	Uncontrolled Control Rod Assembly Withdrawal at Power	DSRA	SRSB				DCIP, DSEA	HOIB, RPAC	The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML063600414
15.4.3	Control Rod Misoperation (System Malfunction or Operator Error)	DSRA	SRSB				DCIP, DSEA	HOIB, RPAC	The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML063600415
15.4.4 - 15.4.5	Startup of an Inactive Loop or Recirculation Loop at an Incorrect Temperature, and Flow Controller Malfunction Causing an Increase in BWR Core Flow Rate	DSRA	SRSB				DCIP, DSEA	HOIB, RPAC	The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 2	Mar-07	ML070550007
15.4.6	Inadvertent Decrease in Boron Concentration in the Reactor Coolant System (PWR)	DSRA	SRSB				DCIP, DSEA	HOIB, RPAC	The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 2	Mar-07	ML070380222

15.4.7	Inadvertent Loading and Operation of a Fuel Assembly in an Improper Position	DSRA	SRSB				DSEA	RPAC	RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 2	Mar-07	ML070550013
15.4.8	Spectrum of Rod Ejection Accidents (PWR)	DSRA	SRSB				DCIP, DSEA	HOIB, RPAC	HOIB - The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC - Accident modeling, including estimates of fuel damage, steam and coolant mass release, thermal hydraulics, etc., evaluated under this SRP section are used to develop the DBA dose analyses reviewed under SRP 15.0.3. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML070550014
15.4.8. Appendix A	Radiological Consequences of a Control Rod Ejection Accident (PWR)	DSEA *N/A for new reactors - use 15.0.3	RPAC							Rev. 1	Jul-81	ML052350416
15.4.9	Spectrum of Rod Drop Accidents (BWR)	DSRA	SRSB	DSEA	RPAC	RPAC as the organization responsible for review of design basis accident radiological consequence analyses is confirmed to have secondary review responsibility. RPAC is not a secondary review branch for the SER. The results of the review under this SRP section are used as inputs to the SRP 15.0.3 review of DBA dose analyses. Information on the accident initiation and progression should be ensured to be consistent with the assumptions underlying the guidance in SRP 15.0.3 and the technical information in RG 1.183, as referenced in SRP 15.0.3.	DCIP, DSEA	HOIB, RPAC	The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML070550015

15.4.9. Appendix A	Radiological Consequences of Control Rod Drop Accident (BWR)	DSEA *N/A for new reactors - use 15.0.3	RPAC							Draft Rev 3	Apr-96	ML052070724
15.5.1 - 15.5.2	Inadvertent Operation of ECCS and Chemical and Volume Control System Malfunction that Increases Reactor Coolant Inventory	DSRA	SRSB				DCIP, DSEA	HOIB, RPAC	The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 2	Mar-07	ML070820081
15.6.1	Inadvertent Opening of a PWR Pressurizer Pressure Relief Valve or a BWR Pressure Relief Valve	DSRA	SRSB				DCIP, DSEA	HOIB, RPAC	The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 2	Mar-07	ML070520094
15.6.2	Radiological Consequences of the Failure of Small Lines Carrying Primary Coolant Outside Containment	DSEA *N/A for new reactors - use 15.0.3	SRSB							Rev. 2	Jul-81	ML052350147
15.6.3	Radiological Consequences of Steam Generator Tube Failure (PWR)	DSEA *N/A for new reactors - use 15.0.3	RPAC							Rev. 2	Jul-81	ML052350149
15.6.4	Radiological Consequences of Main Steam Line Failure Outside Containment (BWR)	DSEA *N/A for new reactors - use 15.0.3	RPAC				DCIP	HOIB, RPAC	The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance.	Rev. 2	Jul-81	ML052350151

15.6.5	Loss of Coolant Accidents Resulting From Spectrum of Postulated Piping Breaks Within the Reactor Coolant Pressure Boundary	DSRA	SRSB				DCIP, DSEA	HOIB, RPAC	HOIB - The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC - Accident modeling, including estimates of fuel damage, steam and coolant mass release, thermal hydraulics, etc., evaluated under this SRP section are used to develop the DBA dose analyses reviewed under SRP 15.0.3. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 3	Mar-07	ML070550016
15.6.5. Appendix A	Radiological Consequences of a Design Basis Loss-of-Coolant Accident Including Containment Leakage Contribution	DSEA *N/A for new reactors - use 15.0.3	RPAC	DEIA	MCB	MCB provides support regarding water chemisty/chemical engineering aspects				Rev. 1	Jul-81	ML052350158
15.6.5. Appendix B	Radiological Consequences of a Design Basis Loss-of-Coolant Accident Leakage From Engineered Safety Feature Components Outside Containment	DSEA *N/A for new reactors - use 15.0.3	RPAC							Rev. 1	Jul-81	ML052350160
15.6.5. Appendix D	Radiological Consequences of a Design Basis Loss-of-Coolant Accident: Leakage From Main Steam Isolation Valve Leakage Control System (BWR)	DSEA *N/A for new reactors - use 15.0.3	RPAC							Rev. 1	Jul-81	ML052350166

