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1 2.1 Evaluation of Defense-in-Depth Attributes and Safety Margins

2 One aspect of the engineering evaluation is to show that the proposed change does not
3 compromise the fundamental safety principles on which the plant design was based. Design-basis
4 accidents (DBAs) play a central role in the design of nuclear power plants. DBAs are a combination of
5 postulated challenges and failure events against which plants are designed to ensure adequate and safe
6 plant response. During the design process, plant response and associated safety margins are evaluated
7 using assumptions of physical properties and operating characteristics that are intended to be
8 conservative. National standards and other considerations such as defense-in-depth attributes and the
9 single-failure criterion constitute additional engineering considerations that also influence plant design
10 and operation. The licensee's proposed LB change may affect margins and defenses incorporated into the
11 current plant design and operation; therefore, the licensee should reevaluate these items to support a
12 requested LB change. As part of this evaluation, the impact of the proposed LB change on the functional
13 capability, reliability, and availability of affected equipment should be determined. The plant's LB
14 identified in the FSAR is the reference point for judging whether a proposed change adversely affects
15 safety margins or defense-in-depth. Sections 2.1.1 and 2.1.2 below provide guidance on assessing
16 whether implementation of the proposed change maintains adequate safety margins and consistency with
17 the defense-in-depth philosophy.

18 2.1.1 *Defense-in-Depth*

19 The engineering evaluation should evaluate whether the impact of the proposed LB change is
20 consistent with the defense-in-depth philosophy. In this regard, the intent of this key principle of risk-
21 informed decision-making is to ensure that any impact of the proposed LB change on defense-in-depth is
22 fully understood and addressed and that the philosophy of defense-in-depth is maintained; not to prevent
23 changes in the way defense-in-depth is achieved. The licensee must fully understand how the change will
24 impact the design, operation and maintenance of the plant, both from risk and traditional engineering
25 perspectives.

26 This section provides some background on the defense-in-depth philosophy. Next is discussion
27 of seven key factors that may be used to evaluate the impact of a proposed change on defense-in-depth.
28 One or more examples are provided to help illustrate what is meant by each factor. Finally, this section
29 provides guidance on a process for evaluating the seven key factors, including an integrated example.

30 2.1.1.1 Background

31 Defense-in-depth is an element of the NRC's safety philosophy that employs successive
32 compensatory measures to prevent accidents or mitigate damage if a malfunction, accident, or naturally
33 caused event occurs at a nuclear facility¹. The defense-in-depth philosophy has traditionally been applied
34 in reactor design and operation to provide multiple means to accomplish safety functions and prevent the
35 release of radioactive material. It has been and continues to be an effective way to account for
36 uncertainties in equipment and human performance and, in particular, to account for the potential for
37 unknown and unforeseen failure mechanisms or phenomena, which (because they are unknown or
38 unforeseen) may not be reflected in either the PRA or traditional engineering analyses.

¹ Staff Requirements Memorandum (SRM) - SECY-98-0144, "White Paper on Risk-Informed and Performance-Based Regulation," March 1, 1999, (Agencywide Document Access and Management System (ADAMS) accession number ML003753601).

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39 For the purposes of this RG, it is useful to consider the following layers of defense (successive
40 measures) when evaluating the impact of the proposed licensing basis change on defense-in-depth:

- 41 • Robust plant design to survive hazards and minimize challenges that could result in an event
42 occurring;
- 43 • Prevention of a severe accident (core damage) should an event occur;
- 44 • Containment of the source term should a severe accident occur; and,
- 45 • Protection of the public from any releases of radioactive material (through, e.g., siting in low
46 population areas and the ability to shelter or evacuate people if necessary).

47 2.1.1.2 Key Factors for Evaluating the Impact of LB Changes on Defense-in-depth

48 Any one or more of the layers of defense discussed above may be adversely impacted by a
49 proposed change to a plant's licensing basis. The NRC has identified seven factors that should be used to
50 assess the impact of the change on defense-in-depth. These are discussed in detail below. Guidance on
51 how to apply these factors is discussed in more detail in section 2.1.1.3.

52 The NRC finds it acceptable for a licensee to use the following seven key factors to evaluate
53 whether a proposed change to the LB maintains the philosophy of defense-in-depth.

54 1. Preserve a reasonable balance among the layers of defense.

55 a. Guidance

56 *A propose change should not* significantly reduce the effectiveness of a layer of defense that
57 exists in the plant design before the proposed change.

