

**CERTIFICATE OF COMPLIANCE
FOR RADIOACTIVE MATERIAL PACKAGES**

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2. PREAMBLE

- a. This certificate is issued to certify that the package (packaging and contents) described in Item 5 below meets the applicable safety standards set forth in Title 10, Code of Federal Regulations, Part 71, "Packaging and Transportation of Radioactive Material."
- b. This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or other applicable regulatory agencies, including the government of any country through or into which the package will be transported.

3. THIS CERTIFICATE IS ISSUED ON THE BASIS OF A SAFETY ANALYSIS REPORT OF THE PACKAGE DESIGN OR APPLICATION

- | | |
|---|---|
| a. ISSUED TO (<i>Name and Address</i>)
AREVA Federal Services LLC
505 S. 336 th Street, Suite 400
Federal Way, WA 98003 | b. TITLE AND IDENTIFICATION OF REPORT OR APPLICATION
AREVA Federal Services LLC
application dated May 30, 2016. |
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4. CONDITIONS

This certificate is conditional upon fulfilling the requirements of 10 CFR Part 71, as applicable, and the conditions specified below.

5.

(a) Packaging

- (1) Model No.: BEA Research Reactor (BRR) Package
- (2) Description

The purpose of the Model No. BRR package is to transport irradiated fuel elements or loose plates of a square fuel element from various test and research reactors. The package is comprised of a lead-shielded package body, payload basket, square loose plate box, an upper shield plug, a closure lid, upper and lower impact limiters, and utilizes American Society for Testing and Materials (ASTM) Type 304 stainless steel as its primary structural material. The package is a right circular cylinder with a dimension of 77.1 inches in length and 38 inches in diameter, not including the impact limiter attachments and the thermal shield. Lead shielding is located between two circular shells, in the lower end structure, and in the shield plug. The payload cavity has a diameter of 16 inches and a length of 54 inches.

Impact Limiters. Impact limiters are attached to each end of the package body. Each impact limiter is 78 inches in diameter and 34.6 inches in length, with a 15-inches long conical section towards the outer end. The impact limiter design consists of ASTM Type 304 stainless steel shells and polyurethane foam with an approximate density of 9 pounds per cubic foot (lb/ft³).

Fuel Baskets. There are five baskets used with the package, one for each type of fuel transported. The baskets are made from welded construction using ASTM Type 304 stainless steel in plate, bar, pipe, and tubular forms. Each basket has a diameter of 15.63 inches and a length of 53.45 inches, and features a number of cavities that fit the size and shape of the fuel. The basket for square fuel accommodates two types of fuel assembly: (1) flat-type fuels and (2) a 5x5 array of fuel rods enclosed within a rectangular can.

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5.(a) Packaging – Description (continued)

Spacer Pedestals. For fuel elements or assemblies shorter than the length of a basket cavity, spacer pedestals are used in each cavity, as required, to support the fuel elements at the top of the basket. All spacer pedestals are made of stainless steel

Square Box or Loose Plate Box. A square box accommodates square fuel loose plates. A loose plate box is used to transport up to 31 loose plates per box. The square fuel basket and loose plate box are made of stainless steel.

The package is designed to be transported as one package per conveyance, with its longitudinal axis vertical, by highway truck or by rail in exclusive use. When loaded and prepared for transport, the package is 119.5 inches in length, 78 inches in diameter (over the impact limiters), and weighs 32,000 pounds (lb).

(3) Drawings

The packaging is constructed in accordance with AREVA Federal Services LLC drawings:

- 1910-01-01-SAR, "BRR Package Assembly SAR Drawing," Sheets 1-4, Rev. 4
- 1910-01-02-SAR, "BRR Package Impact Limiter SAR Drawing," Sheets 1-2, Rev. 1
- 1910-01-03-SAR, "BRR Package Fuel Baskets SAR Drawing," Sheets 1-4, Rev. 6

