Date: Wednesday, June 08, 2016 Time: 3:43:26 PM

#### **TEST REPORT**

Question file: ILT 10 NRC EXAM BANK

Copyright:

Test Date: 7/7/2016

Test Name: ILT-10 SRO NRC EXAM

Test Form: 0

Test Version: 0

Test Points: 100.00

Test File: ILT 10 SRO NRC EXAM

Test Path: O:\ILT 10\NRC EXAM\Written Exam\ILT 10 SRO NRC EXAM.LXRTest

Name:
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Form: 0 Version: 0

#### 1. 201001K6.06 001

Unit 2 was operating at 100% RTP when a loss of RBCCW occurs.

A manual scram is inserted which results in the following:

- o Reactor power is 6% RTP
- o None of the RBCCW pumps can be started

Based on the above conditions and IAW 34AB-P42-001-2, Loss of RBCCW,

The CRD pumps \_\_\_\_\_.

- A. are NOT allowed to operate because the CRD Pump Seal Coolers have lost their cooling medium
- B. are NOT allowed to operate because the CRD Pump Room Area Coolers have lost their cooling medium
- C. can be operated with a MAXIMUM allowable pumping medium temperature of 104°F

DY can be operated with a MAXIMUM allowable pumping medium temperature of 249°F

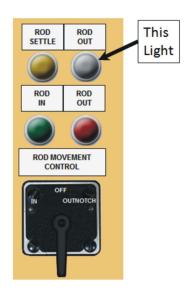
#### 2. 201002A2.04 001

**Unit 1** is operating at 15% RTP and is performing a reactor startup.

All control rods in the currently latched RWM step are at their Insert Limit of position 04.

- o The Withdraw Limit for the latched step is position 08
- o Control rod 30-31 is currently selected

When Control rod 30-31 is withdrawn, the rod DOUBLE NOTCHES to position 10.



Based on the above conditions,

When Control rod 30-31 reaches position 10, the Rod Movement Control light will be	"Rod Out" white
IAW 34GO-OPS-065-0, Control Rod Movement, the RWMBYPASSED to correct the Control rod 30-31 mis-position.	REQUIRED to be

A. ILLUMINATED;

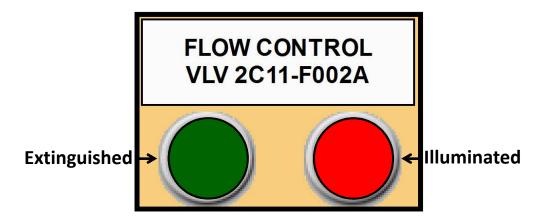
is

- B. ILLUMINATED; is NOT
- C. EXTINGUISHED; is
- DY EXTINGUISHED; is NOT

#### 3. 201003G2.1.30 001

Unit 2 is operating at 100% RTP with the 2C11-F002A, Flow Control Valve, in service.

Subsequently, Drywell pressure increases to 2.5 psig.



One (1) minute later and based on the above indications/conditions,

2C11-F002A \_\_\_\_\_\_ responding correctly.

2C11-F002A indicating LIGHTS are located on \_\_\_\_\_\_.

- A. is;
  - 2H11-P603 and locally at the 2C82-P001, Remote Shutdown Panel
- By is;

2H11-P603 Panel ONLY

C. is NOT;

2H11-P603 and locally at the 2C82-P001, Remote Shutdown Panel

D. is NOT;

2H11-P603 Panel ONLY

#### 4. 203000K5.01 001

Unit 2 has experienced a LOCA.

- o RHR Loop 2B is injecting in the LPCI mode to maintain RWL
- o A leak develops on 2E11-F015B, RHR Inboard Injection valve

Subsequently, all RHR Loop 2B pumps are secured.

Ba	sed on the above conditions,		
	The water leaking from 2E11-F015B will be leaking	ng into	Containment
	The water leaking from 2E11-F015B2E11-F050B, RHR Injection Check valve.	be isolated by opera	ation of
A.	Secondary; will NOT		
B <b>?</b>	Secondary; will		
C.	Primary; will NOT		
D.	Primary;		

will

## 5. 204000K4.07 001

The automatic closure setpoint for 2G31-FCV-F033, RWCU Blowdown Flow Control Valve, is met if \_\_\_\_\_\_ .

- A. RWCU pump flow decreases to 50 gpm
- B. RWCU Dump flow increases to 150 gpm
- C. 2G31-F033 upstream pressure decreases to 5 psig
- D. 2G31-F033 downstream pressure increases to 120 psig

#### 6. 205000K3.02 001

Unit 1 is in Mode 4 with both Recirc pumps off.

RHR Loop B is operating in Shutdown Cooling with a flowrate of 7300 gpm.

Subsequently, a tube rupture occurs in RHR Heat Exchanger 1B.

With the above conditions and NO operator actions,

RWL will start going \_\_\_\_\_\_\_.

IAW 34SO-E11-010-1, RHR System, the MINIMUM listed corrected RWL that must be maintained by the OATC is \_\_\_\_\_\_\_.

A. UP;

34 inches

BY UP;

54 inches

C. DOWN;

34 inches

54 inches

## 7. 205000K6.04 001

Unit 1 is in Mode 3 with RHR pump 1A in Shutdown Cooling (SDC).

Subsequently, RWL lowers to -10 inches and stabilizes.

Three (3) minutes later and based on the above conditions,

1E11-F015A, RHR Injection Valve, will be \_\_\_\_\_\_\_.

RHR pump 1A \_\_\_\_\_\_\_ be running.

A. closed;
will

BY closed;
will NOT

C. open;
will

D. open;
will NOT

#### 8. 206000G2.4.50 001

Unit 2 is operating at 100% RTP when the following occurs:

- o TORUS WATER LEVEL HIGH/LOW, 602-235, is ILLUMINATED
- o Torus water level

151.0 inches and stable

Based on the above conditions and one (1) minute later,

HPCI pump suction will be aligned to the \_\_\_\_\_\_.

RCIC pump suction will be aligned to the \_\_\_\_\_.

AY CST;

Torus

B. CST;

**CST** 

C. Torus;

Torus

D. Torus;

**CST** 

# 9. 209001A3.02 001

Unit 2 is operating at 100% RTP.

At 13:01, a Loss of Offsite Power occurs to BOTH Units.

At 13:10, RWL is -101 inches and slowly lowering.

Based on the above conditions,

The EARLIEST listed time that Core Spray pump 2A will be in operation is \_\_\_\_\_\_.

- A. 13:01
- B. twelve (12) seconds after 13:01
- CY 13:10
- D. twelve (12) seconds after 13:10

#### 10. 211000G2.2.36 001

**Unit 1** is operating at 100% RTP with the following:

o 1C41-C001B, SBLC pump 1B, is DANGER Tagged out of service for a pump leak

Based on the above status of SBLC pump 1B,

If a troubleshooting activity resulted in tripping the feeder breaker to \_\_\_\_\_\_, an additional LCO entry would be required for the SBLC System.

- AY 1R24-S011, Rx. Bldg. MCC 1C
- B. 1R24-S012, Rx. Bldg. MCC 1B
- C. 1R24-S013, Rx. Bldg. MCC 1A
- D. 1R24-S014, Rx. Bldg. MCC 1D

#### 11. 212000K1.03 001

Unit 2 is operating at 75% RTP.

The followin	g occurs at t	the listed time:
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- o 12:01 RPS MG Set 2A trips
- o 12:03 The Power Source Select switch (2H11-P610) is placed in the "ALT B" position

Based on the above conditions,

At 12:02, 2B31-R614, Recirc Drive Flow, recorder \_\_\_\_\_\_ indicate accurate flow.

At 12:05, Recirc Pump 2A Drive Flow will be \_\_\_\_\_ the Recirc Pump 2A Drive Flow at 12:00.

A. will NOT; less than

- B. will NOT; approximately the same as
- Cy will continue to; less than
- D. will continue to; approximately the same as

#### 12. 215002K1.06 001

Unit 2 is operating at 50% RTP.

A central control rod has just been selected for withdrawal.

Based on the above conditions,

When the central control rod was selected, the RBM indication immediately went \_\_\_\_\_\_.

After the RBM nulling sequence is complete, the RBM reading will be \_\_\_\_\_\_.

A. downscale;

50%

By downscale; 100%

C. to 100%; 50%

D. to 100%; 100%

# 13. 215003K2.01 001

Unit 2 is starting up with Reactor power at 5% RTP.

24/48 VDC Cabinet 2A, 2R25-S015, de-energizes and can NOT be restored.

Based on the above conditions,

IRM Channels \_\_\_\_\_ will have lost their power supply.

- A\* 2A & 2C
- B. 2A & 2D
- C. 2B & 2C
- D. 2B & 2D

#### 14. 215004K3.04 001

**Unit 2** is in a Refueling outage (NO fuel removed yet).

The following conditions exist:

o Reactor Mode Switch REFUEL o SRM Shorting Links REMOVED

A Control rod located adjacent to SRM 2A is fully WITHDRAWN resulting in the following SRM indications:

o SRM 2A	20 CPS
o SRM 2B	7 CPS
o SRM 2C	8 CPS
o SRM 2D	7 CPS

Subsequently, a detector failure results in SRM 2D indication reading UPSCALE.

Based on the above conditions,

SRM 2A reactor power indication will \_\_\_\_\_\_.

The SRM RPS SCRAM Setpoint is \_\_\_\_\_\_.

A. reduce to a lower value;

 $7x10^4$  CPS

By reduce to a lower value;

3x10<sup>5</sup> CPS

C. remain approximately the same;

7x10<sup>4</sup> CPS

D. remain approximately the same;

3x10<sup>5</sup> CPS

# 15. 215005K1.08 001

Unit 2 is operating at 100% RTP with the following;
o APRM 2B INOP and BYPASSED
Subsequently, APRM 2C fails INOP.
Based on the above conditions,  The "background" for Reactor power on the SPDS "Primary Display" will be
A. green
B* yellow
C. orange
D. magenta

# 16. 217000K6.01 001

RCIC is injecting into **Unit 2** following a transient.

Subsequently, 2R24-S021, 250V DC RX BLDG ESSEN. MCC 2A, DE-ENERGIZES.