58
59 *The evaluation of the proposed change should* consider insights based on traditional engineering
60 approaches; insights from risk assessments may be used to support engineering insights, but not
61 be the only justification for meeting this factor.

62 b. Discussion

63 A reasonable balance of the layers of defense, minimizing challenges to the plant, preventing any
64 events from progressing to core damage, containing the radioactive source term, and emergency
65 preparedness, helps to ensure an apportionment of the plant's capabilities between limiting
66 disturbances to the plant and mitigating their consequences. The term *reasonable balance* is not
67 meant to imply an equal apportionment of capabilities. A reasonable balance is preserved if the
68 proposed plant change does not significantly reduce the effectiveness of a layer that exists in the
69 plant design before the proposed change. The NRC recognizes that there may be aspects of a
70 plant's design that may cause one of the layers to be adversely affected. For these situations, the
71 balance among the other three layers becomes especially important when evaluating the impact of
72 a proposed change to the LB and its impact on defense-in-depth.

73 If a comprehensive risk analysis is done, it can provide insights into whether the balance among
74 the layers of defense remains appropriate to ensure protection of public health and safety. Such a
75 risk analysis would not only include the likelihood of challenges to the plant (i.e., initiating event
76 frequencies) from various hazards, but would include estimates of core damage frequency,

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77 containment response and, in some cases, dose estimates to the public. It would include
78 implementation of the emergency plan and estimate the effectiveness of actions such as sheltering
79 in place or evacuation.

80 Note that the risk acceptance guidelines in this RG are based on the surrogates for the
81 Commission's quantitative health objectives, CDF and LERF. These risk metrics, developed as
82 part of the risk assessment, can help inform the licensee's assessment of the relative balance
83 between the second and third layers of defense.

84 However, to address the unknown and unforeseen failure mechanisms or phenomena, the
85 licensee's evaluation of this factor of defense-in-depth should also address insights based on
86 traditional engineering approaches. Results of the risk assessment may be used to support the
87 conclusion but should not be the only justification for meeting this factor. The licensee should
88 consider the impact of the proposed change on each of the layers of defense:

- 89 – Robust plant design to survive hazards and minimize challenges that could result in an event
90 occurring - the change should not significantly increase the likelihood of initiating events or
91 create new significant initiating events;
- 92 – Prevention of a severe accident (core damage) should an event occur - the change should
93 maintain the availability and reliability of SSCs that provide the safety functions that prevent
94 plant challenges from progressing to core damage;
- 95 – Containment of the source term should a severe accident occur - the change should maintain
96 the containment and SSCs that support that barrier, such as containment fan coolers and
97 sprays; and,
- 98 – Protection of the public from any releases of radioactive material - the change should not
99 reduce the effectiveness of the EP program, including the ability to detect and measure
100 releases of radioactivity, to notify offsite agencies and the public, to shelter or evacuate the
101 public as necessary

102 c. Examples

103 [Under development]

104 2. Preserve adequate capability of design features without an over-reliance on programmatic
105 activities as compensatory measures.

106 a. Guidance

107 *A proposed change should not* significantly reduce the reliability and availability of design
108 features to perform their safety functions.

109
110 *The evaluation of the proposed change should demonstrate that the change does not* result in the
111 overreliance of programmatic activities to compensate for a proposed reduction in the capability
112 of engineered safety features.

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113 b. Discussion

114 Nuclear power plant licensees implement a number of programs, including, for example,
115 programs for quality assurance, testing and inspection, maintenance, control of transient
116 combustible material, foreign material exclusion, containment cleanliness, training, and so forth.
117 In some cases, activities taken as part of these programs are used to ensure safety functions; for
118 example, reactor vessel inspections that provide assurance that reactor vessel failure is unlikely.

119 A proposed change that does not affect how safety functions are performed or reduce the
120 reliability or availability of the SSCs that perform those functions would meet this defense-in-
121 depth factor. However, a licensee could contemplate a change where a reduction in the capability
122 of those SSCs is compensated in some manner by reliance on plant programs. In such a case, the
123 licensee should assess whether the proposed change would increase the need for programmatic
124 activities to compensate for the lack of engineered features. If the change requires new or
125 additional reliance on such programs, the licensee should justify that reliance on these measures
126 is not excessive. Use of compensatory measures may be considered overreliance when a program
127 is substituted for an engineered means of performing a safety function, or failure of the
128 programmatic activity could prevent an engineered safety feature from performing its intended
129 function.

