

**CERTIFICATE OF COMPLIANCE
FOR SPENT FUEL STORAGE CASKS**

The U.S. Nuclear Regulatory Commission is issuing this certificate of compliance pursuant to Title 10 of the *Code of Federal Regulations*, Part 72, "Licensing Requirements for Independent Storage of Spent Nuclear Fuel and High-Level Radioactive Waste" (10 CFR Part 72). This certificate is issued in accordance with 10 CFR 72.238, certifying that the storage design and contents described below meet the applicable safety standards set forth in 10 CFR Part 72, Subpart L, and on the basis of the Final Safety Analysis Report (FSAR) of the cask design. This certificate is conditional upon fulfilling the requirements of 10 CFR Part 72, as applicable, and the conditions specified below.

Certificate No.	Effective Date (Certificate)	Expiration Date	Docket No.	Amendment No.	Amendment Effective Date	Package Identification No.
1004	1/23/95	1/23/2015	72-1004	7	March 2, 2004	USA/72-1004
	Renewed Effective Date N/A	Renewed Expiration Date N/A		Revision No 1	Revision Effective Date TBD	

Issued To: (Name/Address)

AREVA Inc.
7135 Minstrel Way, Suite 300
Columbia, MD 21045

Safety Analysis Report Title

Transnuclear West, Inc., "Final Safety Analysis Report for the Standardized NUHOMS® Horizontal Modular Storage System for Irradiated Nuclear Fuel"

CONDITIONS

1. Casks authorized by this certificate are hereby approved for use by holders of 10 CFR Part 50 licenses for nuclear power reactors at reactor sites under the general license issued pursuant to 10 CFR 72.210 subject to the conditions specified by 10 CFR 72.212 and the attached technical specifications.
2. The holder of this certificate who desires to change the certificate or technical specifications shall submit an application for amendment of the certificate or technical specifications.
3. CASK:
 - a. Model Nos. Standardized NUHOMS®-24P, -52B, -61BT, -32PT, and -24PHB

The two digits refer to the number of fuel assemblies stored in the dry shielded canister (DSC), the character P for pressurized water reactor (PWR) or B for boiling water reactor (BWR) is to designate the type of fuel stored, and T is to designate that the DSC is intended for transportation in a 10 CFR Part 71 approved package. The characters HB refer to designs qualified for fuel with burnup greater than 45 GWd/Mtu.

b. Description

The Standardized NUHOMS® System is certified as described in the final safety analysis report (FSAR) and in the NRC's safety evaluation report (SER). The Standardized NUHOMS® System is a horizontal canister system composed of a steel dry shielded canister (DSC), a reinforced concrete horizontal storage module (HSM), and a transfer cask (TC). The welded DSC provides confinement and criticality control for the storage and transfer of irradiated fuel. The concrete module provides radiation shielding while allowing cooling of the DSC and fuel by natural convection during storage. The TC is used for transferring the DSC from/to the Spent Fuel Pool Building to/from the HSM.

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The principal component subassemblies of the DSC are the shell with integral bottom cover plate and shield plug and ram/grapple ring, top shield plug, top cover plate, and basket assembly. The shell length is fuel-specific. The internal basket assembly for the 24P, 24PHB, and 52B DSCs is composed of guide sleeves, support rods, and spacer disks. This assembly is designed to hold 24 PWR fuel assemblies or 52 BWR assemblies. An alternate basket assembly configuration, consisting of assemblies of stainless steel fuel compartments held in place by basket rails and a holdown ring, is designed to hold 61 BWR assemblies. The 32PT DSC basket assembly configuration is similar, consisting of welded stainless steel plates or tubes that make up a grid of fuel compartments supported by aluminum basket rails, and is designed to accommodate 32 PWR assemblies. The basket assembly aids in the insertion of the fuel assemblies, enhances subcriticality during loading operations, and provides structural support during a hypothetical drop accident. The DSC is designed to slide from the transfer cask into the HSM and back without undue galling, scratching, gouging, or other damage to the sliding surfaces.

The HSM is a reinforced concrete unit with penetrations located at the top and bottom of the side walls for air flow. The penetrations are protected from debris intrusions by wire screens during storage operation. The DSC Support Structure, a structural steel frame with rails, is installed within the HSM module to provide for sliding the DSC in and out of the HSM and to support the DSC within the HSM.

The TC is designed and fabricated as a lifting device to meet NUREG-0612 and ANSI N14.6 requirements. It is used for transfer operations within the Spent Fuel Pool Building and for transfer operations to/from the HSM. The TC is a cylindrical vessel with a bottom end closure assembly and bolted top cover plate. Two upper lifting trunnions are located near the top of the cask for downending/uprighting and lifting of the cask in the Spent Fuel Pool Building. The lower trunnions, located near the base of the cask, serve as the axis of rotation during downending/uprighting operations and as supports during transport to/from the Independent Spent Fuel Storage Installation (ISFSI). The 32PT DSC is transferred in a TC with a radial liquid neutron shield.

With the exception of the TC, fuel transfer and auxiliary equipment necessary for ISFSI operations are not included as a part of the Standardized NUHOMS® System to be reviewed for a Certificate of Compliance under 10 CFR Part 72, Subpart L. Such equipment may include, but is not limited to, special lifting devices, the transfer trailer, and the skid positioning system.

c. Drawings

The drawings for the Standardized NUHOMS® System are contained in Appendix E, K, M and N of the FSAR.

d. Basic Components

The basic components of the Standardized NUHOMS® System that are important to safety are the DSC, HSM, and TC. These components are described in Section 4.2, Table K.2-8 (Appendix K), and Table M.2-18 (Appendix M) of the FSAR.

4. Fabrication activities shall be conducted in accordance with a Commission approved quality assurance program which satisfies the applicable requirements of 10 CFR Part 72, Subpart G, and which is established, maintained, and executed with regard to the cask system.

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5. Notification of fabrication schedules shall be made in accordance with the requirements of 10 CFR 72.232(c).
6. All Standardized NUHOMS® Systems must be fabricated and used in accordance with CoC No. 1004, Amendment No. 7. Standardized NUHOMS® Systems that were previously fabricated and put into operation by general licensees in accordance with the original CoC, or Amendment Nos. 1, 2, 3, 4, 5, and 6 may continue to be used under the appropriate CoC or Amendment.
7. CONTINUED USE OF PREVIOUS VERSION OF AMENDMENT NO. 7

A general licensee may continue to use the previous version of this certificate, Amendment No. 7, dated March 2, 2004, until **[insert date 180 days from the effective date of the revised certificate]**. By **[insert date 180 days from the effective date of the revised certificate]** general licensees using Amendment No. 7, dated March 2, 2004, must have implemented the changes authorized by this revision and completed the evaluation described below.

The general licensee shall perform written evaluations before use and before applying the changes authorized by this revised certificate which establish that the cask, once loaded with spent fuel or once the changes authorized by this revised certificate have been applied will conform to the terms, conditions, and specifications of this revised certificate. The results of this review shall be documented in accordance with 10 CFR 72.212(b)(5) no later than **[insert date 180 days from the effective date of the revised certificate]**.

In applying the changes authorized by this revised certificate, the general licensee shall register each such cask with the Nuclear Regulatory Commission no later than 30 days after applying the changes authorized by the revised certificate. The registration shall include the information required by 10 CFR 72.212(b)(2).

FOR THE NUCLEAR REGULATORY COMMISSION

Steve Ruffin, Acting Chief
Spent Fuel Licensing Branch
Division of Spent Fuel Management
Office of Nuclear Material Safety
and Safeguards

Attachment: A. Technical Specifications

Dated: