



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

June 27, 2016

Mr. Regis T. Repko
Senior Vice President
Governance, Projects, & Engineering
Duke Energy Carolinas, LLC
P.O. Box 1006/ECO7H
Charlotte, NC 28201-1006

SUBJECT: BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2; SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1; CATAWBA NUCLEAR STATION, UNITS 1 AND 2; MCGUIRE NUCLEAR STATION, UNITS 1 AND 2; AND OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3 – CORRECTION TO LICENSE PAGES RE: ISSUANCE OF AMENDMENTS REGARDING ADOPTION OF TSTF-523 (CAC NOS. MF6413 THROUGH MF6422)

Dear Mr. Repko:

By letter dated April 29, 2016,¹ the U.S. Nuclear Regulatory Commission issued Amendment Nos. 270 and 298 to Renewed Facility Operating License (RFOL) Nos. DPR-71 and DPR-62, respectively, for the Brunswick Steam Electric Plant, Unit Nos. 1 and 2; Amendment No. 150 to RFOL No. NPF-63, for the Shearon Harris Nuclear Power Plant, Unit 1; Amendment Nos. 282 and 278 to RFOL Nos. NPF-35 and NPF-52, respectively, for the Catawba Nuclear Station, Units 1 and 2; Amendment Nos. 285 and 264 to RFOL Nos. NPF-9 and NPF-17, respectively, for the McGuire Nuclear Station, Units 1 and 2 (McGuire 1 and 2); and Amendment Nos. 398, 400, and 399 to RFOL Nos. DPR-38, DPR-47, DPR-55, respectively, for the Oconee Nuclear Station, Units 1, 2, and 3.

The license pages attached to Amendment Nos. 285 and 264 to RFOL Nos. NPF-9 and NPF-17, respectively, for McGuire 1 and 2, contained a typographical error. Specifically, the error consisted of stating the incorrect maximum power level in paragraph 2.C.(1) of the McGuire 1 and 2 licenses. Those pages state that the licensee is authorized to operate the facility at a reactor core full steady state power level of 3411 megawatts thermal (100%). The pages should state that the full steady state power level is 3469 megawatts thermal (100%). The error occurred on the updated page 3 of the McGuire 1 and 2 licenses associated with the license amendment.

A corrected copy of the updated page 3 for both the McGuire 1 and 2 licenses is enclosed. This correction does not change any of the conclusions in the safety evaluation associated with the license amendments.

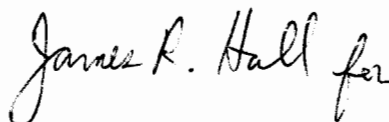
¹ Agencywide Documents Access and Management System Accession No. ML16085A113

R. Repko

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We apologize if this error has caused any inconvenience. If you have any questions regarding this matter, please contact me at (301) 415-4090 or by e-mail at jeffrey.whited@nrc.gov.

Sincerely,



Jeffrey A. Whited, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-325, 50-324, 50-400,
50-413, 50-414, 50-369, 50-370,
50-269, 50-270, and 50-287

Enclosures:

Corrected Page 3 for NPF-9

Corrected Page 3 for NPF-17

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ENCLOSURES

**McGuire Nuclear Station, Units 1 and 2
Corrected Page 3 for License No. NPF-9
Corrected Page 3 for License No. NPF-17
for Amendment Nos. 285 and 264
dated April 29, 2016**

- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproducts and special nuclear materials as may be produced by the operation of McGuire Nuclear Station, Units 1 and 2, and;
- (6) Pursuant to the Act and 10 CFR Parts 30 and 40, to receive, possess and process for release or transfer such byproduct material as may be produced by the Duke Training and Technology Center.

C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at a reactor core full steady state power level of 3469 megawatts thermal (100%).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 285, are hereby incorporated into this renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Updated Final Safety Analysis Report

The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on December 16, 2002, describes certain future activities to be completed before the period of extended operation. Duke shall complete these activities no later than June 12, 2021, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

The Updated Final Safety Analysis Report supplement as revised on December 16, 2002, described above, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following issuance of this renewed operating license. Until that update is complete, Duke may make changes to the programs described in such supplement without prior Commission approval, provided that Duke evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59 and otherwise complies with the requirements in that section.

- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
 - (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproducts and special nuclear materials as may be produced by the operation of McGuire Nuclear Station, Units 1 and 2; and,
 - (6) Pursuant to the Act and 10 CFR Part 30 and 40, to receive, possess and process for release or transfer such byproduct material as may be produced by the Duke Training and Technology Center.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level
The licensee is authorized to operate the facility at a reactor core full steady state power level of 3469 megawatts thermal (100%).
 - (2) Technical Specifications
The Technical Specifications contained in Appendix A, as revised through Amendment No. 264, are hereby incorporated into this renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.
 - (3) Updated Final Safety Analysis Report
The Updated Final Safety Analysis Report supplement submitted pursuant to 10 CFR 54.21(d), as revised on December 16, 2002, describes certain future activities to be completed before the period of extended operation. Duke shall complete these activities no later than March 3, 2023, and shall notify the NRC in writing when implementation of these activities is complete and can be verified by NRC inspection.

The Updated Final Safety Analysis Report supplement as revised on December 16, 2002, described above, shall be included in the next scheduled update to the Updated Final Safety Analysis Report required by 10 CFR 50.71(e)(4), following issuance of this renewed operating license. Until that update is complete, Duke may make changes to the programs described in such supplement without prior Commission approval, provided that Duke evaluates each such change pursuant to the criteria set forth in 10 CFR 50.59, and otherwise complies with the requirements in that section.

R. Repko

- 2 -

We apologize if this error has caused any inconvenience. If you have any questions regarding this matter, please contact me at (301) 415-4090 or by e-mail at jeffrey.whited@nrc.gov.

Sincerely,

/RA James R. Hall for/

Jeffrey A. Whited, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-325, 50-324, 50-400,
50-413, 50-414, 50-369, 50-370,
50-269, 50-270, and 50-287

Enclosures:

Corrected Page 3 for NPF-9

Corrected Page 3 for NPF-17

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