GO2-16-104 Enclosure 1, Attachment 3

Emergency Action Level (EAL) Classification Matrix (Wallcharts)

(For information only)

		GENERAL EMERGENCY	SITE AREA EMERGENC	Y	ALERT	UNUSUAL EVENT
		Release of gaseous radioactivity resulting in offsite dose greater than 1,000 mrem TEDE or 5,000 mrem thyroid CDE	Release of gaseous radioactivity resulting in offsite dose than 100 mrem TEDE or 500 mrem thyroid CDE	e greater	Release of gaseous or liquid radioactivity resulting in offsite dose greater than 10 mrem TEDE or 50 mrem thyroid CDE	Release of gaseous or liquid radioactivity greater than 2 times the ODCM limits for 60 minutes or longer
Rad Levels / Rad Effluent		RG1.1		GTOF (2) Do do at (Note:  RA1.2  Analy: or rele or GT BOUN  RA1.3  Field s beyon	Reading on any Table 3 effluent radiation monitor of T column "ALERT" for GE 15 min.  OR  Ose assessment using actual meteorology indicates oses GT 10 mrem TEDE or GT 50 mrem thyroid CDE to result or beyond the SITE BOUNDARY os 1, 2, 3, 4)  OR  OSE 10 mrem TEDE or GT 50 mrem thyroid CDE to result of the site of the following at or and the SITE BOUNDARY:  Closed window dose rates GT 10 mR/hr expected to continue for GE 60 min.  Analyses of field survey samples indicate thyroid	RU1.1 1 2 3 4 5 DEF  (1) Reading on any Table 3 effluent radiation monitor GT column "UE" for GE 60 min. OR  (2) Sample analyses for a gaseous or liquid release indicates a concentration or release rate > 2 x ODCM limits for GE 60 min.  (Notes 1, 2, 3)
		Spent fuel pool level cannot be restored to at least the top of the			CDE GT 50 mrem for 60 min. of inhalation.	
	2 Irradiated Fuel Event	RG2.1 1 2 3 4 5 DEF  Spent fuel pool level cannot be restored to at least 0.5 ft for GE 60 min. (Note 1)  Table 3 Effluent Monitor  Release Point Monitor GI  Reactor Building Exhaust PRM-RE-1B (I) PRM-RE-1C (H) 2.00E+0  Turbine Building Exhaust TEA-RIS-13 8.35E-02  Radwaste Building Exhaust WEA-RIS-14 3.45E-01  Radwaste Effluent FDR-RIS-606  TSW Effluent TSW-RIS-5  Service Water Process A SW-RIS-604	6.00E+03 cps  04 cps	RA2.2 Dama radioa AN High a	overy of irradiated fuel in the REFUELING PATHWAY  2 1 2 3 4 5 DEF  age to irradiated fuel resulting in a release of eactivity  ND  alarm on any of the following radiation monitors:  ARM-RIS-1 Reactor Building Fuel Pool Area  ARM-RIS-2 Reactor Building Fuel Pool Area  ARM-RIS-34 Reactor Building Elevation 606  REA-RIS-609A-D Rx Bldg Vent	RU2.1 1 2 3 4 5 DEF  UNPLANNED water level drop in the REFUELING PATHWAY as indicated by EITHER of the following:  • SFP level LE 22.3 ft  • SFP low level alarm AND  UNPLANNED rise in area radiation levels as indicated by any of the following radiation monitors:  • ARM-RIS-1 Reactor Building Fuel Pool Area  • ARM-RIS-2 Reactor Building Fuel Pool Area  • ARM-RIS-34 Reactor Building Elevation 606
	Area Radiation Levels	Service Water Process B SW-RIS-605	1.00E+02 cps	(1) Do	1 2 3 4 5 DEF cose rates GT 15 mR/hr in Control Room ARM-RIS-19) or CAS (by survey)	
		Table 9	Safe Operation & Shutdown Rooms/Areas	tha	OR In UNPLANNED event results in radiation levels that prohibit or IMPEDE access to any Table 9 theorems or areas (Note 5)	
<b>E</b> ISFSI	<b>1</b> Confinement Boundary	RW 467' Vital Island RB 422' B RHR Pun	Room/Area Modes Applicabil  Control Room (RHR flush to RW tanks) 3  (RHR-V-9 disconnect) 3  np Rm (local pump temperatures) 3  np Rm (operate RHR-V-85B) 3	lity	None	EU1.1 Storage Operations  Damage to a loaded canister (MPC) CONFINEMENT BOUNDARY as indicated by measured dose rates on a loaded overpack GT EITHER:  • 20 mrem/hr (gamma + neutron) on the top of the overpack  • 100 mrem/hr (gamma + neutron) on the side of the overpack, excluding inlet and outlet ducts
			HOSTILE ACTION within the PROTECTED AREA		HOSTILE ACTION within the OWNER CONTROLLED AREA or airborne attack threat within 30 minutes	Confirmed SECURITY CONDITION or threat
