



Entergy Nuclear Operations, Inc.
Pilgrim Nuclear Power Station
600 Rocky Hill Road
Plymouth, MA 02360

July 11, 2016

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555-0001

SUBJECT: Licensee Event Report 2016-003-00, Spent Fuel Storage Design Feature Exceeded

Pilgrim Nuclear Power Station
Docket No. 50-293
Renewed License No. DPR-35

LETTER NUMBER: 2.16.037

Dear Sir or Madam:

The attached Licensee Event Report 2016-003-00, Spent Fuel Storage Design Feature Exceeded, is submitted in accordance with 10 Code of Federal Regulations 50.73.

If you have any questions or require additional information, contact me at (508) 830-8323.

There are no regulatory commitments contained in this letter.

Sincerely,

A handwritten signature in black ink, appearing to read "Everett P. Perkins, Jr." with a stylized flourish at the end.

Everett P. Perkins, Jr.
Manager, Regulatory Assurance

EPP/jl

Attachment: Licensee Event Report 2016-003-00, Spent Fuel Storage Design Feature Exceeded (5 pages)

IEZZ
NRR

cc: Mr. Daniel H. Dorman
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
2100 Renaissance Blvd., Suite 100
King of Prussia, PA 19406-2713

Ms. Booma Venkataraman, Project Manager
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Mail Stop O-8C2A
Washington, DC 20555

NRC Senior Resident Inspector
Pilgrim Nuclear Power Station

Attachment

Letter Number 2.16.037

Licensee Event Report 2016-003-00

Spent Fuel Storage Design Feature Exceeded

(5 Pages)



LICENSEE EVENT REPORT (LER)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Privacy and Information Collections Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME Pilgrim Nuclear Power Station	2. DOCKET NUMBER 05000293	3. PAGE 1 OF 5
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4. TITLE
Spent Fuel Storage Design Feature Exceeded

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
05	12	2016	2016	- 003	- 00	07	11	2016	N/A	N/A
									FACILITY NAME	DOCKET NUMBER
									N/A	N/A

9. OPERATING MODE	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: <i>(Check all that apply)</i>			
N	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)
	<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input checked="" type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)
100	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)
	<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> 73.77(a)(1)
	<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	<input type="checkbox"/> 73.77(a)(2)(i)
	<input type="checkbox"/> 20.2203(a)(2)(vi)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input type="checkbox"/> 73.71(a)(2)(ii)
	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> OTHER	Specify in Abstract below or in NRC Form 366A	

12. LICENSEE CONTACT FOR THIS LER

LICENSEE CONTACT Mr. Everett P. Perkins, Jr. - Regulatory Assurance Manager	TELEPHONE NUMBER (Include Area Code) 508-830-8323
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13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT


CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX

14. SUPPLEMENTAL REPORT EXPECTED <input checked="" type="checkbox"/> YES (If yes, complete 15. EXPECTED SUBMISSION DATE) <input type="checkbox"/> NO	15. EXPECTED SUBMISSION DATE MONTH: 9 DAY: 15 YEAR: 2016
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ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On May 12, 2016, at 16:47 EDT, an assessment of Boron Areal Density Gauge for Evaluating Racks (BADGER) testing results for spent fuel storage racks revealed a single panel, RR35 South, with a gap in neutron absorber material that exceeds spent fuel storage design feature assumptions. The assumptions ensure compliance with Technical Specification (TS) Section 4.3.1.1.b to maintain a spent fuel pool k-effective (K_{eff}) of less than or equal to 0.95. An extent of condition review indicates additional at-risk rack locations that may further challenge compliance with TS 4.3.1.1.b.

The cause of the noncompliance with TS 4.3.1.1.b is Boraflex degradation greater than assumed in the criticality safety analysis. Interim actions include declaring the spent fuel pool rack cells adjacent to the panel RR35 South inoperable, installing blade guides into cell RR35 and adjacent locations, and prohibiting fuel movement in Boraflex racks. A criticality analysis of the Spent Fuel Pool (SFP) Boraflex racks is being performed and will be used to determine the need to move fuel within or out of Boraflex racks to maintain the SFP K_{eff} less than or equal to 0.95. Preliminary results indicate the SFP is and will remain substantially subcritical and further analyses are expected to demonstrate K_{eff} of less than or equal to 0.95. Criticality analysis results will be provided in a supplement to this report.