130 The NRC also recognizes that compensatory measures are sometimes associated with temporary
131 conditions. A licensee may request a risk-informed change to the plant's licensing basis to permit
132 occasional entry into conditions requiring measures that rely on plant programs to compensate for
133 reduced capability of engineered systems, or for one-time to allow completion of corrective
134 action to restore engineered systems to match the design and licensing basis. For such situations,
135 the licensee should demonstrate that the plant condition requiring such compensatory measures
136 would occur at a sufficiently low frequency or that the time frame to effect corrective action is
137 commensurate with the significance of the non-conforming condition.

138 c. Examples

139 [Under development]

140 3. Maintain sufficient availability and reliability of SSC commensurate with their importance to
141 safety.

142 a. Guidance

143 *A proposed change should not* defeat the redundancy, independence, or diversity of design
144 features.

145
146 *The evaluation of the proposed change should demonstrate that the change does not* result in a
147 substantial reduction in the availability or reliability of the associated SSCs, e.g., introduction of a
148 new single failure.

149 b. Discussion

150 The importance of system redundancy, independence and diversity is to ensure that the system
151 function can be achieved. A proposed risk-informed change should consider both safety-related
152 and nonsafety-related SSCs that are important to core damage or large early release. Redundancy
153 provides for duplicate equipment that enables the failure or unavailability of at least one set of

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154 equipment to be tolerated without loss of function. Independence among equipment implies that
155 the redundant equipment are separate such that they do not rely on the same supports to function.
156 It can sometimes be achieved by the use of physical separation or physical protection. Diversity
157 is accomplished by having equipment that perform the same function rely on different attributes,
158 such as different principles of operation, different physical variables, different conditions of
159 operation, or production by different manufacturers.

160 A substantial reduction in the ability to accomplish a safety function would likely undermine the
161 effectiveness of a layer of defense-in-depth and, therefore, would not be consistent with the
162 defense-in-depth philosophy. A safety function may be compromised if one of the plant features
163 that provides for either system redundancy, independence, or diversity is defeated. This adverse
164 impact could occur by the introduction of a new dependency that could potentially defeat the
165 redundancy, independence or diversity of the affected equipment. That is, system redundancy,
166 independence and diversity can be assumed to be sufficient if, given the proposed licensing
167 change, the affected system safety function can be accomplished assuming a single failure.

168 The licensee should demonstrate that the proposed licensing change would not affect system
169 redundancy, independence, or diversity of the affected equipment; that is, the affected system
170 safety function can still be accomplished assuming a single failure.

171 c. Examples

172 [Under development]

173 4. Preserve adequate defense against potential common-cause failures (CCF).

174 a. Guidance

175 *A proposed change should not reduce defenses against CCFs that could defeat the redundancy,*
176 *independence, and/or diversity of DID layers, fission product barriers, and engineered safety*
177 *features.*

178 *The evaluation of the proposed change should demonstrate that the change does not result in a*
179 *reduction of existing CCF defenses or introduce new CCF dependencies.*
180

181 b. Discussion

182 An important aspect of ensuring defense-in-depth is to guard against CCF. Failure of several
183 devices or components to function may occur as a result of a single specific event or cause. Such
184 failures may simultaneously affect several different items important to risk. The event or cause
185 may be a design deficiency, a manufacturing deficiency, an operating or maintenance error, a
186 natural phenomenon, a human-induced event, or an unintended cascading effect from any other
187 operation or failure within the plant.

188 The licensee should evaluate the proposed change to determine whether it increases the potential
189 for events or causes that would be a CCF. The licensee should also evaluate the proposed change
190 to determine whether new CCF mechanisms could be introduced.

191 c. Examples

192 [Under development]

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193 5. Maintain multiple fission product barriers.

194 a. Guidance

195 *A proposed change should not significantly reduce the effectiveness of the multiple fission*
196 *product barriers.*

197

198 *The evaluation of the proposed change should demonstrate that the change does not:*

199

200 — Create a significant increase in the likelihood or consequence of an event that simultaneously
201 challenges multiple barriers and is within the plant's existing licensing basis.

202

203 — Introduce the possibility of a new event that would simultaneously impact multiple barriers.

204

205 b. Discussion

206 This factor refers to the physical fission product barriers e.g., the fuel cladding, reactor coolant
207 system pressure boundary, and containment. This includes the physical barriers themselves and
208 any equipment relied upon to protect the barriers (e.g., containment spray). In general, these
209 barriers are designed to perform independently so that a complete failure of one barrier does not
210 disable the next subsequent barrier. For example, one barrier, the containment, is designed to
211 withstand a double-ended guillotine break of the largest pipe in the reactor coolant system,
212 another barrier.