(b) Contents

(1) Type and form of material

- (i) *Irradiated MURR Fuel Element.* Irradiated University of Missouri Research Reactor (MURR) fuel element to a maximum burnup of 180 megawatt-day (MWD) or a depletion of 30.9% of Uranium-235 (²³⁵U). The minimum cooling time is 180 days after reactor shutdown. Each MURR element contains 24 fuel plates. Each fresh MURR fuel element contains 775.0 ± 7.8 g ²³⁵U. The enrichment range is 93 ± 1 wt.% ²³⁵U. The MURR element overall length, including irradiation growth, is 32.75 inches. The maximum decay heat per fuel element is 158 watts (W). The maximum number of fuel elements per basket is 8. The bounding weight of one element is 15 lb. Table 1.1 includes characteristics of a pre-irradiated MURR fuel element.

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5.(b)(1) (i) Type and form of material - *Irradiated MURR Fuel Element* (continued)

Table 1.1. MURR - Key Fuel Element Parameters

<i>Maximum active fuel length (inches)</i>	24.8
<i>Overall length (inches)</i>	32.75
<i>Minimum cladding thickness (inch)</i>	0.008
<i>Nominal fuel matrix thickness (inch)</i>	0.02
<i>Fuel matrix</i>	UAl _x
<i>Cladding material</i>	Aluminum
<i>Maximum ²³⁵U per element (g)</i>	782.8
<i>Maximum enrichment (wt.%)</i>	94.0
<i>Maximum ²³⁵U per fuel plate (g)</i>	46.0

(ii) *Irradiated MITR-II Fuel Element.* Irradiated Massachusetts Institute of Technology Research Reactor (MITR-II) fuel element to a maximum burnup of 165 MWD or a ²³⁵U depletion of 43.9%. The minimum cooling time is 120 days after reactor shutdown. Each MITR-II element contains 15 fuel plates. Each fresh MITR-II element contains 510.0 +3.0/-10.0 g ²³⁵U, which is 500 - 513 g ²³⁵U. The enrichment range is 93 ±1 wt.% ²³⁵U. The MITR-II element overall length, including irradiation growth, is 26.52 inches. The maximum decay heat per element is 150 W. The maximum number of fuel elements per basket is 8. The bounding weight of one element is 10 lb. Table 1.2 includes the key parameters for a pre-irradiated MITR-II fuel element.

Table 1.2 MITR-II - Key Fuel Element Parameters

<i>Maximum active fuel length (inches)</i>	22.76
<i>Overall length (inches)</i>	26.52
<i>Minimum cladding thickness (inch)</i>	0.008
<i>Nominal fuel matrix thickness (inch)</i>	0.03
<i>Maximum fuel matrix width (inches)</i>	2.171
<i>Fuel matrix</i>	UAl _x
<i>Cladding material</i>	Aluminum
<i>Maximum ²³⁵U per element (g)</i>	513
<i>Maximum enrichment (wt.%)</i>	94.0
<i>Maximum ²³⁵U per fuel plate (g)</i>	34.3

(iii) *Irradiated ATR Fuel Element.* Irradiated Advanced Test Reactor (ATR) fuel element to a maximum burnup of 480 MWD or a ²³⁵U depletion of 58.6%. The minimum cooling time is 1,670 days (4.6 years) after reactor shutdown. Each ATR fuel element contains 19 plates. The YA fuel element has 19 plates, but only 18 contain fuel.

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5.(b)(1) (iii) Type and form of material - *Irradiated ATR Fuel Element* (continued)

There are two general classes of ATR fuel element, XA and YA. The enrichment range is 93 ± 1 wt.% ^{235}U . The XA fuel element has a fresh fuel loading of $1,075 \pm 10$ g ^{235}U . The YA fuel element has a fresh fuel loading of $1,022.4 \pm 10$ g ^{235}U . A second YA fuel element design (YA-M) has the side plate width reduced by 15 mils. The ATR element overall maximum length, after removal of the end box structures, 51.0 inches. The maximum number of fuel elements per basket is 8. The bounding weight of one element is 25 lb. The maximum decay heat per element is 30 W. Table 1.3 includes characteristics of a pre-irradiated ATR fuel element.

Table 1.3. ATR - Key Fuel Element Parameters

<i>Maximum active fuel length (inches)</i>	48.77
<i>Overall length (inches)</i>	51
<i>Minimum cladding thickness for Plate 1 (inch)</i>	0.018
<i>Minimum cladding thickness for Plates 2-18 (inch)</i>	0.008
<i>Minimum cladding thickness for Plate 19 (inch)</i>	0.018
<i>Nominal fuel matrix thickness (inch)</i>	0.02
<i>Fuel matrix</i>	UAl _x
<i>Cladding material</i>	Aluminum
<i>Maximum ^{235}U per element (g)</i>	1,085
<i>Maximum enrichment (wt.%)</i>	94.0
<i>Maximum ^{235}U per fuel plate (g)</i>	85.2

(iv) *Irradiated TRIGA fuel elements*. Table 1.4 includes the dimensions of pre-irradiated Training, Research, Isotopes, General Atomics (TRIGA) fuel elements. The TRIGA fuel matrix is uranium mixed with zirconium hydride. The BRR package is limited to the transportation of the following types of TRIGA fuel:

1. Standard 100 series.
2. Instrumented 200 series. The fuel region is as the same as 100 series but contain thermocouples used to measure temperature during reactor operation. Instrumented rods may be longer than 100 series.
3. Fueled Follower Control Rods (FFCR) (300 series). The rods contain boron carbide neutron absorber outside the active fuel region.
4. Cluster Rods (400 series). It is typically built with three or four cluster rods to make a cluster assembly.
5. Instrumented Cluster Rods (500 series). Fuel is the same as cluster rod but thermocouples used to measure temperature during reactor operation. Instrumented cluster rods may be longer.

Cluster rods (i.e., TRIGA fuel series 400 and 500) must be disassembled from the cluster assembly for transport in the BRR package.

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5.(b)(1) (iv) Type and form of material - *Irradiated TRIGA fuel elements* (continued)

Table 1.4. Characteristics of Pre-Irradiated TRIGA Fuel

Type	ID ¹	Cladding	Fuel Length (in.)	U (wt. % Fuel)	²³⁵ U (wt. %)	U (g)	²³⁵ U (g)	Fuel OD ² (in.)	Rod OD (in.)	Cladding Thickness (in.)	H/Zr	Overall Length ³ (in.)	Erbium (wt. %)
Standard 100 series	101	Aluminum	14	8.0	20	166	32	1.41	1.48	0.03	1.0	28.62	0
	101		15	8.5	20	189	37	1.41	1.48	0.03	1.6	28.62	0
	103	Stainless Steel	15	8.5	20	197	39	1.44	1.48	0.02	1.6	29.15	0
	105		15	12	20	285	56	1.44	1.48	0.02	1.6	29.15	0
	107		15	12	20	271	53	1.4	1.48	0.02	1.6	30.14	0
	109		15	8.5	70	194	136	1.44	1.48	0.02	1.6	29.15	1.2
	117		15	20	20	503	99	1.44	1.48	0.02	1.6	29.93	0.5
	119		15	30	20	825	163	1.44	1.48	0.02	1.6	29.93	0.9
Instrumented 200 series	201	Aluminum	15	8.5	20	189	37	1.44	1.48	0.03	1.6	28.78	0
	203	Stainless Steel	15	8.5	20	197	39	1.44	1.48	0.02	1.6	45.5	0
	205		15	12	20	285	56	1.44	1.48	0.02	1.6	45.5	0
	207		15	12	20	271	53	1.4	1.48	0.02	1.6	45.5	0
	217		15	20	20	503	99	1.44	1.48	0.02	1.6	40.35	0.5
	219		15	30	20	825	163	1.44	1.48	0.02	1.6	40.35	0.9
Fueled Follower Control Rods (FFCR) (300 series)	303	Stainless Steel	15	8.5	20	163	32	1.31	1.35	0.02	1.6	44	0
	305		15	12	20	237	47	1.31	1.35	0.02	1.6	44	0
	317		15	20	20	418	82	1.31	1.35	0.02	1.6	44	0.5
	319		15	30	20	685	135	1.31	1.35	0.02	1.6	44	0.9
Cluster rods (400 series)	403	Stainless Steel	15	8.5	20	166	33	1.37	1.41	0.02	1.6	30.38	0
	405		15	12	20	243	48	1.37	1.41	0.02	1.6	30.38	0
	417		15	20	20	427	85	1.37	1.41	0.02	1.6	30.38	0.5
	419		15	30	20	710	141	1.37	1.41	0.02	1.6	30.38	0.9
Instrumented cluster rods (500 series)	503	Stainless Steel	15	8.5	20	166	33	1.34	1.41	0.02	1.6	45.5	0
	505		15	12	20	243	48	1.34	1.41	0.02	1.6	45.5	0
	517		15	20	20	427	85	1.34	1.41	0.02	1.6	45.5	0.5
	519		15	30	20	710	141	1.34	1.41	0.02	1.6	45.5	0.9