Ba	sed on the above conditions,
	RCIC will injecting into the RPV.
	2E51-F008, Outboard Isolation Valve, LOST power.
A <b>:</b>	continue;
	has
B.	continue;
	has NOT
C.	stop;
	has
D.	stop;
	has NOT

#### 17. 218000A2.06 001

**Unit 1** is operating at 100% RTP when the ADS timer associated with the SPDS "Primary" Display screen INADVERTENTLY actuates.

The OATC reports ALL Key Parameters are NORMAL

Ba	sed on the above conditions,
	The ADS timer that is counting down is the
	IAW 34AB-E10-001-1, Inadvertent Initiation Of ECCS/RCIC, the Immediate Operator Action is to

- A. ADS Low Water Level Timer; shutdown ADS per the system operating procedure
- B. ADS Low Water Level Timer; place the ADS INHIBIT switches to the INHIBIT position
- C. ADS Logic Timer; shutdown ADS per the system operating procedure
- D. ADS Logic Timer; place the ADS INHIBIT switches to the INHIBIT position

18. 218000A3.09 001

A LOCA has occurred on **Unit 2**.

AUTO BLOWDOWN TIMERS INITIATED, 602-306, has just ILLUMINATED.

Based on the above conditions,

One (1) minute later, the AUTO BLOWDOWN TIMERS INITIATED, 602-306, will AUTOMATICALLY reset (turn GREEN) if \_\_\_\_\_\_\_.

A. RWL increases to -90 inches

- B. Drywell pressure decreases to 1.5 psig
- C. ALL of the RHR and CS pumps are secured
- D. the ADS Inhibit Switches are placed in the "INHIBIT" position

#### 19. 223002A1.04 001

**Unit 2** is operating at 20% RTP when the following occurs:

At 10:00, 2B21-F022A, Inboard MSIV fully closes At 10:02, 2B21-F028B, Outboard MSIV fully closes At 10:04, 2B21-F022C, Inboard MSIV fully closes

Based ONLY on the above conditions with respect to MSIV position input to the RPS Logic,

At 10:03, the TOTAL number of 2C71-K14, RPS Scram Relays, that are ENERGIZED is \_\_\_\_\_\_.

At 10:05, the TOTAL number of 2C71-K14, RPS Scram Relays, that are

A**!** four (4); zero (0)

ENERGIZED is .

- B. four (4); four (4)
- C. eight (8); zero (0)
- D. eight (8); four (4)

20. 223002A4.04 001

	RCIC ISOLATION VLV F007/F008 NOT FULLY OPEN, 602-336, will ILLUMINATE based on exceeding the setpoint, followed by a minute time delay.
A.	RCIC Equipment Room Temp High; 14
В.	RCIC Equipment Room Temp High; 29
C.	Suppression Chamber Area Air Temp High; 14
D <b>?</b>	Suppression Chamber Area Air Temp High; 29

# 21. 234000A3.01 001

Unit 2 is in a refueling outage with the following conditions.

	o Rx Mode Switch Locked in the REFUEL position o Control rods One control rod is fully WITHDRAWN
	o Main Fuel Grapple Lowered, but verified to be able to clear the cattle chute by 3 ft.  (Empty grapple move)
	o NO rod is selected
	e Refuel Bridge is over the Spent Fuel Pool and is moving toward the core to pick up the fuel ndle at position 17-42.
Ba	sed on the above conditions,
	The Refuel Bridge move to the intended core location.
	When the Refuel Bridge stops, if the Grapple Raise/Lower switch is placed in the "LOWER" position, the Main Grapple lower.
A.	will; will
B.	will; will NOT
C.	will NOT; will
D <b>Y</b>	will NOT; will NOT

#### 22. 239002A2.06 001

Unit 2 experienced a Main Turbine trip, resulting in a RPV Pressure INCREASE to 1135 psi	Unit 2 experienced a Ma	n Turbine trip, resu	lting in a RPV Press	sure INCREASE to	1135 psig.
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The following conditions currently exist:

o Reactor Pressure 940 psig

o Torus temp 96°F and steady

Based on the above conditions,

The MAXIMUM number of SRV's that will have automatically opened is \_\_\_\_\_\_.

IAW 34SO-E11-010-2, RHR Sytem, the MINIMUM number of loop(s) of RHR REQUIRED to be placed in Torus Cooling is \_\_\_\_\_\_\_.

A\* eight (8); one (1)

- B. eight (8): two (2)
- C. four (4); one (1)
- D. four (4); two (2)

	Unit 2	is	operating	at 25%	RTP.
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o Main Generator load is 150 MWE

Ba	sed on the above plant conditions and IAW 34SO-N30-001-2, Main Turbine Operation,
	Turbine Control Valve (TCV) number 4 will be
	If the "CLOSE VALVES" button on the "Control" → "Speed" Screen is DEPRESSED, of the TCVs will CLOSE.
A.	throttled open; NONE
B.	throttled open; ALL
C <b>?</b>	closed; NONE
D.	closed; ALL

#### 24. 259001A1.04 001

**Unit 2** is operating at 100% RTP with the following RWL indications:

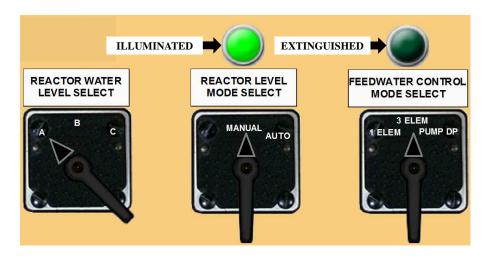
- o 2C32-R606A, GEMAC, indication: +36.9 inches
- o 2C32-R606B, GEMAC, indication: +36.0 inches
- o 2C32-R606C, GEMAC, indication: +37.9 inches

Subsequently, the REFERENCE leg for RWL instrument 2C32-R606A develops a leak equivalent to a 5 inch per minute change on 2C32-R606A.

Bas	sed on the above conditions,
	INITIALLY, RFPT speeds will
	Five (5) minutes later, FINAL RFPT speed will be equivalent to
A <b>Y</b>	DECREASE; windmilling speed

- B. DECREASE; turning gear speed
- C. INCREASE; windmilling speed
- D. INCREASE; turning gear speed

**Unit 2** is at 75% RTP with the following RWLC System indications:



NOTE: 2C32-R600, FW Master Controller, is operating in AUTOMATIC.

Based on the above conditions,

2C32-R600, FW Master Controller, is currently using the \_\_\_\_\_\_ as its value for RWL.

The FW Master Controller is currently operating in \_\_\_\_\_\_ Control.

- A. instrument selected from the Reactor Water Level Select switch; Single Element
- B. instrument selected from the Reactor Water Level Select switch; Three Element
- C. Median Level Signal Processor, 2C32-K648 output; Single Element
- D. Median Level Signal Processor, 2C32-K648 output; Three Element

# 26. 261000A4.01 001

**BOTH Units** are operating at 100% RTP when a RWCU System break occurs in the Unit 2 Reactor Building.

<u>At 10:05</u>: 2D11-K609A-D, Rx. Bldg. Contaminated Area Radiation, monitors indicate 20 mR/hr

Based on the	above conditions and IAW 34S0	O-T46-001-1/2, Standby Gas Treatment System,
	LIEST listed time that the is	SBGT fan will have automatically
A. 1B; 10:15		
B <b>Y</b> 1B; 10:10		
C. 2B; 10:15		
D. 2B; 10:10		

#### 27. 262001A1.04 001

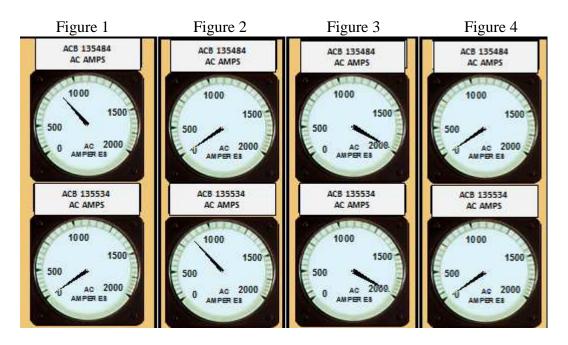
**Unit 2** is at 70% RTP with an operator transferring 4160 VAC Bus 2D to its NORMAL supply. The following conditions currently exist:

- o Voltages are matched
- o Sync switch for the 4160 VAC 2D Normal breaker is in the ON position
- o Sync light is at its dimmest (12 O'Clock position and steady)
- o Station SVC Interlock Cutout switch for ACB 135484-135534 is in the OFF-(DOWN) position

Subsequently, the operator places the control switch for 135484 (Normal supply breaker) in the close position for one (1) second and then releases the switch.

Based on the above conditions,

The ampere indication will be as shown on \_\_\_\_\_.



- A. Figure 1
- B. Figure 2
- CY Figure 3
- D. Figure 4

<b>Unit 2</b> was operating at 100% RTP when the following occur	ıred
--	------

- o Loss of Off-Site Power (LOSP)
- o EDG 2A fails to start

Based on the above conditions and without any operator actions,

After the LOSP, the Unit 2 Vital AC Bus will INITIALLY receive power from \_\_\_\_\_\_.

Subsequently, if this power supply is lost, the Vital AC Bus \_\_\_\_\_ automatically transfer to ANOTHER power source.

A. the Vital AC batteries; will

B**y** the Vital AC batteries; will NOT

- C. 600 V Bus 2D via Vital AC battery charger; will
- D. 600 V Bus 2D via Vital AC battery charger; will NOT

# 29. 263000A1.01 001

**Unit 2** Division 1 125VDC Station Service Battery Chargers are being operated in the EQUALIZE Mode.

	sed on the above conditions and IAW 34SO-R42-001-2, 125 VDC And 125/250 VDC stem,
	In EQUALIZE Mode, the charger output voltage to the battery will be than when the charger is operating in the FLOAT Mode.
	Without re-charging, the 125 VDC Station Service batteries are sized to have adequate storage capacity to carry the required load for approximately
A.	equal; 2 hours
B.	equal; 8 hours
C <b>Y</b>	higher; 2 hours
D.	higher; 8 hours

## 30. 264000K1.05 001

Which ONE of the following knobs can be used to completely shut off the fuel supply to a running Plant Hatch Emergency Diesel Generator?