NRC FORM 366A (11-2015)	U.S. NUCLEAR REGULATORY COMMISSION  LICENSEE EVENT REPORT (LER) CONTINUATION SHEET	APPROVED BY OMB: NO. 3150-0104 EXPIRES: 10/31/2018 Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA, Privacy and Information Collections Branch (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.
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Pilgrim Nuclear Power Station	05000293	YEAR	SEQUENTIAL NUMBER	REV NO.
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NARRATIVE

BACKGROUND

Boraflex degradation is a known industry issue that has warranted monitoring and evaluation to ensure the margin to criticality is maintained in the SFP. The Boraflex Monitoring Program is described in Updated Final Safety Analysis Report Appendix S, License Renewal Commitments, as a program that assures degradation of the Boraflex panels in the spent fuel racks does not compromise the criticality analysis in support of the design of the spent fuel storage racks. The program relies on (1) neutron attenuation testing, (2) determination of boron loss through correlation of silica levels in SFP water samples and periodic areal density measurements, and (3) analysis of criticality to assure that the required 5% subcriticality margin is maintained.


RACKLIFE is a commercial software program, based on the degradation mechanism of Boraflex, developed to calculate the average Boron loss of a Boraflex panel and to predict the future Boron loss of Boraflex.

BADGER is a commercial in-situ nondestructive diagnostic tool for measuring the areal density of Boron and the gaps in a neutron absorber as installed in fuel racks. BADGER testing measures the neutron transmission rate through SFP Boraflex panels by placing a neutron source and neutron detectors on opposite sides of a panel. Measured neutron transmission is related to Boron areal density and allows for measurement of gaps in the neutron absorber. Overall, RACKLIFE and BADGER approach the loss of Boron areal density from Boraflex from opposite sides. RACKLIFE calculates the percent of Boron that is lost while BADGER measures the areal density of Boron that remains. To quantify the loss of Boron in spent fuel pool racks, the RACKLIFE and BADGER methods are combined.

The average number of neutrons that cause new fission events is called the effective neutron multiplication factor, denoted by K_{eff} . When K_{eff} is equal to 1, the assembly is called critical, if K_{eff} is less than 1 the assembly is said to be subcritical, and if K_{eff} is greater than 1 the assembly is called supercritical.

EVENT DESCRIPTION

During a BADGER test campaign conducted in the spent fuel pool from March 28, 2016 to April 5, 2016, a 38 inch cumulative gap was found that exceeds the maximum analyzed gap size of 10 inches, and challenged compliance with TS 4.3.1.1.b SFP storage rack K_{eff} requirement of less than or equal to 0.95. This result was unexpected and far exceeded the previous results or predictions of total gap size. No other panel tested had a cumulative gap size greater than or equal to 6.0 inches nominal. Sixteen panels tested had previously been BADGER tested in 2006 and 2012. None of these panels were found with a total gap greater than anticipated. The 2016 BADGER testing was the first measurement for panel RR35 South which is located in SFP rack E5. Panels in rack E5 were chosen for testing to ensure that the highest dose Boraflex panel in the SFP was tested. Panel RR35 South was not the highest dose panel, but was in the region that was cleared for testing and included in the test population. Previous BADGER test panels were selected to ensure that the highest predicted boron areal density loss panel was tested. This is not necessarily the highest dose panel. Other panels were selected to obtain a sampling of high, medium, and low dose and or Boron loss panels with consideration for the amount of fuel movement required for the test. This most recent test used the same approach but also included the panel predicted to have the highest dose. There were 71 Boraflex panels tested during the 2016 BADGER testing. These panels were predicted by the RACKLIFE model to have varying degrees of Boron loss and accumulated dose. The panel selection was made jointly by Reactor

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Engineering, Fuels Engineering and the BADGER test vendor.

Fuels Engineering reviewed the preliminary 2016 BADGER test results and determined that additional testing was not required at that time since 21 other panels with a similar dose and history were tested during this most recent test with a maximum cumulative gap preliminarily reported at 6.7 inches (final value of 6.0 inches). These 21 similar panels included all six panels in cells that share the RR35 South panel, as well as several other panels within the same rack as RR35 South. Since the 2016 BADGER measurements found only one panel out of the 22 similar panels to have a large cumulative gap beyond the analysis basis, the cumulative gap measured in panel RR35 South was considered to be an isolated occurrence.. There was no fuel in RR34, RR35 and other surrounding cells as fuel in this area was removed for the BADGER tests. Therefore, based on panel RR35 South test results, the six rack locations surrounding panel RR35 South were declared inoperable and blocked with blade guides to preclude inadvertent loading of fuel adjacent to the defective panel.

