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August 19, 2016

Dr. Jae-Yong Lee
Director, APR1400 Design Certification
Korea Hydro and Nuclear Power Co.
70-1312-gil, Yuseong-daero
Yuseong-Gu, Daejeon
305-343, Korea

SUBJECT: PROPRIETARY AND FACTUAL REVIEW OF THE ADVANCED POWER REACTOR 1400 ADVANCED TOPICAL REPORT SAFETY EVALUATION, "KCE-1 CRITICAL HEAT FLUX CORRELATION FOR PLUS7 THERMAL DESIGN"

Dear Dr. Lee:

The U.S. Nuclear Regulatory Commission (NRC) staff has prepared an advanced Topical Report Safety Evaluation (TRSE) for APR1400-F-C-TR-12002-P, Revision 0, "KCE-1 Critical Heat Flux Correlation for PLUS7 Thermal Design." This is in support of the review of the Advanced Power Reactor (APR) 1400 design certification application submitted by Korea Hydro & Nuclear Power Co., LTD (KHNP) and Korea Electric Power Corporation (KEPCO) on December 23, 2014.

The enclosed advanced TRSE is being provided to KHNP for review of proprietary information pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR) 2.390 "Public Inspections, Exemptions, Requests for Withholding." The NRC will withhold the enclosed TRSE from public disclosure for a total of 20 business days. This includes 10 business days from the date of this letter to allow you the opportunity to verify the staff's TRSE for proprietary content or security-related information not previously identified and 10 business days for a check of any factual errors. To facilitate the NRC staff's review, please provide a marked up copy of the advanced TRSE identifying any proprietary information in addition to any factual errors or clarity concerns within 20 days from the date of this letter. If within that 20 day period, you do not request that all or portions of the TRSE be withheld from public disclosure, the enclosure will be made publically available through the NRC Public Document Room and the Publicly Available Records component of the NRC Agencywide Documents Access and Management System (ADAMS) and placed on the NRC public web page for this application.

Following your review, the staff's advanced TRSE for APR1400-F-C-TR-12002-P, "KCE-1 Critical Heat Flux Correlation for PLUS7 Thermal Design," will be provided to the Advisory Committee on Reactor Safeguards (ACRS) Subcommittee to support an upcoming meeting of the ACRS Subcommittee, scheduled for February 8, 2017.

Document transmitted herewith contains sensitive unclassified information. When separated from the enclosure, this document is "DECONTROLLED."
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Dr. Lee

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If you have any questions or comments concerning this matter, please contact me at (301) 415-6391 or via e-mail address at Jeff.Ciocco@nrc.gov.

Sincerely,

/RA/

Jeffrey Ciocco, Senior Project Manager
Licensing Branch 2
Division of New Reactor Licensing
Office of New Reactors

Docket No. 52-046

Enclosure:
As stated

cc w/o encl: See next page

Dr. Lee

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If you have any questions or comments concerning this matter, please contact me at (301) 415-6391 or via e-mail address at Jeff.Ciocco@nrc.gov.

Sincerely,

/RA/

Jeffrey Ciocco, Senior Project Manager
Licensing Branch 2
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