## AY LOAD LIMIT

- **B. SYN INDICATOR**
- C. SPEED DROOP
- D. SYNCHRONIZER

#### 31. 264000K5.05 001

IAW 34SO-R43-001-2, Diesel Generator Standby AC System, Section 7.2.1.2, Synchronizing Diesel Generator 2A to an Energized Bus,

When manually	y synchronizing l	EDG 2A to an energized bus, the synchroscope is
REQUIRED to	be rotating in a	direction which will REDUCE the probability of
causing a	trip.	

After EDG 2A output breaker is closed, exceeding the Crankcase pressure setpoint automatically trip EDG 2A.

- A. differential current; will
- B. differential current; will NOT
- Cy reverse power; will
- D. reverse power; will NOT

#### 32. 272000K5.01 001

Unit 2 is at 50% RTP increasing power to 100% RTP.

The following conditions exist:

- o The Hydrogen Injection System controller is in AUTOMATIC/EXTERNAL
- o The Hydrogen flow setpoint is set at 5 SCFM

At 10:00, Reactor Power is 100% RTP.

At 10:30, Hydrogen flow is ISOLATED.

Based on the above conditions,

As reactor power is increased, the Hydrogen flow rate will \_\_\_\_\_.

At 10:35, the Main Steam Line radiation levels will have \_\_\_\_\_.

AY INCREASE;

**DECREASED** 

B. INCREASE;

**INCREASED** 

C. REMAIN the SAME;

**DECREASED** 

D. REMAIN the SAME;

**INCREASED** 

# 33. 286000K2.02 001

IAW 34SO-X43-001-1, Fire Pumps Operating Procedure,

The power supply to 1X43-C001, Electric Fire Pump, is \_\_\_\_\_\_.

AY 4160 VAC 1E, 1R22-S005

- B. 4160 VAC 1F, 1R22-S006
- C. 600 VAC 1A, 1R25-S051
- D. 600/208 VAC 1A, 1R24-S028

#### 34. 290002K5.01 001

**Unit 2** is operating at 100% RTP. The following information was obtained from a 3D MONICORE Monitor Case printout:

MFLCPR	LOC	MFLPD	LOC
1.011	11-26	0.974	29-42-20

Based on the above conditions,

At this time, the	thermal limit has been EXCEEDE	ED.
The basis for the th	nermal limit identified above is to avoid the p	potential for .

#### A. MFLPD;

fuel rod transition boiling

## B. MFLPD;

fuel cladding plastic strain

## CY MFLCPR;

fuel rod transition boiling

## D. MFLCPR;

fuel cladding plastic strain

/FW1\_ ebchh[VWbWtSbb'[US`fcgWtf[a` during exam administrationfi? 8>B6 Ž? Sj [\_ g\_ 8cbUf[a` aX>[\_ [ff]`YBai Wtf6Weffk? 8>5BDŽ? Sj [\_ g\_ 8cbUf[a` aX>[\_ [ff]`Y5clff[US^Bai WtDSf[a

# 35. 290003K3.02 001

	it 1 is in Mode 3. it 2 is in Mode 4, moving <u>irra</u>	diated fuel in the Unit 2 Fuel Pool.	
Sub	osequently, the following occu	ırs;	
	o 600 VAC Bus 1C o 600 VAC Bus 1D		
Bas	sed on the above conditions,		
	LCO 3.7.5, Control Room Ai to BOTH Units.	r Conditioning (AC) System,	_ currently applicable
	Main Control Room instrume	entation cooling been lost.	
	is NOT; has		
	is NOT; has NOT		
C <b>Y</b>	is; has		
D.	is; has NOT		

## 36. 295001AK3.04 001

Unit 2 is operating at 60% RTP.
Subsequently, ONE Recirc Pump trips.
Unit 2 stabilizes inside the Region of Potential Instabilities on the power to flow map.
APRM bandwidth oscillations are observed and are INCREASING.
Based on the above conditions,  The LOWEST listed APRM peak to peak oscillation that REQUIRES a reactor scram is  The reason for the reactor scram is to avoid EXCEEDING the Limit for
A. 3%; MCPR
B. 3%; MAPRAT
C. 7%; MCPR
D. 7%; MAPRAT

#### 37. 295003AK2.03 001

**Unit 1** is operating at 100% RTP when 1R25-S036, Essential Cabinet 1A, de-energizes and CANNOT be re-energized.

The SRO directs a NPO to cross connect Instrument Buses IAW 34AB-R25-002-1, Loss Of Instrument Buses.

Based on the above conditions and IAW 34AB-R25-002-1,

To energize 1R25-S064, Instrument Bus 1A, from 1R25-S065, Instrument Bus 1B, the NPO must utilize \_\_\_\_\_\_.

Once power is restored to 1R25-S064, 1N62-F527, Off Gas Stack Inlet Valve, \_\_\_\_\_\_.

- A. breakers ONLY; will automatically open
- B**y** breakers ONLY; must be manually opened
- C. breakers and disconnects; will automatically open
- D. breakers and disconnects; must be manually opened

#### 38. 295004AA1.03 001

Unit 2 is operating at 100% RTP.

- o A loss of 2R22-S016, 125/250VDC Switchgear 2A, occurs
- o Subsequently, a Main Turbine trip occurs

Based on the above conditions and IAW 34AB-R22-001-2, Loss Of DC Buses,				
	The Main Generator output breakers			
	The 4160 VAC Station Service Busesalternate supply.	automatically transfer to their		
A.	will automatically open; will			
В.	will automatically open; will NOT			
C.	must be manually opened; will			
D <b>?</b>	must be manually opened; will NOT			

#### 39. 295005G2.2.37 001

Unit 2 was operating at 100% RTP with 4160 VAC 2G Emergency Bus on Alternate Supply.

Subsequently, the Unit 2 Main Turbine trips on Generator Differential Overcurrent.

Based on the above conditions and one (1) minute after the Main Turbine trips,

The Recirc pumps \_\_\_\_\_\_ be operating.

4160 VAC 2C \_\_\_\_\_ to its Startup Auxiliary Transformer.

A. will;

must be manually transferred

B. will;

will automatically transfer

CY will NOT;

must be manually transferred

D. will NOT;

will automatically transfer

#### 40. 295006AK1.02 001

Unit 2 was at 100% RTP when a spurious reactor scram occurred.

## ALL Control Rods fully inserted with the following EXCEPTIONS:

Control Rod	Position	Control Rod	Position
26-31	02	10-27	02
26-23	02	30-11	02
22-35	02	30-47	02
22-27	02	46-19	02
50-27	02	02-31	02

Based on the above conditions,

Based ONLY on the current rod positions, the reactor \_\_\_\_\_ in a Cold Shutdown Rod Configuration.

IAW Plant Procedures, ALL automatic scram signals \_\_\_\_\_ ALLOWED to be BYPASSED.

- A. is; are
- B\* is; are NOT
- C. is NOT; are
- D. is NOT; are NOT

#### 41. 295007AA2.02 001

Unit 2 has experienced a transient.

The following conditions exist at the given time:

At 10:05, RPV pressure is 1056 psig At 10:08, RPV pressure is 1075 psig

At 10:11, LLS is controlling RPV pressure with the following:

- o 2B LLS valve continuously OPEN
- o 2G LLS valve cycling on LLS setpoint

Based on the above conditions,

IAW 34AB-C71-001-2, Scram Procedure, the EARLIEST listed time that LLS is ALLOWED to be manually ARMED is \_\_\_\_\_\_.

At 10:11, based on SRV positions, Reactor Power is approximately \_\_\_\_\_\_.

A. 10:05;

6% RTP

B. 10:05;

11% RTP

C. 10:08;

6% RTP

DY 10:08

11% RTP

#### 42. 295010AK3.01 001

Unit 1 is operating at 100% RTP when Drywell pressure starts slowly INCREASING.

Drywell venting is placed in service using:

- o 1T48-R615A, Drwl Flow Controller
- o 1T48-F336A, Drywell Vent Flow Cntl Valve

Based on the above conditions,

The bases for placing Drywell venting in service \_\_\_\_\_\_ to prevent a high Drywell pressure reactor scram.

IAW 34SO-T48-002-1, Containment Atmospheric Control and Dilution Systems, DO NOT EXCEED 100% demand on Controller, 1T48-R615A, to prevent damaging \_\_\_\_\_\_\_.

- A. is NOT; 1T48-R615A
- B. is NOT; 1T48-F336A
- C. is; 1T48-R615A
- D**Y** is; 1T48-F336A

#### 43. 295014G2.1.28 001

Unit 2 is operating at 75% RTP when HPCI inadvertently starts.

34SO-E41-001-2, High Pressure Coolant Injection (HPCI) System, has been entered.

- o HPCI Initiation signal WHITE light will NOT reset
- o A NPO has just depressed the HPCI Trip Pushbutton

Based on the above conditions and IAW 34SO-E	41-001-2,
The HPCI Trip Pushbutton will be RELEASE	ED
The purpose for this action is to	on a subsequent initiation signal.

- A. WHEN the 2E41-F001, HPCI Turb Steam Supply valve, indicates full CLOSED; prevent HPCI from automatically restarting
- B. WHEN the 2E41-F001, HPCI Turb Steam Supply valve, indicates full CLOSED; allow HPCI to automatically restart
- CY ONLY after HPCI TURBINE BRG OIL PRESS LOW, 601-112, alarm is received; prevent HPCI from automatically restarting
- D. ONLY after HPCI TURBINE BRG OIL PRESS LOW, 601-112, alarm is received; allow HPCI to automatically restart

#### 44. 295016AA2.01 001

The Main Control Room has been evacuated.

The **Unit 2** reactor was NOT shutdown prior to leaving the Control Room.

- o Local actions have been taken to scram the reactor
- o ALL RSDP transfer switches have been placed in the EMERGENCY position

Based on the above conditions and IAW 31RS-OPS-001-2, Shutdown From Outside Control Room,

From the Remote Shutdown Panel, \_\_\_\_\_ can be started.

An operator stationed in the TSC will utilize \_\_\_\_\_ to verify automatic actions, isolations and initiations will occur.