In order to evaluate the preliminary 2016 BADGER test results, a conservative Boraflex gap model was developed based on BADGER and blackness testing measurements performed at Pilgrim Nuclear Power Station (PNPS) and blackness testing measurements at Grand Gulf Nuclear Station (GGNS) over the last 28 years. Both GGNS and PNPS use fuel storage racks designed and manufactured by the same manufacturer and installed circa 1986. The model includes site specific factors that affect Boraflex degradation, such as gamma dose absorbed by each panel and the fuel storage history of each storage cell. The other key factor that impacts Boraflex degradation is the rate which SFP water circulates through the rack volume that encapsulates the Boraflex panel. This parameter is a function of the design and manufacturing process so it is common to both sites' racks. Application of this model determined the extent of condition to be that approximately 589 panels are susceptible to gap losses of 10 inches (maximum gap loss considered in the current analysis of record) or larger. Based on these results, all Boraflex racks and any adjacent non-Boraflex cells that credit the neutron absorber in the Boraflex racks were declared inoperable.

CAUSE OF THE EVENT

The cause of the noncompliance with TS 4.3.1.1.b is Boraflex gaps greater than assumed in the criticality safety analysis. Boraflex has been observed to be subject to in-service degradation from the combined effects of gamma radiation from spent fuel and long term exposure to the aqueous pool environment. Boraflex polymer shrinks due to irradiation beyond certain dose and forms gaps, which may have caused flow induced washout of the Boraflex material. The synergistic effects of gamma radiation and water cause the chemical composition of the polymer matrix of Boraflex to change from polydimethyl siloxane to amorphous silica, which is somewhat soluble in water. As the transformed matrix dissolves, it releases Boron and crystalline silica filler materials to the interior of the fuel racks and eventually to the SFP, resulting in gap growth. A contributing cause of the event is that water infiltration into cell RR35 South could be higher due to variation in fabrication quality. This cause would contribute to this condition by enabling boron washout.

The apparent cause of the unanalyzed condition is reliance on the method of BADGER Testing / RACKLIFE alone to determine the condition of Boraflex panels in the SFP.

NRC FORM 366A
(11-2015)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB: NO. 3150-0104

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CORRECTIVE ACTIONS

Corrective actions include performing a criticality evaluation to determine acceptable fuel storage configurations in Boraflex racks and achieving a SFP arrangement, in accordance with the criticality evaluation, to meet and maintain SFP K_{eff} less than or equal to 0.95 as required by TS 4.3.1.1.b. Results of the evaluation will be provided in a supplement to this report.

The BADGER/RACKLIFE method of prediction of cumulative gap was augmented by a gap model based on PNPS and GGNS gap measurements to determine that approximately 589 locations were susceptible to gaps that could exceed the current analysis assumption. Further criticality analysis and corrective actions will utilize this gap model to determine the cumulative gap size to utilize within analysis models of the Boraflex racks. A more refined number of locations will also be calculated and provided in the supplement to this report.

SAFETY CONSEQUENCES


There was no actual safety consequence related to the event because analyses have demonstrated the current fuel storage configuration is and will be maintained substantially subcritical. This analysis utilized assumptions concerning Boraflex gap size and location based upon the results of the most recent BADGER test to conservatively model all Boraflex rack cells, including those determined to be susceptible to gaps that could exceed the current analysis assumption. In addition, SFP rack cells adjacent to panel RR35 South have been declared inoperable, blade guides have been installed into cell RR35 and adjacent locations, and fuel movement in Boraflex racks has been stopped until completion of additional criticality analysis that supports movement of fuel within the Boraflex racks.

REPORTABILITY

A condition prohibited by TS occurred when the cumulative gap loss in SFP panel RR35 South invalidated the loss limit in the SFP Nuclear Criticality Boron Loss Study and invalidated the conclusion that the Boron loss projections would assure the SFP K_{eff} less than 0.95 until July 28, 2016. This was a noncompliance with TS 4.3.1.1.b and is reportable in accordance with 10 Code of Federal Regulations (CFR) 50.73(a)(2)(i)(B). Condition Report CR-PNP-2016-02205 addresses the noncompliance. Based on initial extent of condition review, this event was reported under Event Notification Report Number 51924 on May 12, 2016, as an unanalyzed condition in accordance with 10 CFR 50.72(b)(3)(ii)(B). Based on the unknown condition of other panels that are susceptible to gap losses, this condition is reportable as an unanalyzed condition in accordance with 10 CFR 50.73(a)(2)(ii)(B).

PREVIOUS EVENTS

None

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ENERGY INDUSTRY IDENTIFICATION SYSTEM (EIIS) CODES

The EIIS codes for Components and Systems referenced in this report are as follows:

COMPONENTS: None
SYSTEMS: None

CODES: None
CODES: None

REFERENCES: CR-PNP-2016-02205