



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

August 24, 2016

Mr. Peter P. Sena, III  
President  
PSEG Nuclear LLC - N09  
P.O. Box 236  
Hancocks Bridge, NJ 08038

SUBJECT: HOPE CREEK GENERATING STATION – REQUEST FOR ADDITIONAL  
INFORMATION REGARDING RELIEF REQUEST VR-02, ASSOCIATED WITH  
THE FOURTH 10-YEAR INSERVICE TEST INTERVAL (CAC NO. MF7201)

Dear Mr. Sena:

By letter dated December 18, 2015 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15352A127), as supplemented by letter dated June 10, 2016 (ADAMS Accession No. ML16162A709), PSEG Nuclear LLC (PSEG or the licensee) submitted Relief Requests GR-01, PR-01, PR-02, VR-01, and VR-02, for the fourth 10-year inservice testing interval for the Hope Creek Generating Station. These proposed alternatives apply to certain requirements of the American Society of Mechanical Engineers Code for the Operation and Maintenance of Nuclear Power Plants and were submitted pursuant to Title 10 of the *Code of Federal Regulations* Sections 50.55a(a)(z)(1) and 50.55a(a)(z)(2).

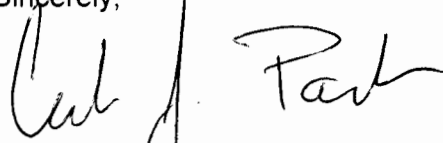
The U.S. Nuclear Regulatory Commission staff has reviewed the licensee's application and, based upon this review, determined that additional information is needed for Relief Request VR-02, "Main Steam Safety Relief Valves," as set forth in the enclosed Request for Additional Information. On August 3, 2016, a draft of these questions was sent to Mr. Paul Duke of your staff to ensure that the questions were understandable, the regulatory basis for the questions was clear, and to determine if the information was previously docketed. On August 18, 2016, a teleconference was held to clarify the questions. Mr. Duke indicated that PSEG will submit a response within 30 days of the date of this letter.

P. Sena

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If you have any questions, please contact me at (301) 415-1603 or by e-mail at [Carleen.Parker@nrc.gov](mailto:Carleen.Parker@nrc.gov).

Sincerely,

A handwritten signature in black ink, appearing to read "Carleen J. Parker". The signature is written in a cursive style with a vertical line separating the first and last names.

Carleen J. Parker, Project Manager  
Plant Licensing Branch I-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-354

Enclosure:  
Request for Additional Information

cc w/enclosure: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION  
REGARDING RELIEF REQUEST VR-02  
FOR THE INSERVICE TESTING PROGRAM FOURTH 10-YEAR INTERVAL  
PSEG NUCLEAR LLC  
HOPE CREEK GENERATING STATION  
DOCKET NO. 50-354

By letter dated December 18, 2015 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML15352A127), as supplemented by letter dated June 10, 2016 (ADAMS Accession No. ML16162A709), PSEG Nuclear LLC submitted Relief Requests GR-01, PR-01, PR-02, VR-01, and VR-02, for the fourth 10-year inservice testing interval for the Hope Creek Generating Station. These proposed alternatives apply to certain requirements of the American Society of Mechanical Engineers Code for the Operation and Maintenance of Nuclear Power Plants (ASME OM Code) and were submitted pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR) Sections 50.55a(a)(z)(1) and 50.55a(a)(z)(2). The U.S. Nuclear Regulatory Commission staff has reviewed the licensee's application and, based upon this review, determined that following additional information is needed for Relief Request VR-02:

**Relief Request VR-02:**

The regulations in 10 CFR 50.55a(f), "Inservice testing requirements," require, in part, that inservice testing of certain ASME Code Class 1, 2, and 3 components must meet the requirements of the ASME OM Code and applicable addenda, except where alternatives have been authorized pursuant to paragraphs 10 CFR 50.55a(z)(1) or 10 CFR 50.55a(z)(2).

1. How will the ASME OM Code sample test requirement of I-1320(a) (i.e., 20 percent per 24 months) be applied to the main steam pressure relief valves?
2. How will the 20 percent be determined?
3. Will the two stage and three stage Target Rock valves be in separate groups (a group being defined in accordance with I-1200)?
4. How will the change in the number of three stage versus two stage Target Rock valves affect the grouping(s)?

P. Sena

- 2 -

If you have any questions, please contact me at (301) 415-1603 or by e-mail at [Carleen.Parker@nrc.gov](mailto:Carleen.Parker@nrc.gov).

Sincerely,

**/RA/**

Carleen J. Parker, Project Manager  
Plant Licensing Branch I-2  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-354

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**ADAMS Accession No.: ML16231A427**

*\*by e-mail*

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