- A. ONLY one CRD pump; the Plant Process Computer
- B**.⁴** ONLY one CRD pump; SPDS
- C. BOTH CRD pumps; the Plant Process Computer
- D. BOTH CRD pumps; SPDS

#### 45. 295017AA1.05 001

A Site Area Emergency has just been declared on **Unit 2** due to a leak into secondary containment. A Prompt Offsite Dose Assessment is in progress but results are unavailable at this time.

SPDS "MIDAS" Screen is attached on the next page.

The following Table is from NMP-EP-111-002, Emergency Notification Network Communicator Instructions - Hatch:

Wind Direction From	Rally Point	Site Exit Route	Evacuation Route/ State Reception Center
340° - 60°	Gate 17	Main Access Road	U.S. Hwy. 1 - North to Toombs Co. High School/ Lyons
61º - 110º	PESB	Road behind Low Level Radwaste Building	U.S. Hwy. 1 - South to Appling Co. High School/ Baxley
111º - 225º	PESB	Main Access Road	U.S. Hwy. 1 - South to Appling Co. High School/ Baxley
226º - 339º	PESB	Main Access Road	EITHER direction on U.S. Hwy. 1 to Toombs Co. High School/ Lyons OR Appling Co. High School/ Baxley

Based on the above conditions,				
	Radiation levels in the Main Stack exceeded the HI-HI setpoint.			
	IAW NMP-EP-111-002, the site evacuation route is			
A <b>"</b>	have; US #1 North			
B.	have; US #1 South			

C. have NOT; US #1 North

D. have NOT; US #1 South

# MIDAS INEODIAINATION

			5	KAMAN			
Z O	100M WIND DIR 1Y33-R603 <b>50</b>	RAINFALL 15 MIN. AVG .000	U2 RX. BLDG. VENT	NORMAL RANGE	5.4E01	5.4E01	
KMAII	UD DIR		<b>-</b>	KAMAN			
MIDAS INFORMATION	10M WIND DIR 1Y33-R601 <b>65</b>	DELTA T 100-10 -1.0	U1 RX, BLDG, VENT	NORMAL RANGE	6.7E01	6.7E01	1
	D SPD 003		5	NORM	6.7	6.7	-
≥	100M WIND SPD 1Y33-R603 4.0	60-10 -0.5		KAMAN	4.0E-02		
METEOROLOGICAL	10M WIND SPD 1Y33-R601 5.0	AMBIENT TEMP (F) 10M 55	RADIOLOGICAL MAIN STACK	NORMAL RANGE			STABILITY CLASS

46. 295018AA1.01 001

Unit 2 is operating at 50% RTP.

Subsequently, one (1) RBCCW pump and one (1) PSW pump trips.

Both RBCCW Header and PSW Header pressures stabilize at 93 psig.

Based on the above conditions and IAW 34AB-P42-001-2, Loss Of Reactor Building Closed Cooling Water, the NPO will \_\_\_\_\_\_ the Standby RBCCW pump.

A Standby RBCCW pump automatic start signal \_\_\_\_\_\_ when the condition clears.

- A. confirm the automatic start of; will automatically reset
- B. confirm the automatic start of; must be manually reset
- C. manually start; will automatically reset
- D. manually start; must be manually reset

#### 47. 295019G2.1.32 001

On **Unit 2**, IAW 34AB-P51-001-2, Loss of Instrument And Service Air System Or Water Instrusion Into the Service Air System,

	2P52-F565, Rx Bldg Inst $N_2$ To Non-Int Air El. 185 Isol Vlv, will FIRST open when Non-Interruptible Essential Air Pressure DECREASES to
	IF 2P52-F565 is continuously cycling, THEN 2P52-F565 is OPENED and the supply breaker is turned OFF to
A.	90 psig; stabilize nitrogen system pressure
B.	90 psig; prevent failure of the valve motor
C.	80 psig; stabilize nitrogen system pressure
D <b>?</b>	80 psig; prevent failure of the valve motor

## 48. 295020AK2.01 001

Unit 2 is operating at 100% RTP when the following occurs:

2R22-S017, 125/250 VDC Batt Swgr 2B, DE-ENERGIZES.

Based on the above electrical loss,					
	A loss of	would result in an inadvertent	Group 1 Isolation.		
	DDC Day 2 A .				
A.	RPS Bus 2A; Outboard				
B.	RPS Bus 2A;				
	Inboard				
C <b>Y</b>	RPS Bus 2B; Outboard				
	Outobald				
D.	RPS Bus 2B;				
	Inboard				

## 49. 295021AA1.06 001

**Unit 2** is shutdown with RHR Loop A aligned for Shutdown Cooling with the following conditions:

	o Reactor Shutdown o The Drywell is open and available for access o 2B21-F003, Reactor Head Vent Valve o 2B21-F004, Reactor Head Vent Valve o 2B21-F005, Reactor Manual Head Vent Valve	Two (2) days ago  OPEN  OPEN  CLOSED
Sul	osequently, a loss of SDC occurs.	
Bas	sed on the above conditions and with NO Operator a	ctions,
	IAW 34AB-E11-001-2, Loss of Shutdown Cooling, expected to reach 212°F in approximately	
	If Reactor bulk coolant temperature reaches 250°F, will	Drywell temperature indications
RE	FERENCE PROVIDED	
A <b>Y</b>	1 hour 5 minutes; be slowly increasing	
B.	1 hour 5 minutes; still be approximately the same	
C.	5 hours 54 minutes; be slowly increasing	
D.	5 hours 54 minutes;	

still be approximately the same

#### 50. 295023AK1.01 001

Both Units are operating at 100% RTP with the following activities in progress:

- o Irradiated fuel movement in progress between the Fuel Pools
- o **Unit 1** Normal Drywell venting in progress
- o **Unit 2** Normal Drywell venting in progress

At 13:00, the Refuel Bridge reports that a bundle has been bent and visible gas bubbles are rising from the currently latched fuel bundle.

In the Main Control Room there are NO lit annunciators from this malfunction.

At 13:03, the following indications are observed in the Main Control Room:

o ALL Refueling Floor (RF) ARMs	30  mR/hr
o 1D11-K611 A-D, RF Vent Exhaust Radiation	17 mR/hr
o 2D11-K611 A-D, RF Vent Exhaust Radiation	15 mR/hr
o 2D11-K634 A-D, RF Vent Exhaust Radiation	7 mR/hr
o 2D11-K635 A-D, RF Vent Exhaust Radiation	7 mR/hr
,	

Based on the above conditions,

Normal Drywell venting \_\_\_\_\_\_ isolated on BOTH Units.

The EARLIEST listed time that entry into 34AB-J11-001-2, Irradiated Fuel Damage During Handling, is REQUIRED is \_\_\_\_\_\_.

A. has;

13:00

B. has:

13:03

CY has NOT;

13:00

D. has NOT;

13:03

#### 51. 295024EK2.13 001

Unit 2 is operating at 75% RTP. Reactor power is being reduced due to a leaking SRV.

The following conditions exist:

0	Drywell pressure	0.4 psig
o	Torus pressure	0.2 psig
o	RHR pump 2A	operating
o	Torus Cooling	in service
o	Torus Spray	in service

At 13:00, Drywell pressure starts increasing at 0.3 psig/minute.

At 13:03, DRYWELL PRESS HIGH, 602-210, ILLUMINATED.

Based on the above conditions,

The EARLIEST listed time that RHR Loop A in Torus Spray will automatically isolate is \_\_\_\_\_\_.

At 13:10, to return Torus Spray to service, the Containment Spray Vlv Control switch \_\_\_\_\_\_ REQUIRED to be placed in the MANUAL position.

AY 13:05;

is

- B. 13:05; is NOT
- C. 13:03; is
- D. 13:03; is NOT

#### 52. 295025EA2.01 001

31EO-EOP-107-2, Altenate RPV Pressure Control, is in progress.

- o RPV pressure 1060 psig and slowly rising
- o HPCI system is aligned in Pressure Control Mode
- o 2E41-R612, HPCI flow controller is in AUTOMATIC with the setpoint at 2500 gpm

Based on the above conditions and IAW 31EO-EOP-107-2,

To stabilize RPV pressure, the operator will \_\_\_\_\_\_.

- A. RAISE the setpoint on 2E41-R612, HPCI flow controller
- B. LOWER the setpoint on 2E41-R612, HPCI flow controller
- C. throttle 2E41-F011, Test to CST VLV, in the CLOSE direction
- D. throttle 2E41-F011, Test to CST VLV, in the OPEN direction

#### 53. 295026EK3.05 001

Unit 2 is operating at 100% RTP with a SRV experiencing leakage.

31EO-EOP-012-2, PC Primary Containment Control, is in progress.

Based on the above conditions and IAW 31EO-EOP-012-2,

The LOWEST listed Torus temperature REQUIRING a reactor scram is \_\_\_\_\_\_.

The reason for the reactor scram is to ensure the reactor is shutdown before Torus temperature reaches \_\_\_\_\_\_ limit.

- A. 106°F; the BIIT Curve
- B. 106°F; any ECCS NPSH
- CY 111°F; the BIIT Curve
- D. 111°F; any ECCS NPSH

#### 54. 295028G2.4.31 001

Unit 2 is operating at 100% RTP when a small pipe break in the Drywell occurs.

- o Drywell Pressure has increased to 2.5 psig
- o Drywell Temperature has increased to 130°F

Based on the above conditions and IAW 31EO-EOP-100-2, Miscellaneous Emergency Overrides,

The HIGHEST listed Drywell temperature point at which the Drywell Chillers are allowed to be restarted is \_\_\_\_\_\_ .

- A. 149°F
- B**Y** 249°F
- C. 279°F
- D. 339°F

## 55. 295030EK3.01 001

Unit 1 has experienced a leak resulting in Torus water level decreasing.