213 A plant's licensing basis may contain events that, by their very nature, challenge multiple barriers
214 simultaneously. Examples include interfacing-system LOCA and SGTR. Therefore, complete
215 independence of barriers, while a goal, may not be achievable for all possible scenarios.

216 To demonstrate that this factor is met, the licensee should demonstrate that the change does not
217 create a significant increase in the likelihood or consequence of an event that simultaneously
218 challenges multiple barriers and is within the plant's existing licensing basis.

219 Furthermore, the licensee should demonstrate that the change does not introduce the possibility of
220 a new event that would simultaneously impact multiple barriers. If this cannot be shown, the
221 licensee should:

222 — Perform a deterministic analysis to show that the simultaneous challenge to multiple barriers
223 caused by the new event can be mitigated. This may be done by assuming that the new event
224 has occurred and performing an analysis (using conservative assumptions) demonstrating that
225 affected barriers would perform their safety function or;

226 — Use the results of the plant's PRA to demonstrate that the likelihood of the new event is
227 sufficiently low such that independence of barriers would not be significantly affected by the
228 proposed change.

229 c. Examples

230 [Under development]

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- 231 6. Preserve sufficient defense against human errors.
- 232 a. Guidance
- 233 *A proposed change should not significantly increase the potential for or create new human errors*
234 *that may adversely affect one or more layers of defense.*
- 235 *The evaluation of the proposed change should demonstrate that the change does not*
- 236 — create new human failure events that have a significant adverse impact on risk;
237 — significantly increase the burden on the operators responding to events; or,
238 — significantly increase the human error probability of existing operator actions.
- 239 b. Discussion
- 240 Human errors include the failure of operators to perform the actions necessary to operate the plant
241 or respond to off-normal conditions and accidents, errors committed during test and maintenance,
242 and operators performing an incorrect action. Human errors can result in the degradation or
243 failure of a system to perform its function, thereby significantly reducing the effectiveness of one
244 of the defense-in-depth layers or one of the fission product barriers.
- 245 The plant design and operation includes defenses to prevent the occurrence of such errors and
246 events. These defenses generally involve the use of procedures, training, and human engineering;
247 however, other factors, e.g., communication protocols, may also be important.
- 248 In determining whether these defenses are preserved, the licensee should assess whether the
249 proposed change would create new operator actions that significantly impact the change in risk,
250 place a greater mental/physical demand on operators in responding to events, or increase the
251 probability of existing operator errors. The licensee should consider whether the change creates
252 new situations that are likely to cause errors, not only for operators, but for maintenance
253 personnel and other plant staff.
- 254 c. Examples
- 255 [Under development]
- 256 7. Continue to meet the intent of the plant's design criteria. **[NRC staff is considering deleting**
257 **this evaluation factor and expanding the narrative of the first paragraph of Section 2.1.1 of**
258 **this document to more fully explain the concept of this factor.]**
- 259 a. Guidance
- 260 *A proposed change should not affect meeting the intent of the plant's design criteria referenced in*
261 *the licensing basis.*
- 262
- 263 *The evaluation of the proposed change should demonstrate that the change does not affect*
264 *meeting the intent of the plant's design criteria referenced in the licensing basis.*
265

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266 b. Discussion

267 The plant’s design criteria establish the necessary design, fabrication, construction, testing, and
268 performance requirements for SSCs important to safety; that is, SSCs that provide reasonable
269 assurance that the facility can be operated without undue risk to the health and safety of the
270 public. The plant’s design criteria define minimum requirements that achieve aspects of the
271 defense-in-depth philosophy; as a consequence, a compromise to those design criteria can directly
272 result in a significant reduction in the effectiveness of one or more of the defense-in-depth layers.
273 When evaluating the effect of the proposed change, the licensee should demonstrate that the
274 intent of the plant’s design criteria continue to be met.

275 The General Design Criteria of Appendix A to 10 CFR 50 form the basis for the design criteria
276 for newer plants. In some cases, exemptions to specific GDC may have been granted. Older
277 plants may have been licensed to other criteria, such as the AEC draft design criteria. A given
278 plant’s design criteria are summarized in its UFSAR. This factor of defense-in-depth should
279 consider the current licensing basis of the plant.

280 c. Examples

281 [Under development]