Hazards	1 Security	None	A HOSTILE ACTION is occurring or has occurred with the PROTECTED AREA as reported by the Security Sergeant or Security Lieutenant	with by OF (2) A	HOSTILE ACTION is occurring or has occurred within the OWNER CONTROLLED AREA as reported by the Security Sergeant or Security Lieutenant	<ul> <li>HU1.1 1 2 3 4 5 DEF</li> <li>(1) A SECURITY CONDITION that does <u>not</u> involve a HOSTILE ACTION as reported by the Security Sergeant or Security Lieutenant OR</li> <li>(2) Notification of a credible security threat directed at the site OR</li> <li>(3) A validated notification from the NRC providing information of an aircraft threat</li> </ul>
	2 Seismic Event	None	None		See CA6.1/MA8.1 for potential for upgrade to an Alert based on degraded safety system performance or damage	HU2.1 1 2 3 4 5 DEF  Seismic event GT Operating Basis Earthquake (OBE) as indicated by H13.P851.S1.5-1 (OPERATING BASIS EARTHQUAKE EXCEEDED) activated
	3 Natural or Tech. Hazard	<ol> <li>The Emergency Director should declare the event promptly upon determining that time limit has been exceeded, or will likely be exceeded</li> <li>If an ongoing release is detected and the release st time is unknown, assume that the release duration lexceeded the specified time limit</li> <li>If the effluent flow past an effluent monitor is known have stopped, indicating that the release path is iso the effluent monitor reading is no longer VALID for classification purposes</li> <li>The pre-calculated effluent monitor values presente EALs RA1.1, RS1.1 and RG1.1 should be used for emergency classification assessments until the rest from a dose assessment using actual meteorology available</li> <li>If the equipment in the listed room or area was alreatinoperable or out-of-service before the event occurr then no emergency classification is warranted</li> <li>If CONTAINMENT CLOSURE is re-established prioexceeding the 30-minute time limit, declaration of a General Emergency is not required</li> </ol>	to lated, None  d in ults are ady ed, r to		See CA6.1/MA8.1 for potential for upgrade to an Alert based on degraded safety system performance or damage	HU3.1 1 2 3 4 5 DEF  (1) A tornado strike within the PROTECTED AREA OR  (2) Volcanic ash fallout requiring plant shutdown  HU3.2 1 2 3 4 5 DEF  Internal room or area FLOODING of a magnitude sufficient to require manual or automatic electrical isolation of a SAFETY SYSTEM component needed for the current operating mode  HU3.3 1 2 3 4 5 DEF  (1) Movement of personnel within the PROTECTED AREA is IMPEDED due to an offsite event involving hazardous materials (e.g., an offsite chemical spill, 618-11 event or toxic gas release)  OR  (2) A hazardous event that results in on-site conditions sufficient to prohibit the plant staff from accessing the site via personal vehicles (Note 7)
	4	<ul> <li>7 This EAL does not apply to routine traffic impedimel such as fog, snow, ice, or vehicle breakdowns or accidents</li> <li>8 A manual scram action is any operator action, or se actions, which causes the control rods to be rapidly inserted into the core, and does not include manual driving in control rods or implementation of boron in strategies</li> </ul>	t of	<ul> <li>Vital portions</li> <li>467' eleva</li> <li>487' eleva</li> <li>Main Conf</li> <li>525' eleva</li> <li>Reactor Build</li> <li>Vital portions</li> <li>DEH press</li> </ul>	Table 5 ctures Containing Safe Shutdown Systems or Components  so of the Rad Waste/Control Building: ration vital island ration cable spreading room introl Room and vertical cable chase ration HVAC area ding so of the Turbine Building ssure switches tches on turbine throttle valves	HU4.1  A FIRE is not extinguished within 15 min. of any of the following FIRE detection indications (Note 1):  Report from the field (i.e., visual observation)  Receipt of multiple (more than 1) fire alarms or indications  Field verification of a single fire alarm AND  The FIRE is located within any Table 5 area  HU4.2  1  2  3  4  5  DEF  Receipt of a single fire alarm (i.e., no other indications of a FIRE)  AND