Ba	sed on the above conditions and IAW 31EO-EOP-012-1, Primary Containment Control,
	If the leak <u>is isolated</u> , to restore Torus water level, the System suction piping from the CST can be used.
	If the leak <u>can NOT</u> be isolated, the reason for performing an Emergency Depressurization on low Torus water level is to prevent
A <b>"</b>	Core Spray; direct pressurization of the Torus air space
B.	Core Spray; excessive loading of the SRV tail pipes and supports
C.	HPCI; direct pressurization of the Torus air space
D.	HPCI; excessive loading of the SRV tail pipes and supports

#### 56. 295031EK2.11 001

### **Unit 1** is operating at 100% RTP.

An event occurs causing RWL to decrease and results in the following 1C32-R606A-C GEMAC, indications:

<u>Time</u>	<u>RWL</u>
10:00	+40 inches and steady
10:01	+5 inches and decreasing
10:02	-5 inches and decreasing
10:15	+35 inches and steady

Based on the above conditions and ONLY on plant automatic response,

The NPO is first REQUIRED to perform RWL control actions per Placard RC-2
at
At 10:15, alarm, REACTOR VESSEL WATER LEVEL HIGH/LOW, 603-141,
will be

A. 10:01;

ILLUMINATED (Revised to FLASHING WHITE)

- B. 10:01; EXTINGUISHED (Revised to SOLID GREEN)
- C. 10:02; ILLUMINATED (Revised to FLASHING WHITE)
- DY 10:02;

EXTINGUISHED (Revised to SOLID GREEN)

(Answer revisions provided per applicant question during exam administration)

#### 57. 295032EA2.01 001

**Unit 2** is operating at 100% RTP with the following HPCI Pump Room Cooler alignment:

- o HPCI Pump Room Cooler "A" (2T41-B005A) is operating with its control switch in the "RUN" position
- o HPCI Pump Room Cooler "B" (2T41-B005B) is NOT running with its control switch in the "AUTO" position

Subsequently, a steam leak develops in the HPCI Room causing HPCI Area temperatures to increase.

<u>Time</u>	<b>HPCI Room Temp</b>		
01:00	95°F		
01:05	105°F		
01:15	135°F		
01:20	170°F		

Based on the above conditions,

The EARLIEST listed time that HPCI Pump Room Cooler B, (2T41-B005B), will
automatically start is
•
The EARLIEST listed time that HPCI will have RECEIVED an automatic isolation signal
is

- A. 01:00;
  - 01:15
- B. 01:00;
  - 01:20
- C. 01:05;
  - 01:15
- DY 01:05;
  - 01:20

Unit 2 was operating at 100% RTP when an ATWS results in the following:

- o SBLC System fails to initiate from the Control Room
- o An Operator attempts to locally initiate SBLC, but SBLC fails to initiate

Based on the above conditions and IAW 31EO-EOP-109-2, Alternate Boron Injection,

To inject Boron into the RPV using the CRD System, the operator will DIRECTLY connect hoses to \_\_\_\_\_\_.

When boron is being injected, 2C11-F015A & F015B, CRD Pump Minimum Flow valves, are REQUIRED to be in the \_\_\_\_\_\_ position.

- AY 2C41-F034, SBLC Combined Drain Valve; closed
- B. 2C41-F034, SBLC Combined Drain Valve; open
- C. 2C41-F015, SBLC Suction Line Drain Valve; closed
- D. 2C41-F015, SBLC Suction Line Drain Valve; open

#### 59. 295038EK1.02 001

**Unit 2** was operating at 100% RTP when a radiological event occurred resulting in an automatic start of the SBGT system.

An emergency is declared based on a radiological release at the following times:

At 09:00, Alert (RA1) At 10:00, Site Area (RS1)

Based on the above conditions,

The EARLIEST listed time that an entry condition into the <u>RR portion</u> of 31EO-EOP-014-2, SC-Rad Release Control, EOP Flowchart, will exist is \_\_\_\_\_\_.

IAW 34SO-T46-001-2, SBGT System Operation, operation of \_\_\_\_\_\_ normally REQUIRED to maintain adequate negative Reactor Building dP.

A\* 09:01;

ONLY one (1) SBGT Train is

B. 09:01;

**BOTH SBGT Trains are** 

C. 10:01;

ONLY one (1) SBGT Train is

D. 10:01;

**BOTH SBGT Trains are** 

#### 60. 300000K2.01 001

The power supply for **Unit 2** Station Service Air Compressor (SSAC) 2A is \_\_\_\_\_\_.

The power supply for **Unit 2** SSAC 2C is \_\_\_\_\_\_.

A**Y** 600VAC Bus 2C; 600VAC Bus 2A

- B. 600VAC Bus 2C; 600VAC Bus 2BB
- C. 600VAC Bus 2D; 600VAC Bus 2A
- D. 600VAC Bus 2D; 600VAC Bus 2BB

## 61. 400000A2.04 001

The following annunciator on **Unit 1** is in the alarm condition:

o SERVICE WATER EFFLUENT RADIATION HIGH, 601-407

Ba	sed on the above condition,
	The flowpath containing this alarm AUTOMATICALLY isolate.
	IAW 601-407, the NPO will notify
A.	will; Chemistry to sample Circulating Water to determine if effluent limits have been exceeded
B.	will; the Radwaste Operator to confirm closed / close this discharge flowpath
C <b>?</b>	will NOT; Chemistry to sample Circulating Water to determine if effluent limits have been exceeded
D.	will NOT; the Radwaste Operator to confirm closed / close this discharge flowpath

#### 62. 400000K3.01 001

**Unit 1** was operating at 100% RTP when a total loss of Reactor Building Closed Cooling Water (RBCCW) occurred.

Based on the above conditions,

The LOWEST RWCU Non-Regenerative Heat Exchanger outlet temperature which will result in a RWCU system isolation is \_\_\_\_\_\_.

This isolation signal will result in automatic closure of ONLY the \_\_\_\_\_\_, Rx Water Cleanup Valve.

- A. 130°F; 1G31-F001
- B. 130°F; 1G31-F004
- C. 140°F; 1G31-F001
- D**Y** 140°F; 1G31-F004

## 63. 500000EK1.01 001

	The LOWEST listed Hydrogen concentration that will REQUIRE entry into 31EO-EOP-012-1, PC Primary Containment Control, is					
	IAW 31EO-EOP-104-1, Primary Containment Venting for Hydrogen/Oxygen Control, the PREFERRED method to vent containment is to align the CAD System vent valves to the					
A <b>Y</b>	1.6%; Torus					
B.	1.6%; Drywell					
C.	1.9%; Torus					
D.	1.9%; Drywell					

#### 64. 600000AA2.14 001

Unit 1 is shutdown with RPV pressure at 100 psig.

Subsequently, a fire has started in Fire Zone 1408, Switchgear Room 1F.

The fire brigade has requested all electrical switchgear located in the room to be de-energized to aid in suppressing the fire.

Based on the above conditions,

Shutdown Cooling can be placed into service using \_\_\_\_\_\_ to achieve and maintain a Cold Shutdown condition.

- A. RHR pump 1B and RHRSW pump 1C
- BY RHR pump 1B and RHRSW pump 1D
- C. RHR pump 1C and RHRSW pump 1C
- D. RHR pump 1C and RHRSW pump 1D

#### 65. 700000G2.4.45 001

**Unit 1** is operating at 100% RTP when a grid disturbance results in all 4160 VAC Emergency Buses indicating 3700 VAC.

The load dispatcher reports that these conditions will exist for 4 hours.

Ba	sed on the above conditions,
	IAW 34AB-S11-001-0, Operation With Degraded Voltage, one (1) hour later, 4160 VAC Bus is REQUIRED to be powered by its associated EDG on <b>Unit</b> 1
	When transferring the 4160 VAC Emergency Bus to its associated EDG, the respective LOSS OF OFF SITE POWER alarm on 1H11-P652 be RECEIVED.
A <b>Y</b>	1E; will
B.	1E; will NOT
C.	1G; will
D.	1G; will NOT

Both Units are operating at 100% RTP.

Based	l on the	e above	conditions	and IAW	NMP-	AD-016-0	03, Sched	luling and	Calculating	Work
Hours	S.									

The MAXIMUM number of hours that a Nuclear Plant Operator may w	vork in a	ıny
<b>24 hour</b> period is		

The MAXIMUM number of hours that a Nuclear Plant Operator may work in any **7-day/168-hour** period is \_\_\_\_\_\_\_.

- A. 14 hours;
  - 84 hours
- B. 14 hours;
  - 72 hours
- C. 16 hours;
  - 84 hours
- **DY** 16 hours;
  - 72 hours

Unit 1 is in REFUEL with a core shuffle currently in progress.

Based on the above conditions and IAW 34FH-OPS-001-0, Fuel Movement Operation,
The individual in the Main Control Room on the headset with the Refueling SRO REQUIRED to have a NRC License.
The Reactor Mode Switch REQUIRED to be locked in the REFUEL position.
And t

A\* is;

is

B. is; is NOT

C. is NOT;

D. is NOT; is NOT

68. G2.2.38 001

IAW the **Unit 1** Facility Operating License, Southern Nuclear is authorized to operate Plant Hatch at a MAXIMUM <u>steady-state</u> reactor core power level of \_\_\_\_\_\_.

- A. 2777 MWth
- B. 2790 MWth
- C. 2800 MWth
- DY 2804 MWth

**Unit 2** is shutdown with the following conditions:

o RPV pressure	134 psig
o Recirculation pump 2A	OFF
o Recirculation pump 2B	Running

Based on the above conditions and IAW Tech Spec 3.4.7, Residual Heat Removal (RHR) Shutdown Cooling System - Hot Shutdown,

The MINIMUM number of RHR Shutdown Cooling (SDC) subsystems required to be operable, without requiring entry into a Required Action Statement (RAS), is \_\_\_\_\_\_.

A RHR SDC subsystem \_\_\_\_\_ REQUIRED to be in operation.

- A. one (1); is
- B. one (1); is NOT
- C. two (2); is
- D**Y** two (2); is NOT

On Unit 2 the annunciator card for the following has been PULLED and DE-ACTIVATED:

o TURBINE TROUBLE, 650-105

Based on the above condition and IAW 31GO-OPS-014-0,	
	With 650-105 de-activated, a magnetic tile will be installed beside the annunciator number label under the annunciator window.
	The installed magnetic tile will be
A٢	yellow; labeled with a "P"
B.	yellow; blank
C.	white; labeled with a "P"
D.	white; blank

#### 71. G2.3.7 001

**Unit 2** is operating at 100% RTP with a steam leak in the 2A SJAE room. A SO has been assigned to investigate. RP reports that the general area radiation levels are 120 mR/hr.