¹ General Atomics catalog numbers are not necessarily unique. TRIGA elements with the same ID could have different fuel parameters. Table 1.4 includes two variants of the Type 101 element.

² Outer Diameter.

³ Overall length includes 0.25 inches for irradiation growth.

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5.(b)(1) (iv) Type and form of material - *Irradiated TRIGA fuel elements* (continued)

The maximum length of a TRIGA fuel element, including irradiation growth, is 45.50 inches. For all fuel elements, stainless steel spacers are utilized within the TRIGA baskets. The bounding weight of any TRIGA fuel element is 10 lb. The maximum decay heat per element is 20 W. The number of TRIGA rods per element is 1. Table 1.5 includes parameters for irradiated TRIGA fuel.

Table 1.5 Maximum Burnup and Minimum Cooling Time for TRIGA Fuel Elements⁴

TRIGA Fuel Type (Enrichment)	Maximum Burnup (MWD)	Minimum Cooling Time (days)
101 (8.0%)	23	90
201/101 (8.5%)	26	90
109	88	350
	70	250
	52	170
	34	90
203/103	27	90
205/105	39	120
	33	90
207/107	38	120
	33	90
217/117	71	280
	52	180
	34	90
219/119	122	600
	91	370
	63	220
	34	90
303	22	90
305	32	90
317	58	210
	46	150
	34	90
319	97	420
	76	290
	55	180
	34	90
503/403	23	90
505/405	33	90
517/417	60	220
	47	150
	34	90
519/419	101	430
	79	290
	56	180
	34	90

⁴ Based on an in-core residence time of 4 years resulting on a decay heat less than or equal to 20 W. Not applicable to fuel with an in-core residence time less than 4 years with a decay heat greater than 20 W.

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5.(b)(1) Type and form of material (continued)

- (v) *PULSTAR Fuel*. Table 1.6 includes the characteristics of the PULSTAR fuel. A 5x5 array of fuel rods enclosed within a rectangular can. Each fuel rod contains cylindrical uranium oxide fuel pellets. The weight of a PULSTAR element is 48 lb, including a spacer pedestal. The maximum heat load of the square fuel basket per compartment is 30 W.

Table 1.6. Characteristics of PULSTAR Fuel

Parameter	Value
<i>Nominal ²³⁵U Enrichment (%)</i>	4.0/6.0
<i>Fuel matrix</i>	UO ₂
<i>Maximum burnup (MWD/MTU)</i>	20,000
<i>Decay time (years)</i>	1.5
<i>Maximum fuel pellet diameter (in.)</i>	0.423
<i>Minimum cladding thickness (in.)</i>	0.0185
<i>Cladding material</i>	Zirconium alloy
<i>Maximum cladding OD (in.)</i>	0.474
<i>Maximum active fuel length (in.)</i>	24.1
<i>Fuel rod pitch X (in.)</i>	0.607
<i>Fuel rod pitch Y (in.)</i>	0.525
<i>Box outer dimensions (in.)</i>	3.15 x 2.74
<i>Box thickness (in.)</i>	0.06
<i>Box material</i>	Zirconium alloy
<i>Maximum overall length (in.)</i> ^①	38.23

Note: Maximum length includes 0.25 in. for irradiation growth.