	Fire	Table 9		<ul><li>Turbine B</li><li>Main stea</li></ul>	See CA6.1/MA8.1 for potential for upgrade to an Alert based on degraded safety system performance or damage	The fire alarm is indicating a FIRE within any Table 5 area AND  The existence of a FIRE is not verified within 30 min. of alarm receipt (Note 1)  HU4.3  1 2 3 4 5 DEF  (1) A FIRE within the ISFSI or plant PROTECTED AREA not extinguished within 60 min. of the initial report, alarm or indication (Note 1) OR  (2) A FIRE within the ISFSI or plant PROTECTED AREA that requires firefighting support by an offsite fire response agency to extinguish
	<b>5</b> Hazardous Gases	None RW 467' Vital Islan RB 422' B RHR Pu	Room/Area Modes Applicate Control Room (RHR flush to RW tanks) 3  Ind (RHR-V-9 disconnect) 3  Iump Rm (local pump temperatures) 3  Iump Rm (operate RHR-V-85B) 3	HA5.1 Release into an	ase of a toxic, corrosive, asphyxiant or flammable gas  ny Table 9 rooms or areas	
	6 Control Room Evacuation	Other conditions existing which in the judgment of the Emergency	Inability to control a key safety function from outside the Room  HS6.1  1 2 3 4 5  An event has resulted in plant control being transferred the Control Room to the Remote Shutdown Panel or Alternate Remote Shutdown Panel AND  Control of any of the following key safety functions is preestablished within 15 min. (Note 1):  Reactivity (Modes 1 and 2 only)  RPV water level RCS heat removal	Control  HA6.1  An every from the Alternation of	Control Room evacuation resulting in transfer of plant control to alternate locations  1	Other conditions existing which in the judgment of the
	<b>7</b> Judgment	HG7.1 1 2 3 4 5 DEF  Other conditions exist which, in the judgment of the Emergency Director, indicate that events are in progress or have occurred which involve actual or IMMINENT substantial core degradation or melting with potential for loss of containment integrity or HOSTILE ACTION that results in an actual loss of physical control of the facility. Releases can be reasonably expected to exceed EPA Protective Action Guideline exposure levels offsite for more than the immediate site area.	Director warrant declaration of Site Area Emergency  HS7.1  1  2  3  4  5  DEF  Other conditions exist which, in the judgment of the Emergency Director, indicate that events are in progreshave occurred which involve actual or likely major failst plant functions needed for protection of the public or HOSTILE ACTION that results in intentional damage of malicious acts, (1) toward site personnel or equipment could lead to the likely failure of or, (2) that prevent effect access to equipment needed for the protection of the plant releases are not expected to result in exposure leading to the SITE BOUNDARY.	ess or ures of have of substant to site ective public. expose	In the judgment of the regency Director, indicate that events are in progress or exocurred which involve an actual or potential tantial degradation of the level of safety of the plant or curity event that involves probable life threatening risk be personnel or damage to site equipment because of ITILE ACTION. Any releases are expected to be limited that fractions of the EPA Protective Action Guideline issure levels.	HU7.1  1 2 3 4 5 DEF  Other conditions exist which, in the judgment of the Emergency Director, indicate that events are in progress or have occurred which indicate a potential degradation of the level of safety of the plant or indicate a security threat to facility protection has been initiated. No releases of radioactive material requiring offsite response or monitoring are expected unless further degradation of SAFETY SYSTEMS occurs.
Мо	des:	1 2 3 Power Operations Startup Hot Shutdown		<b>EF</b>	ENERGY NORTHWEST	13.1.1A EAL Classification Matrix Page 1 of 3 ALL CONDITIONS