Bas	sed on the above conditions,
	A RWP will be used to enter the SJAE room and is the type of information you would expect to find on this type of RWP.
A.	Specific; the current Reactor Mode
B <b>?</b>	Specific; Digital Alarming Dosimeter (DAD) settings
C.	General; the current Reactor Mode
D.	General; Digital Alarming Dosimeter (DAD) settings

Unit 2 Radwaste is discharging Waste Sample Tank A to the canal.

Subsequently, the following indication is received:

o 2G11-R045, Total Plant Dilution Flow, recorder indicates 9500 gpm

Ba	sed on the above conditions,	
	The Radwaste discharge to the canal	
	With the existing Specific Release Permit,Waste Sample Tank A to the canal.	_ permitted to restart the discharge of
A <b>*</b>	will automatically terminate; ONLY one (1) restart is	
В.	will automatically terminate; NO restarts are	
C.	must be manually terminated; ONLY one (1) restart is	
D.	must be manually terminated;	

NO restarts are

	The power supplies for the <b>Unit 1</b> Post-Treatment Radiation monitors, 1D11-K615A & K615B, is
	The location of the <b>Unit 1</b> Post-Treatment Radiation detectors NORMAL sample point is just prior to
A.	RPS Bus B & Instrument Bus 1A; EXITING the Main Stack

- B. RPS Bus B & Instrument Bus 1A; ENTERING the Main Stack
- C. 24/48 VDC Cabinet 1A & 1B EXITING the Main Stack
- DY 24/48 VDC Cabinet 1A & 1B ENTERING the Main Stack

#### 74. G2.4.17 001

#### An ATWS exists on **Unit 1**.

- o Reactor Power is slowly DECREASING
- o Boron has NOT been injected into the RPV
- o All SRMs and IRMs have been fully inserted

The following conditions occur:

<u>Time</u>	Reactor Power
01:00	APRMs indicate 4% RTP
01:10	IRMs on Range 7
01:20	IRMs on Range 5
01:30	SRMs indicate 6.0 x 10 <sup>4</sup> CPS

Based on the above conditions and IAW the EOPs,

The EARLIEST	time the reacto	r can be declar	ed SHUTDOWN	with the reactor
SUBCRITICAL	is at	_		

- A. 01:00
- B. 01:10
- CY 01:20
- D. 01:30

An emergency	has	been	declared	at I	Plant	Hatch.
--------------	-----	------	----------	------	-------	--------

Based on the above conditions,

The LOWEST listed emergency classification that will REQUIRE activation of the Technical Support Center (TSC) is \_\_\_\_\_\_ .

The ALTERNATE location for the TSC is \_\_\_\_\_\_.

A. an Alert Emergency; Classroom 172, Simulator Building

B**y** an Alert Emergency; the Main Control Room

- C. a Notification of Unusual Event; Classroom 172, Simulator Building
- D. a Notification of Unusual Event; the Main Control Room

Unit 2 has experienced a non-isolable RCIC pipe break in the Main Steam Tunnel (Chase) resulting in the following conditions:

	o MSIVs o RWL o HPCI ISOL TIMER INITIATI	CLOSED -25 inches and lowering ED, 601-105, ILLUMINATED
Bas	sed on the above conditions,	
	The HIGHEST listed RWL value is	e that HPCI will receive an automatic start signal
	To maintain HPCI for RWL cont HPCI High Area Temperature Iso	rol, the Shift Supervisor will direct bypassing the blation using
A <b>Y</b>	-36 inches; 31EO-EOP-100-2, Miscellaneou	s Emergency Overrides
B.	-36 inches; 31EO-EOP-110-2, Alternate RP	V Water Level Control
C.	-48 inches; 31EO-EOP-100-2, Miscellaneou	s Emergency Overrides
D.	-48 inches; 31EO-EOP-110-2, Alternate RP	V Water Level Control

#### 77. 212000A2.05 001

**Unit 1** is in Mode 2 making preparations to enter Mode 1.

At 08:00, I & C reports the following instruments have failed high:

- o 1B21-N080A, Narrow Range RWL
- o 1B21-N080C, Narrow Range RWL

At 08:30, the Channel associated 1B21-N080A has been placed in the TRIPPED condition.

Based on the above conditions,

At 08:01, if RWL decreases to the RPS setpoint, a FULL Reactor Scram \_\_\_\_\_automatically occur.

At 09:01, IAW TS 3.0.4, WITHOUT performing a Risk Evaluation, Tech Specs \_\_\_\_\_ ALLOW the Reactor Mode switch to be placed in the RUN position.

#### REFERENCE PROVIDED

- A. will;
  - does
- B. will; does NOT
- C**Y** will NOT; does
- D. will NOT; does NOT

#### 78. 214000A2.03 001

Unit 2 is at 90% RTP, performing fully withdrawn control rod exercises.

- o Control rod 26-27 is inop with all required TS actions complete
- o Control rod 18-19 has just been withdrawn to position 48

A coupling check on control rod 18-19 results in the following:

- o 18-19 four rod display position BLANK
- o 18-19 Full-Out light EXTINGUISHED
- o ROD OVERTRAVEL, 603-248, ILLUMINATED

Based on the above conditions,

With NO operator action, one (1) minute later, control rod 18-19 four rod display position
ndication will
AW TS 3.1.3, Control Rod Operability, if control rod 18-19 can NOT be
re-coupled, the EARLIEST REQUIRED action is to

#### AY remain BLANK;

fully insert and disarm the control rod

- B. remain BLANK; place the reactor in hot shutdown
- C. have returned to position 48; fully insert and disarm the control rod
- D. have returned to position 48; place the reactor in hot shutdown

#### 79. 233000G2.2.22 001

IAW TS Bases 3.7.8, Spent Fuel Pool Water Level,

The LOWEST listed Spent Fuel Pool water level that will still meet the LCO 3.7.8 requirements WITHOUT entering a Required Action Statement (RAS) is \_\_\_\_\_\_ .

The MINIMUM water level in the Spent Fuel Pool is REQUIRED to \_\_\_\_\_\_.

#### A. 21.1 feet;

remove decay heat from irradiated fuel assemblies

#### By 21.1 feet;

provide for absorption and transport delay of fission product gases

#### C. 22.1 feet;

remove decay heat from irradiated fuel assemblies

#### D. 22.1 feet;

provide for absorption and transport delay of fission product gases

#### 80. 239002G2.4.31 001

Unit 2 is operating at 90% RTP when SRV 2B21-F013B inadvertently OPENS.

The following conditions occur:

o Torus water temperature INCREASES to 85°F

Five (5) seconds after opening and with NO operator action, the SRV goes CLOSED.

Based on the above conditions and IAW 34AB-B21-003-2, Failure of Safety/Relief Valves,

Tech Spec surveillance requirement \_\_\_\_\_\_ is REQUIRED to be performed.

- A. SR 3.6.2.1, verify Torus average temperature is within the applicable limits
- B. SR 3.4.4.1, verify RCS unidentified & total leakage and unidentified leakage increase are within limits
- C. SR 3.6.1.6.2, verify the LLS system actuates on an actual or simulated automatic initiation signal
- DY SR 3.6.1.8.2, perform a functional test of each required vacuum breaker

#### 81. 262002G2.2.36 001

**Unit 1** is operating at 100% RTP.

Maintenance is troubleshooting a burnt smell coming from the 1R25-S067, Critical Instrument Bus 1B.

Subsequently, at Maintenance request, 1R25-S067 is DE-ENERGIZED.

Based on the above conditions,

IAW U1 TS Bases, a LCO Required Action Statement (RAS) \_\_\_\_\_\_ REQUIRED to be entered for each TS supported load on 1R25-S067, Critical Instrument Bus 1B.

IAW TS 3.8.7, the MAXIMUM amount of time 1R25-S067, Critical Instrument Bus 1B, can remain de-energized without requiring entry into a Shutdown RAS is \_\_\_\_\_\_.

- A. is; two (2) hours
- B. is; eight (8) hours
- C. is NOT; two (2) hours
- Dy is NOT; eight (8) hours

#### 82. 290001A2.04 001

**Unit 1** was operating at 100% RTP when an <u>unisolable</u> steam line break occurred in the Main Steam Tunnel (Chase) area resulting in a Secondary Containment isolation signal.

Based on the above conditions,

A Secondary Containment isolation signal will be received on \_\_\_\_\_\_.

IAW 31EO-EOP-014-1, SC EOP flowchart, exceeding the Max Safe Operating Value in one area requires shutdown IAW \_\_\_\_\_\_ .

A. Unit 1 ONLY;

34GO-OPS-014-1, Fast Reactor Shutdown

B. Unit 1 ONLY;

31EO-EOP-010-1, RPV Control (Non-ATWS)

C. Unit 1 AND Unit 2;

34GO-OPS-014-1, Fast Reactor Shutdown

DY Unit 1 AND Unit 2;

31EO-EOP-010-1, RPV Control (Non-ATWS)

Unit 2 is operating at 90% RTP when Recirc Pump 2A trips.

Bas	sed on the above conditions,
	The APRM setpoint is REQUIRED to be adjusted for Single Loop operation IAW LCO 3.4.1.
	IAW TS Bases, the OPRM Upscale Function OPERABILITY for Technical Specifications purposes is based ONLY on the Based Algorithm.
A <b>Y</b>	Simulated Thermal Power - High; Period
B.	Simulated Thermal Power - High; Amplitude
C.	Neutron Flux - High; Period
D.	Neutron Flux - High; Amplitude

2B

Unit 2 is operating at 100% RTP when the following occurs:

- o Upstream power supply to Instrument Bus 2A is lost and cannot be recovered
- o 34AB-R25-002-2, Loss of Instrument Buses, is entered by the crew
- o Subsequently, the Instrument Buses are cross-tied

В	Based on the above conditions and after power is restored,	
	Instrument Bus 2A OPERABLE, IAW TS 3.8.7 Ba	uses.
	Operator action will be directed to transfer RFPTt	o the M/A station.
A	A. is; 2A	
В	3. is;	
C	2B C¥ is NOT;	
D	2A  D. is NOT;	

Unit 2 is operating at 100% RTP.