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5.(b)(1) Type and form of material (continued)

- (vi) *Square Fuel and Loose Plates (excluding PULSTAR)*. Table 1.7 includes the main characteristics of square fuel and square-loose-plate fuel. These types of fuel have a square, or nearly square-rectangular cross section. The flat-type fuels consist of either a uranium-oxide dispersion or uranium-silicide dispersion meat in an aluminum matrix, bonded with an aluminum alloy cladding. The maximum heat load of the square fuel basket per compartment is 30 W.

Table 1.7. Square Plate Fuel Characteristics

Parameter	RINSC	Ohio State	Miss. S&T	U-Florida	Purdue	U-Mass (Al)	U-Mass (Si)
²³⁵ U loading (g)	275±7.7	200±5.6	225±6.3	175±4.9	129.92±2.52	167±3.3	200±5.6
Nominal ²³⁵ U enrichment (%)	19.75	19.75	19.75	19.75	19.75	19.75	19.75
Fuel matrix	U ₃ Si ₂ +Al	U ₃ Si ₂ +Al	U ₃ Si ₂ +Al	U ₃ Si ₂ +Al	U ₃ Si ₂ +Al	UAl _x	U ₃ Si ₂ +Al
Maximum burnup per fuel element (MWD)	52.5	64.0	74.0	87.0	0.57	9.7	9.7
Minimum decay time (D)	120	120	365	120	120	1,000	1,000
Nominal fuel meat width (in.)	2.395	2.395	2.395	2.395	2.395	2.320	2.395
Nominal fuel meat thickness (in.)	0.02	0.02	0.02	0.02	0.02	0.03	0.02
Nominal fuel plate thickness (in.)	0.05	0.05	0.05	0.05	0.05	0.06	0.05
Nominal active fuel length (in.)	23.25	23.25	23.25	23.25	23.25	23.25	23.25
Number of fuel plates	22	16	18	14	14	18	16
Maximum channel spacing (in.)	0.099	0.127	0.139	0.117	0.175	0.119	0.122
Weight (lb)	14	12	14	10	10	12	12
Maximum overall length (in.) ^④	39.75	35.25	34.50	27.38	32.49	39.75	39.75
Maximum cross section (in.)	3.097×3.097	3.05×3.05	3.036×3.212	2.9×2.424	3.011×3.011	3.097×3.097	3.097×3.097
Loose plate ^{④⑤}	no	no	no	yes ^②	yes ^③	yes ^①	no

Notes:

- U-Mass (Al) loose plates have a ²³⁵U loading of 9.28 ± 0.18g and dimensions of 2.78 inches wide by 24.88 inches long.
- U-Florida loose plates have a ²³⁵U loading of 12.5 ± 0.35g and dimensions of 2.85 inches wide by 25.88 inches long.
- Purdue loose plates have a ²³⁵U loading of 9.28 ± 0.18g and dimensions of 2.85 inches wide by 25.88 inches long.
- Maximum length includes 0.25 inches for irradiation growth.
- Loose plates shall be extracted from fuel elements that meet the per-element burnup limits provided in this table.

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5.(b)(2) Maximum quantity of material per package

(i) For the contents described in 5(b)(1)(i):

8 irradiated MURR fuel elements. Only one fuel element is allowed per basket location.

(ii) For the contents described in 5(b)(1)(ii):

8 irradiated MITR-II fuel elements. Only one fuel element is allowed per basket location.

(iii) For the contents described in 5(b)(1)(iii):

8 irradiated ATR fuel elements. Only one fuel element is allowed per basket location.

(iv) For the contents described in 5(b)(1)(iv):

19 irradiated TRIGA fuel elements. Only one fuel element is allowed per basket location.

26 types of TRIGA fuel.