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		GENERAL EME	RGENCY	SITE F	AREA EMERGENCY	ALERT		UNL	ISUAL EVENT										
		Prolonged loss of <u>all</u> offsite and <u>all</u> onsite AC power to emergency buses		Loss of <u>all</u> offsite and <u>all</u> onsite AC power to emergency buses for 15 minutes or longer		Loss of <u>all</u> <b>but one</b> AC power source to emergency buses for 15 minutes or longer		Loss of <u>all</u> offsite AC power capability to emergency buses for 15 minutes or longer											
	Loss of Emergency AC Power	emergency buses SM-7 and SM-8  AND EITHER:  Restoration of emergency bus SM-7 or SM-8 in LT 4 hours is not likely (Note 1)  OR  RPV level cannot be restored and maintained GT -186 in.  Loss of all emergency AC and vital DC power sources for 15 minutes or longer		and all onsite AC power capability to ss SM-7 and SM-8 for GE 15 min. (Note 1)	MA1.1 1 2 3  AC power capability, Table 2, to emergency buses SM-7 and SM-8 reduced to a single power source for GE 15 min. (Note 1)  AND		MU1.1 1 2 3  Loss of <u>all</u> offsite AC power capability, Table 2, to emergency buses SM-7 and SM-8 for GE 15 min. (Note 1)  Table 2 AC Power Sources												
						Any additional single power source failur of all AC power to emergency buses SM			Offsite Transformer TR-S										
	AOTOWE								<ul> <li>Backup Transformer TR-B</li> <li>Backfeed 500 KV power through Main Transformers (if already</li> </ul>										
	Loss of Vital DC Power	Loss of <u>all</u> offsite AND <u>all</u> onsite AC power capability to emergency buses SM-7 and SM-8 for GE 15 min. (Note 1)						aligned in modes 4, 5, def only)  Onsite  DG1											
		AND Indicated voltage is LT 108 VDC on <u>both</u> 125 VDC buses DP-S1-1 and DP-S1-2 for GE 15 min. (Note 1)		Loss of <u>all</u> vital DC power for 15 minutes or longer  MS2.1  1 2 3 Indicated voltage is LT 108 VDC on <u>both</u> 125 VDC buses DP-S1-1 and DP-S1-2 for GE 15 min. (Note 1)		None	Nana		DG2     Main Generator via TR-N1/N2  None										
						UNPLANNED loss of Control Room indications for 15 minutes or		UNPLANNED loss of Control Room indications for 15 minutes or											
						Ionger with a significant transient in progress  MA3.1		MU3.1 1 2 3											
	Loss of Control	None		None		An UNPLANNED event results in the inability to monitor one or more Table 10 parameters from within the Control Room for GE 15 min. (Note 1)  AND  Any Table 11 transient event in progress		An UNPLANNED event results in the inability to monitor one or more Table 10 parameters from within the Control Room for GE 15 min. (Note 1)											
	Room Indications																		
		None  • Vital po  - 467  - 487  - Ma  - 525  • Reactor		Table 5 Int Structures Containing Safe Shutdown Systems or Components  Doortions of the Rad Waste/Control Building:  167' elevation vital island  167' elevation cable spreading room  167' elevation Room and vertical cable chase  165' elevation HVAC area  167' to elevation and vertical cable chase  168' elevation HVAC area  169 to Building  169 to ortions of the Turbine Building  169 EH pressure switches  169 PS switches on turbine throttle valves  160 ain steam line radiation monitors  160 ain steam line piping up to MS-V-146 and the first stop valves  169 Service Water Pump Houses		<ul> <li>Table 10 Safety System Parameters</li> <li>Reactor power</li> <li>RPV level</li> <li>RPV pressure</li> <li>Primary containment pressure</li> <li>Wetwell level</li> <li>Wetwell temperature</li> </ul> Table 11 Transient Events <ul> <li>Reactor scram</li> </ul>		Reactor coolant allowable limits	activity greater than Technical Specification										
