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Fax: 724-643-8069August 5, 2016  
L-16-254

10 CFR 50.55a

ATTN: Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555-0001**SUBJECT:**  
Beaver Valley Power Station, Unit No. 2  
Docket No. 50-412, License No. NPF-73  
Supplement to Request to Extend Certain Reactor Vessel Inspections  
From 10 to 20 Years (Request 2-TYP-3-BN-01)

In accordance with 10 CFR 50.55a(z), FirstEnergy Nuclear Operating Company (FENOC) requested to extend certain reactor vessel inspections by letter dated December 22, 2015 (ADAMS Accession Number ML15356A340). The affected reactor vessel interior attachments referenced in Request 2-TYP-3-BN-01 are located beyond the beltline region instead of within the beltline region. Therefore, this supplement provides corrections to the referenced ASME Code Item Number B13.50 and related description in Request 2-TYP-3-BN-01. The correct item number is listed in the inservice inspection 10-year plan examination schedule.

References to item number B13.50 are to be replaced with item number B13.60 in five places in Request 2-TYP-3-BN-01; once in sections 1 and 3, twice in the last paragraph of Section 5, and once in Section 6. The description of item number B13.50 in sections 1 and 3 is to be changed from "Within Beltline Region" and "within the beltline region" to read "Beyond Beltline Region" and "beyond the beltline region," respectively. The words "within the reactor pressure vessel beltline region" in the first paragraph of Section 4, and "within the reactor vessel beltline region" in the first paragraph of Section 5, are to be replaced with the words "beyond the reactor pressure vessel beltline region." These changes do not impact the reason, basis or proposed alternative for this request.

In addition, the following sentence is to be added to the end of the first paragraph of Request 2-TYP-3-BN-01, Section 5.

If there is an opportunity to perform examinations due to removal of the core support structure from the reactor vessel prior to the 2027 refueling outage, the code required VT-3 examinations will be performed on the core support

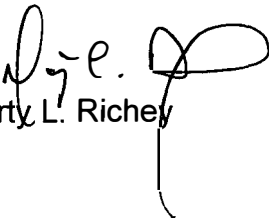
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structure and interior reactor vessel attachments in order to fulfill the exam requirements for the extended interval.

FENOC continues to request approval of the revised 10 CFR 50.55a request by December 31, 2016.

There are no regulatory commitments contained in this submittal. If there are any questions or if additional information is required, please contact Mr. Thomas A Lentz, Manager – Fleet Licensing at (330) 315-6810.

Sincerely,



Marty L. Richey

cc: NRC Region I Administrator  
NRC Resident Inspector  
NRR Project Manager  
Director BRP/DEP  
Site BRP/DEP Representative