(v) For the contents described in 5(b)(1)(v)

8 irradiated PULSTAR fuel elements. Only one fuel element is allowed per basket location.

(vi) For the contents described in 5(b)(1)(vi)

8 irradiated square fuel elements or loose plate boxes. Only one fuel element or loose plate box is allowed per basket location (i.e., compartment). Up to 31 loose plates may be placed in each loose plate box.

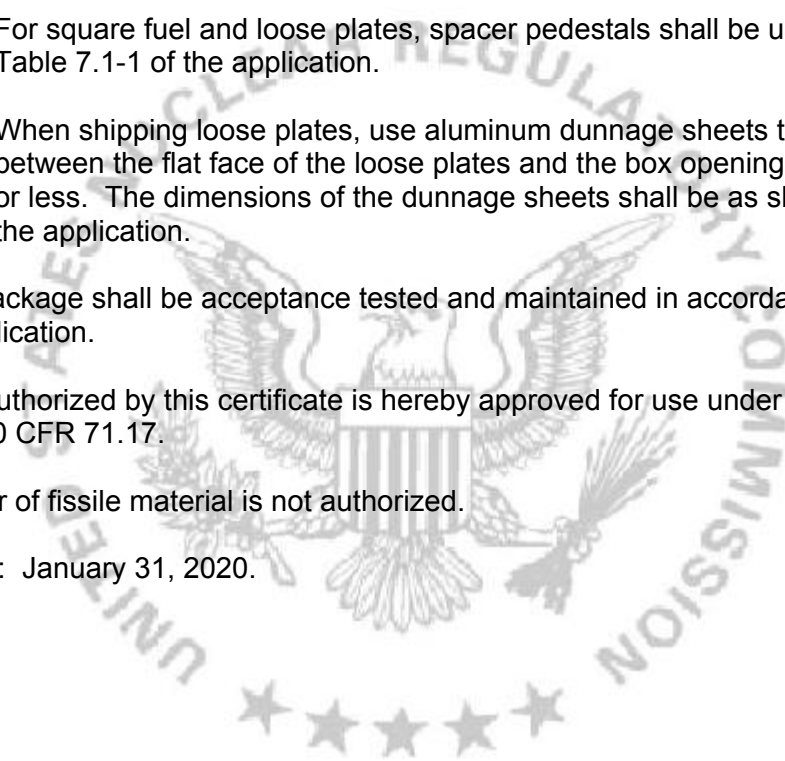
(vii) *Plutonium Quantity.* The maximum quantity of plutonium in the BRR package is 6,500 Ci (at 4% ²³⁵U enrichment of PULSTAR fuel).

(c) Criticality Safety Index (CSI): 0

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6. In addition to the requirements of Subpart G of 10 CFR Part 71:
- (a) Each package shall be operated and prepared for shipment in accordance with Chapter 7 of the application, as supplemented
 - (i) For TRIGA fuel, spacer pedestals shall be used as described in Table 7.1-2 of the application.
 - (ii) For PULSTAR fuel, spacer pedestals shall be used as described in Table 7.1-1 of the application.
 - (iii) For square fuel and loose plates, spacer pedestals shall be used as described in Table 7.1-1 of the application.
 - (iv) When shipping loose plates, use aluminum dunnage sheets to reduce the free space between the flat face of the loose plates and the box opening to a value of $\frac{1}{4}$ inches or less. The dimensions of the dunnage sheets shall be as shown in Figure 7.1-1 of the application.
 - (b) Each package shall be acceptance tested and maintained in accordance with Chapter 8 of the application.
7. The package authorized by this certificate is hereby approved for use under the general license provisions of 10 CFR 71.17.
8. Transport by air of fissile material is not authorized.
9. Expiration date: January 31, 2020.



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REFERENCES

AREVA Federal Services LLC application dated May 30, 2016. (Model No. BRR Safety Analysis Report, Revision 10)

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

/RA/

John McKirgan, Chief
Spent Fuel Licensing Branch
Division of Spent Fuel Management
Office of Nuclear Material Safety
and Safeguards

Date: July 21, 2016