	RCS Activity							MU4.1 1 2 3 SJAE CONDSR OUTLET RAD HI-HI alarm (P602)											
								MU4.2 1 2 3 Coolant activity GT 0.2 μCi/gm dose equivalent I-131											
M System		- DE - RP - Ma - Tu None - Ma				Runback GT 25% thermat power     Electrical load rejection G			15 minutes or longer										
Malfunct.	5 RCS					electrical load  ECCS injection  Thermal power oscillation	ns GT 10%	<ul> <li>(1) RCS unidentified or pressure boundary leakage         GT 10 gpm for GE 15 min.         OR</li> <li>(2) RCS identified leakage GT 25 gpm for GE 15 min.</li> </ul>											
	Leakage		• Diesel	el Generator Building				OR (3) Leakage from the RCS to a location outside containment GT 25 gpm for GE 15 min.											
				Inability to shut down the reactor causing a challenge to RPV water level or RCS heat removal  MS6.1  1  2  An automatic OR manual scram fails to shut down the reactor		Automatic or manual scram fails to shut down the reactor, and subsequent manual actions taken at the reactor control consoles are not successful in shutting down the reactor  MA6.1  1  2  An automatic OR manual scram fails to shut down the reactor		Automatic or manual scram fails to shut down the reactor  MU6.1 1 2											
	6 RPS Failure	indicated I		All actions to shu	ut down the reactor are <u>not</u> successful as ctor power GT 5% R:	(mode switch in shutdown, manual push	Manual scram actions taken at the reactor control console (mode switch in shutdown, manual push buttons or ARI) are not successful in shutting down the reactor as indicated by		AND  A subsequent automatic scram OR manual scram action taken at the reactor control console (mode switch in shutdown, manual push buttons or ARI) is successful in										
		Table 4 Commun	ication Methods	above -18 OR	cannot be restored and maintained 66 in. or cannot be determined erature and RPV pressure cannot be	reactor power GT 5% (Note 8)	ŕ	_	actor as indicated by reactor power LE										
		Plant Public Address (PA) System X  Plant Telephone System X  Plant Radio System Operations and X  Security Channels			d below the HCTL														
	<b>7</b> Loss of	Security Channels  Offsite calling capability from the Control Room via direct telephone and fax lines  Long distance calling capability on the commercial phone system							or offsite communications capabilities										
					None	None		(1) Loss of <u>all</u> Table 4 onsite communication methods OR											
	Comm.	the confinercial phone system						(2) Loss of <u>all</u> Table 4 ORO communication methods OR (3) Loss of <u>all</u> Table 4 NRC communication methods											