- o 2R22-S016, 125/250VDC Switchgear 2A, de-energizes and can NOT be restored
- o 34AB-R22-001-2, Loss of DC Buses, is entered by the crew

Based on the above conditions,

A 34GO-OPS-014-2, Fast Reactor Shutdown, \_\_\_\_\_ REQUIRED.

Upon a trip of the Main Turbine and after TC-1 is complete, to trip the RFPTs, the trip pushbuttons are REQUIRED to be depressed \_\_\_\_\_\_\_.

Ay is;

locally at the RFP front standard

B. is; on panel 2H11-P650

C. is NOT; locally at the RFP front standard

D. is NOT; on panel 2H11-P650

Unit 2 has	experienced	an ATWS.
------------	-------------	----------

o MSIVs Open o Main Generator 135 MWe

The following conditions occur:

<u>Time</u>	<u>Event</u>
07:45	Main Generator load is 115 MWe
08:00	Main Generator load is 75 MWe

Based on the above conditions,

The EARLIEST listed time that the Main Turbine is procedurally REQUIRED to be tripped is \_\_\_\_\_\_.

IAW 31EO-EOP-011-2, RCA EOP flow chart, following the Main Turbine trip, RPV Pressure is REQUIRED to be stabilized below a MAXIMUM value of \_\_\_\_\_\_.

- A. 07:45; 845 psig
- B. 07:45; 1074 psig
- **CY** 08:00; 845 psig
- D. 08:00; 1074 psig

### 87. 295008AA2.01 001

Unit 2 was operating at 100% RTP when a scram occurred. The following conditions exist:

	<ul><li>o RPV Pressure</li><li>o RWL</li><li>o SPDS</li><li>o HPCI injecting into the RPV</li></ul>	950 psig, slowly decreasing 72 inches, slowly increasing Unavailable 4000 gpm
Ba	sed on the above conditions,	
	IAW 34AB-B21-002-2, RPV Water REQUIRED to be corrected.	Level Corrections, RWL indication
	Steps containing the operator actions located in	s to terminate ALL RPV injection, except CRD, are
A.	is; 34AB-C71-001-2, Scram Procedure	
B.	is; 34AB-C32-001-2, Reactor Water Le	evel Above +60 Inches
C.	is NOT; 34AB-C71-001-2, Scram Procedure	

34AB-C32-001-2, Reactor Water Level Above +60 Inches

Dy is NOT;

#### 88. 295009G2.4.30 001

**Unit 2** was operating at 100% RTP when a transient resulted in the following RWL values at the given times:

<u>Time</u>	$\underline{\mathbf{RWL}}$
11:30	-36 inches
12:00	-102 inches
12:30	-156 inches
13:00	-194 inches

Based on the above conditions and IAW NMP-EP-111, Emergency Notifications,

The EARLIEST listed time that a <u>condition</u> exists which requires a notification to State and Local Governments is \_\_\_\_\_\_\_.

- A. 11:30
- B. 12:00
- CY 12:30
- D. 13:00

#### 89. 295012AA2.01 001

Unit 2 is operating at 100% RTP when a loss of Drywell Cooling occurs.

- o Drywell pressure is being manually controlled between 0.5 psig and 1.2 psig
- o 34AB-T47-001-2, Complete Loss of Drywell Cooling, is in progress

At 13:00, Drywell temperatures are as follows:

2T47-N002 279°F, increasing 0.5°F/minute 200°F, increasing 0.5°F/minute

Based on the above conditions,

IAW 34AB-T47-001-2, the EARLIEST listed time that Drywell temperatures REQUIRE entry into 34GO-OPS-014-2, Fast Reactor Shutdown, is \_\_\_\_\_\_\_.

#### REFERENCE PROVIDED

- A. 13:43
- BY 14:14
- C. 14:21
- D. 14:52

#### 90. 295031EA2.01 001

Unit 1 was operating at 100% RTP when a transient resulted in the following conditions:

o	Reactor power	18% RTP
o	RWL	9 inches
o	MSIVs	CLOSED
o	Torus temperature	is above BIIT
o	RPV Pressure	controlled by LLS

Based on the above conditions and IAW 31EO-EOP-017-1, CP-3, ATWS Level Control,

The SS will direct the operator to \_\_\_\_\_\_\_.

Once a RWL band is established, if reactor power remained above 10% RTP throughout the RWL decrease, the SS will order an UPPER RWL band of \_\_\_\_\_\_\_.

- A. lower RWL to maintain between -60 inches and -90 inches with Table L-3 Systems; -60 inches
- B. lower RWL to maintain between -60 inches and -90 inches with Table L-3 Systems; -155 inches
- C. perform 31EO-EOP-113-1, Terminating and Preventing Injection Into The RPV; -60 inches
- Dr perform 31EO-EOP-113-1, Terminating and Preventing Injection Into The RPV; -155 inches

#### 91. 295037G2.4.41 001

An ATWS is in effect on **Unit 1**.

Reactor Power remains at 33% RTP.

Subsequently, initiation of Alternate Rod Insertion (ARI) results in an ALL RODS IN condition.

Based on the above conditions and IAW NMP-EP-110-GL02 - HNP EALs - ICs, Threshold Values and Basis,

Initiating ARI from the Main Control Room \_\_\_\_\_ considered a Manual Scram.

An EAL Threshold value \_\_\_\_\_ been EXCEEDED.

Ay is; has

- B. is; has NOT
- C. is NOT;
- D. is NOT; has NOT

#### 92. 295030EA2.03 001

**Unit 1** was operating at 100% RTP when an event occurred causing Torus Water level to begin decreasing.

At 10:00, the following conditions exist:

o Torus water level	130 inches decreasing 2 inches/minute
---------------------	---------------------------------------

o MSIVs Closed

o RPV Pressure controlled between 800 - 1074 psig
o RWL -65 inches increasing 1 inch/minute
o RCIC Danger Tagged out of service

o HPCI System injecting

ALL Low Pressure Injection Systems are available.

At 10:15, Maintenance isolates the leak and Torus level stabilizes at 100 inches.

Based on the above conditions,

At 10:08, HPCI is REQUIRED to be \_\_\_\_\_.

At 10:15, RPV pressure is REQUIRED to be controlled using EOP flowchart \_\_\_\_\_\_.

A. injecting to maintain Reactor water level;

RC

B. injecting to maintain Reactor water level;

CP-1 Point G

C. shutdown and prevented from operating;

RC

Dy shutdown and prevented from operating;

CP-1 Point G

**Unit 2** is operating at 100% RTP when the following occurs:

- o INSTR AIR PREFLTR D103A DIFF PRESS HIGH, 700-225, ILLUMINATED
- o INSTR AIR DRYERS SYS PRESS LOW, 700-219, ILLUMINATED
- o Prefilter D103A Differential pressure is 15 PSID
- o Non-Essential Instrument Air Header pressure is 55 psig

The following procedures are entered:

- o 34AR-700-225-2, INSTR AIR PREFLTR D103A DIFF PRESS HIGH
- o 34AB-P51-001-2, Loss Of Instrument And Service Air System Or Water Intrusion Into The Service Air System

Based on the above conditions,

The Non-Essential Instrument Air Header Isolation Valve, 2P52-F015, will \_\_\_\_\_\_.

The procedure that contains detailed guidance on how to place Prefilter D103B in service and remove Prefilter D103A from service is \_\_\_\_\_\_.

A. remain open;

34AB-P51-001-2

By remain open;

34AR-700-225-2

C. close and remain closed;

34AB-P51-001-2

D. close and remain closed;

34AR-700-225-2

#### 94. G2.1.3 001

The following Modes exist for both units at the given time:

<u>Time</u>	<u>Unit 1</u>	<u>Unit 2</u>
10:00	Mode 4	Mode 3
11:00	Mode 4	Mode 4
17:00	Mode 5	Mode 4
20:00	Mode 5	Mode 5

Based on the above conditions and IAW Tech Specs,

The EARLIEST listed time that an extra (Licensed) Reactor Operator (RO) can assume the Control Room Command Function is \_\_\_\_\_\_ .

- A. 10:05
- BY 11:05
- C. 17:05
- D. 20:05

95. G2.2.5 001

A proposed plant modificat	ion must ALWAYS	have <u>prior</u> approva	al from the NRC if it
involves any			

- A. system that requires a 50.59 screening
- B. change to any system included in Tech Specs
- C. design basis limit for Primary Containment being altered
- D. change to the Technical Requirements Manual (TRM) Bases

#### 96. G2.2.20 001

Operations has identified an issue that needs troubleshooting activities to be performed in an area posted as "PROTECTED EQUIPMENT AREA".

Based on the above conditions and IAW NMP-OS-010, Protected Train/Division and Protected Equipment Program,

The MINIMUM level of authorization REQUIRED to <u>enter</u> the area near the "PROTECTED" equipment to perform troubleshooting activities is the \_\_\_\_\_\_.

AY Shift Manager

- B. Shift Supervisor
- C. Shift Support Supervisor
- D. Shift Operations Manager

#### 97. G2.3.4 001

During a Radiological Emergency, an Operator is needed to enter an area to protect valuable equipment. The Operator will exceed their Federal exposure limit during this entry.

Based on the above conditions,

IAW 73EP-EIP-017-0, Emergency Exposure Control, the MAXIMUM Dose Limit allowed
to protect valuable property is

IAW NMP-EP-110, Emergency Classification Determination and Initial Action, the Emergency Director \_\_\_\_\_\_ REQUIRED to authorize exceeding this exposure limit.

AY 10 Rem

is

- B. 10 Rem is NOT
- C. 25 Rem is
- D. 25 Rem is NOT

Unit 2 is operating at 100% RTP.

The RB 130' N-E working area radiation levels UNEXPECTEDLY begin increasing from 2 mR/hr to the following:

Time Rad l		ad Level
11:00	7	mR/hr
11:30	25	mR/hr
12:00	70	mR/hr
12:30	125	mR/hr

IAW 73EP-RAD-001-0, Radiological Event,

The EARLIEST listed time that a Radiological Event will be declared is \_\_\_\_\_\_.

- A. 11:00
- BY 11:30
- C. 12:00
- D. 12:30

An event occurs on **Unit 2** resulting in the need to Anticipate an Emergency Depress of the RPV.

Subsequently, the event degrades to where the Severe Accident Guidelines (SAGs) are entered.