15.7.3	Postulated Radioactive Releases Due to Liquid-Containing Tank Failures	DSEA *N/A for new reactors - use 15.0.3	RPAC							Rev. 2	Jul-81	ML052350171
15.7.4	Radiological Consequences of Fuel Handling Accidents	DSEA *N/A for new reactors - use 15.0.3								Rev. 1	Jul-81	ML052350313
15.7.5	Spent Fuel Cask Drop Accidents	DSEA *N/A for new reactors - use 15.0.3	RPAC							Rev. 2	Jul-81	ML052350315
15.8	Anticipated Transients Without Scram						DCIP, DSEA	HOIB, RPAC	The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 2	Mar-07	ML070570008
15.9	Boiling Water Reactor Stability	DSRA	SRSB				DCIP, DSEA	HOIB, RPAC	The guidance requires the reviewer to assess credited operator actions/operator errors but does not identify HFE as a review interface/secondary review or cite relevant guidance. RPAC applies the same information to ESRP Sections 7.1. This ensures consistency between the SER and the EIS.	Rev. 0	Mar-07	ML070550017

16.0	Technical Specifications	NRR/DSS	STSB				DSEA	RPAC	Radiation Protection reviews this section to identify the Technical Specifications associated with radiation monitoring equipment described in Chapter 12, and Radiation Protection program elements, in accordance with 10 CFR 20 and 10 CFR 50.34, and leakage control program elements, consistent with 10 CFR50.34(f)(2)(xxvi) RPAC is end user of administrative programs on radioactive effluent controls and monitoring in SRP 11.2, 11.3, 11.4, and 11.5.	Rev. 3	Mar-10	ML100351425
16.1	Risk-Informed Decision Making: Technical Specifications	NRR/DSS	STSB	Multiple	SPRA, ICE	Tech Spec COE in NRR by FY'17. SPRA reviews the PRA technical adequacy for the risk-informed application and provides SE input. SRSB may provide SE input if risk-informed Tech Specs affect chapter 15 analysis	DSEA	RPAC	Radiation Protection reviews this section to identify the Technical Specifications associated with radiation monitoring equipment described in Chapter 12, and Radiation Protection program elements, in accordance with 10 CFR 20 and 10 CFR 50.34, and leakage control program elements, consistent with 10 CFR50.34(f)(2)(xxvi)	Rev. 1	Mar-07	ML070380228
17.1	Quality Assurance During Design and Construction Phases	DCIP	QVIB				DSEA	RPAC	RPAC is end user of QA program for any portion of the radiactive waste management system that may be covered by 10 CFR 50, Appendix B in SRP 11.2 and 11.3.	Rev. 2	Jul-81	ML052350349
17.2	Quality Assurance During the Operations Phase	DCIP	QVIB							Rev. 2	Jul-81	ML052350361
17.3	Quality Assurance Program Description	DCIP	QVIB							Rev. 0	Aug-90	ML052350376
17.4	Reliability Assurance Program	DSRA	SPRA							Rev. 1	May-10	ML13296A435

17.5	Program Description - Design Certification, Early Site Permit and New License Applicants	DCIP	QVIB							Rev. 1	Aug-15	ML15037A441
17.6	Maintenance Rule	DSRA	SPRA							Rev. 2	Jul-14	ML14099A044
18.0	Human Factors Engineering Introduction	DCIP	HOIB	DRA	APHB					Rev. 3	Dec-16	ML16125A114
18.0 Appendix A	Crediting Manual Operator Actions in Diversity and Defense-in-Depth (D3) Analyses	DCIP	HOIB	DRA	APHB					Rev. 0	Apr-14	ML13115A156
19.0	Probabilistic Risk Assessment and Severe Accident Evaluation for New Reactors	DSRA	SPRA	DEIA, DSEA	SEB, ICE	**ICE** is not the lead but the scope of the review includes I&C systems.The scope inclues the assessment of structures which is addressed by **SEB**.	DSEA	RPAC, RMOT	RMOT is responsible for the review of design-basis external climatic hazards and should have in put to the PRA of these hazards. RPAC applies the same information to ESRP Sections 7.2 and 7.3. This ensures consistency between the SER and the EIS.	Rev.3	Dec-15	ML15089A068
19.1	Determining the Technical Adequacy of Probabilistic Risk Results for Risk-Informed License Amendment Requests After Initial Fuel Load	DSRA	SPRA	Multiple	Multiple		DSEA	RPAC	RPAC applies the same information to ESRP Sections 7.2 and 7.3. This ensures consistency between the SER and the EIS.	Rev 3	Sep-12	ML12193A107
19.2	Review of Risk Information Used to Support Permanent Plant-Specific Changes to the Licensing Basis: General Guidance	DSRA	SPRA	MULTIPLE	MULTIPLE		DSEA	RPAC	RPAC applies the same information to ESRP Sections 7.2 and 7.3. This ensures consistency between the SER and the EIS.	Rev. 3	Jun-07	ML071700658
19.3	Regulatory Treatment of Non-Safety Systems (RTNSS) for Passive Advanced Light Water Reactors	DSRA	SPRA	DEIA, DSRA	MEB, SEB, SPRA,	**MEB**: branch contributes to the content of the SRP but is not the owner. MEB addresses pumps and valves that are in NSS for IST and mechanical qualification related topics. **SEB**: Structures are included in the scope and SEB provides support with the impact on structures.	DSEA	RMOT	RMOT can help verify the design basis wind speeds for RTNSS SSCs.	Rev. 0	Jun-14	ML14035A149

19.4	Strategies and Guidance to Address Loss-of-Large Areas of the Plant Due to Explosions and Fires	DSRA	SPSB	MULTIPLE	MULTIPLE					Rev. 0	Jun-15	ML13316B202.
19.5	Adequacy of Design features and functional capabilities identified and described for withstanding Aircraft Impacts	DSRA	SPSB	DEIA	SEB	Structures are included in the scope and SEB provides support with the impact on structures				Rev. 0	Apr-13	ML12276A112