Hazardous Event Affecting Safety Systems		Seismic eve Internal or e High winds Tornado stri FIRE EXPLOSION Volcanic asi Other event		external FLOODING event strike ON		Hazardous event affecting a SAFETY SYSTEM needed for the current operating mode  MA8.1  1 2 3 The occurrence of any Table 8 hazardous event AND EITHER:  Event damage has caused indications of degraded performance in at least one train of a SAFETY SYSTEM needed for the current operating mode OR  The event has caused VISIBLE DAMAGE to a SAFETY SYSTEM component or structure, Table 5, needed for the current operating mode  FA1.1  1 2 3  The occurrence of any Table 8 hazardous event AND EITHER:  Event damage has caused indications of degraded performance in at least one train of a SAFETY SYSTEM component or structure, Table 5, needed for the current operating mode		None											
										Fission Product Barrier Degradation					loss of <u>any</u> two barriers (Table F-1)	Any loss or any potential loss of EITHER Fuel Clad or RCS barrier (Table F-1)		None	
													Т	able F-1	Fission Product Bar	rier Threshold Matrix	,		
												FC - Fuel	Clad Barrier Potential	Loss	RCS - Reactor Coo	lant System Barrier  Potential Loss	Lo	PC - Containn	nent Barrier Potential Loss
A RPV Water Level  B RCS Leak Rate		SAG entry required	RPV level <u>cannot</u> be res maintained GT -161 in. or <u>cannot</u> be determine		RPV level <u>cannot</u> be restored and maintained GT -161 in. or <u>cannot</u> be determined.	None	None		SAG entry required										
						UNISOLABLE primary system leakage that results in exceeding EITHER:	UNISOLABLE primary system leakage that results in exceeding EITHER:  RB area maximum safe operating temperature (PPM 5.3.1 Table 23)  OR  RB area maximum safe operating radiation (PPM 5.3.1 Table 24)												
		None				RB area temperature alarm level (PPM 5.3.1 Table 23) OR RB area radiation alarm level (PPM			None										
						5.3.1 Table 24)													
C PC Conditions		None None  Containment Radiation Monitor					UNPLANNED rapid drop in PC pressure following PC pressure rise		PC pressure GT 45 psig OR Explosive mixture exists inside PC										
					PC pressure GT 1.68 psig due to RCS leakage	None	OR PC pressure response not consistent with LOCA conditions		$(H_2 \text{ GE } 6\% \text{ and } O_2 \text{ GE } 5\%)$ OR WW temperature and RPV pressure										
									cannot be maintained below the HCTL										
D PC Rad / RCS Activity		CMS-RIS-27E or CMS-RIS-27F reading GT 3,600 R/hr OR	None		Containment Radiation Monitor CMS-RIS-27E or CMS-RIS-27F reading GT 70 R/hr	None	N	one	Containment Radiation Monitor CMS-RIS-27E or CMS-RIS-27F reading GT 14,000 R/hr										
		Primary coolant activity GT 300 μCi/gm Dose Equivalent I-131					UNISOLABLE direct												
E PC Integrity or Bypass		None	None		None	None	pathway to the environment exists after PC isolation signal OR Intentional PC venting per EOPs		None										
F		Any condition in the opinion of the	· · · · · · · · · · · · · · · · · · ·		Any condition in the opinion of the Emergency Director that indicates loss	Any condition in the opinion of the Any condition in the		opinion of the	Any condition in the opinion of the Emergency Director that indicates										
Emergency Director Judgment		Emergency Director that indicates loss of the fuel clad barrier			Emergency Director that indicates loss of the RCS barrier	Emergency Director that indicates potential loss of the RCS barrier	Emergency Director that indicates loss Emergency Dir		potential loss of the Containment										