Based on the above conditions,

IAW the EOPs, to Anticipate the Emergency Depress, the Main Turbine Bypass Valves will be opened using \_\_\_\_\_\_ .

After the SAGs are entered, the EOP Flowcharts will be \_\_\_\_\_\_.

- A. 34SO-N30-001-2, Main Turbine Operation; performed concurrently
- B**Y** 34SO-N30-001-2, Main Turbine Operation; exited
- C. 31EO-EOP-107-2, Alternate RPV Pressure Control; performed concurrently
- D. 31EO-EOP-107-2, Alternate RPV Pressure Control; exited

**Unit 2** is operating at 100% RTP when a fire occurs on Startup Auxillary Transformer (SAT) 2D resulting in a large amount of smoke.

While the fire is burning, the SS will direct an operator to align the Main Control Room Environmental Control (MCREC) System to the
34AB-X43-001-2, Fire Procedure, does contain guidance to startup and tie an Emergency Diesel Generator to an affected Emergency Bus in the event of a loss of a

AY Isolation Mode;

Startup Auxiliary Transformer (SAT)

B. Isolation Mode; Unit Auxiliary Transformer (UAT)

- C. Pressurization Mode; Startup Auxiliary Transformer (SAT)
- D. Pressurization Mode; Unit Auxiliary Transformer (UAT)

# **NRC SRO REFERENCES**

# **SRO EXAM**

- 1. 34AB-E11-001-2, Loss Of Shutdown Cooling, Page 15 (Attachment 1)
- 2. U1 TS 3.3.1.1 RPS Instrumentation, Pages 3.3-1 3.3-3
- 3. U1 TS Table 3.3.1.1-1, RPS Instrumentation
- 4. 34AB-T47-001-2, Complete Loss of Drywell Cooling, Att. 1

SNC PLANT E. I. HATCH	Pg 15 of 34	
DOCUMENT TITLE:	Ver No:	
LOSS OF SHUTDOWN COOLING	34AB-E11-001-2	6.15
ATTACHMENT 1	Att. Pg.	
TITLE: CORE FUEL IN UNFLOODED RPV		1 of 2

# Case 1: CORE FUEL in UNFLOODED RPV

Days After Shutdown	Heat Load (MBTU/hr)	Saturation Time	Evaporation Time	Boil-Off Time
1.0	57.275	<b>52</b> min	<b>4</b> hr <b>50</b> min	5 hr 42 min
2.0	47.051	1 hr 4 min	<b>5</b> hr <b>53</b> min	6 hr 57 min
3.0	40.644	1 hr 14 min	6 hr 48 min	8 hr 3 min
4.0	36.112	1 hr 23 min	7 hr 40 min	9 hr 3 min
5.0	32.822	1 hr 32 min	8 hr 26 min	9 hr 58 min
6.0	30.372	1 hr 39 min	9 hr 6 min	<b>10</b> hr <b>46</b> min
7.0	28.494	1 hr 45 min	9 hr 42 min	11 hr 29 min
8.0	27.011	1 hr 51 min	<b>10</b> hr <b>14</b> min	<b>12</b> hr <b>6</b> min
9.0	25.806	1 hr 57 min	<b>10</b> hr <b>43</b> min	<b>12</b> hr <b>40</b> min
10.0	24.799	2 hr 1 min	<b>11</b> hr <b>9</b> min	13 hr 11 min
11.0	23.935	2 hr 6 min	<b>11</b> hr <b>34</b> min	13 hr 40 min
12.0	23.177	2 hr 10 min	<b>11</b> hr <b>56</b> min	<b>14</b> hr <b>7</b> min
13.0	22.501	2 hr 14 min	<b>12</b> hr <b>18</b> min	14 hr 32 min
14.0	21.887	2 hr 18 min	<b>12</b> hr <b>38</b> min	<b>14</b> hr <b>57</b> min
15.0	21.323	2 hr 21 min	<b>12</b> hr <b>59</b> min	<b>15</b> hr <b>20</b> min
16.0	20.800	2 hr 25 min	13 hr 18 min	<b>15</b> hr <b>44</b> min
17.0	20.310	2 hr 28 min	<b>13</b> hr <b>37</b> min	<b>16</b> hr <b>6</b> min
18.0	19.850	2 hr 32 min	13 hr 56 min	16 hr 29 min
19.0	19.415	2 hr 35 min	<b>14</b> hr <b>15</b> min	<b>16</b> hr <b>51</b> min
20.0	19.002	2 hr 39 min	14 hr 34 min	17 hr 13 min
21.0	18.610	2 hr 42 min	14 hr 52 min	<b>17</b> hr <b>35</b> min
22.0	18.235	2 hr 45 min	<b>15</b> hr <b>10</b> min	<b>17</b> hr <b>56</b> min
23.0	17.877	2 hr 49 min	<b>15</b> hr <b>29</b> min	<b>18</b> hr <b>18</b> min
24.0	17.535	2 hr 52 min	<b>15</b> hr <b>47</b> min	<b>18</b> hr <b>39</b> min
25.0	17.207	2 hr 55 min	16 hr 5 min	19 hr 1 min

#### 3.3 INSTRUMENTATION

### 3.3.1.1 Reactor Protection System (RPS) Instrumentation

LCO 3.3.1.1 The RPS instrumentation for each Function in Table 3.3.1.1-1 shall be OPERABLE.

APPLICABILITY: According to Table 3.3.1.1-1.

Δ	C	$\Gamma I \ell$	$\cap$	N	9
↗	$\mathbf{c}$	ı١٧	JI	IV	O

-----NOTE------

Separate Condition entry is allowed for each channel.

	CONDITION	F	REQUIRED ACTION	COMPLETION TIME
A.	One or more required channels inoperable.	A.1 Place channel in trip.  OR		12 hours
		A.2	NOTE Not applicable for Functions 2.a, 2.b, 2.c, 2.d, and 2.f.	
			Place associated trip system in trip.	12 hours
В.	Not applicable for Functions 2.a, 2.b, 2.c, 2.d, and 2.f.	B.1 <u>OR</u>	Place channel in one trip system in trip.	6 hours
	One or more Functions with one or more required channels inoperable in both trip systems.	B.2	Place one trip system in trip.	6 hours

(continued)

# ACTIONS (continued)

7 10 11	iorio (continuca)	1		
	CONDITION	F	REQUIRED ACTION	COMPLETION TIME
C.	One or more Functions with RPS trip capability not maintained.	C.1	Restore RPS trip capability.	1 hour
D.	Required Action and associated Completion Time of Condition A, B, or C not met.	D.1	Enter the Condition referenced in Table 3.3.1.1-1 for the channel.	Immediately
E.	As required by Required Action D.1 and referenced in Table 3.3.1.1-1.	E.1	Reduce THERMAL POWER to < 27.6% RTP.	4 hours
F.	As required by Required Action D.1 and referenced in Table 3.3.1.1-1.	F.1	Be in MODE 2.	6 hours
G.	As required by Required Action D.1 and referenced in Table 3.3.1.1-1.	G.1	Be in MODE 3.	12 hours
Н.	As required by Required Action D.1 and referenced in Table 3.3.1.1-1.	H.1	Initiate action to fully insert all insertable control rods in core cells containing one or more fuel assemblies.	Immediately

(continued)

#### ACTIONS (continued)

	CONDITION	R	EQUIRED ACTION	COMPLETION TIME
I.	As required by Required Action D.1 and referenced in Table 3.3.1.1-1.	I.1 Initiate alternate method to detect and suppress thermal-hydraulic instability oscillations.		12 hours
		AND		
		1.2	Restore required channels to OPERABLE.	120 days
J.	Required Action and associated Completion Time of Condition I not met.	J.1	Be in MODE 2.	4 hours

#### SURVEILLANCE REQUIREMENTS

NOTES	
110120	

- 1. Refer to Table 3.3.1.1-1 to determine which SRs apply for each RPS Function.
- 2. When a channel is placed in an inoperable status solely for performance of required Surveillances, entry into associated Conditions and Required Actions may be delayed for up to 6 hours provided the associated Function maintains RPS trip capability.

	SURVEILLANCE	FREQUENCY
SR 3.3.1.1.1	Perform CHANNEL CHECK.	In accordance with the Surveillance Frequency Control Program

(continued)

(continued)

Table 3.3.1.1-1 (page 2 of 3) Reactor Protection System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER TRIP SYSTEM	CONDITIONS REFERENCED FROM REQUIRED ACTION D.1	SURVEILLANCE REQUIREMENTS	ALLOWABLE VALUE
Average Power Range Moni (continued)	tor				
e. Two-out-of-Four Voter	1, 2	2	G		
f. OPRM Upscale	1	3(c)	1		
<ol> <li>Reactor Vessel Steam Dome Pressure - High</li> </ol>	e 1, 2	2	G		
Reactor Vessel Water Level Low, Level 3	- 1,2	2	G		
Main Steam Isolation Valve - Closure	1	8	F		
6. Drywell Pressure - High	1, 2	2	G		
7. Scram Discharge Volume Water Level - High					
a. Resistance Temperatur Detector	e 1,2	2	G		
	5(a)	2	н		
b. Float Switch	1, 2	2	G		
	5(a)	2	н		

(a) With any control rod withdrawn from a core cell containing one or more fuel assemblies.

<sup>(</sup>c) Each APRM channel provides inputs to both trip systems.

SNC PLANT E. I. HATCH		Pg 6 of 6
DOCUMENT TITLE:	DOCUMENT NUMBER:	VERSION No:
COMPLETE LOSS OF DRYWELL COOLING	34AB-T47-001-2	1.9
ATTACHMENT <u>1</u>		Att. Pg.
TITLE: PEAK DRYWELL TEMPERATURE		1 of 1

### PEAK DRYWELL TEMPERATURES

Drywell Elevation	*Temperature Elements	(°F)
Upper	2T47-N002 2T47-N010 2T47-N001A 2T47-N001K	300°F
Middle	2T47-N003 2T47-N009	240°F
Lower	2T47-N001L 2T47-N004 2T47-N007 2T47-N008	200°F

<sup>\* &</sup>lt;u>IF</u> desired, use SPDS Drywell Temperature Diagnostic Display instruments/values for the upper, middle, and lower drywell sections.