Modes:

Power Operations

2 Startup

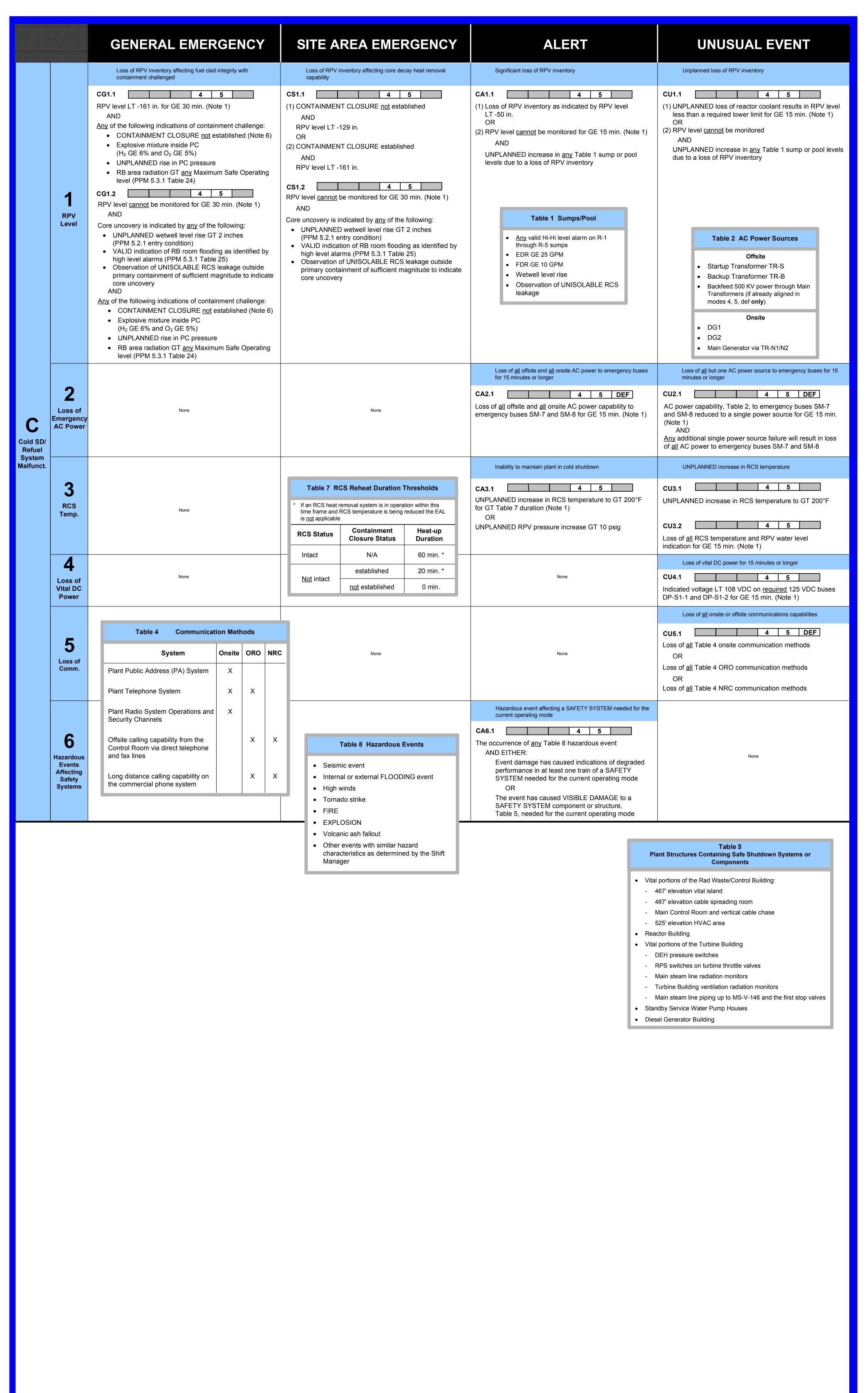
p Hot Shutdown

n Cold Shutdown

**5**Refueling

**DEF**Defueled





Modes:

Power Operations

Startup

Hot Shutdown

3 4 Cold Shutdown

**5**Refueling

**DEF**Defueled



13.1.1A

EAL Classification Matrix
Page 3 of 3

COLD CONDITIONS

(RCS ≤ 200°F)