

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 7902050127 DOC. DATE: 79/01/30 NOTARIZED: NO  
 FACIL: 50-270 Oconee Nuclear Station, Unit 2, Duke Power Co.  
 AUTH. NAME AUTHOR AFFILIATION  
 LEWIS, S.R. Duke Power Co.  
 RECIP. NAME RECIPIENT AFFILIATION  
 Region 2, Atlanta, Office of the Director

DOCKET #  
 05 000270

SUBJECT: LER 78-014/03L-0 on 781230: during power escalation, RCS leakage in excess of 1 gpm was noted. Leak resulted from packing on pressurizer spray control bypass valve 2RC-2. Valve backseated to halt leakage.

DISTRIBUTION CODE: A002S COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 2+1  
 TITLE: INCIDENT REPORTS

NOTES: M CUNNINGHAM - ALL AMOTS TO FSAR & CHANGES TO TECH SPECS.

ACTION:	RECIPIENT	COPIES		RECIPIENT	COPIES	
	ID CODE/NAME	LTR	ENCL		ID CODE/NAME	LTR
	05 BC <u>ORB# 4</u>	4	4			
INTERNAL:	<del>01 REG FILE</del>	1	1	02 NRC PDR	1	1
	09 I&E	2	2	11 MPA	3	3
	14 TA/EDO	1	1	15 NOVAK/KNIEL	1	1
	16 EEB	1	1	17 AD FOR ENGR	1	1
	18 PLANT SYS BR	1	1	19 I&C SYS BR	1	1
	20 AD PLANT SYS	1	1	21 AD SYS/PROJ	1	1
	22 REAC SAFT BR	1	1	23 ENGR BR	1	1
	24 KREGER	1	1	25 PWR SYS BR	1	1
	26 AD/SITE ANAL	1	1	27 OPERA LIC BR	1	1
	28 ACIDENT ANLYS	1	1	E JORDAN/IE	1	1
EXTERNAL:	03 LPDR	1	1	04 NSIC	1	1
	29 ACRS	16	16			

FEB 6 1979

TOTAL NUMBER OF COPIES REQUIRED: LTR 45 ENCL 45

DUKE POWER COMPANY  
OCONEE UNIT 2

Report Number: RO-270/78-14

Report Date: January 30, 1979

Occurrence Date: December 30, 1978

Facility: Oconee Unit 2, Seneca, South Carolina

Identification of Occurrence: Unidentified RCS Leakage in Excess of 1 GPM

Conditions Prior to Occurrence: 39% Full Power

Description of Occurrence:

On December 30, 1978, during power escalation testing following a refueling outage, a problem with the computer program used to calculate Reactor Coolant System (RCS) leakage was discovered. The problem was corrected at approximately 1400. At approximately 2000, it was determined that the RCS had unidentified leakage in excess of 1 GPM. This is a violation of Oconee Nuclear Station Technical Specification 3.1.6.2. An increase in the normal sump level was also observed. An evaluation was started as required by Technical Specification 3.1.6.6. At 2330 it was determined that the leak was inside the Reactor Building, and, from RIA and sample results on the Reactor Building atmosphere, that there was no significant increase in Reactor Building gaseous activity. Early on the morning of December 31, 1978, shutdown of the unit was initiated to allow entry of the Reactor Building to determine the location of the leak. At approximately 0400, a leak was identified to be coming from the packing on Valve 2RC-2, which is the bypass valve for the pressurizer spray valve. The valve was opened fully and backseated to stop the leak. RCS leakage was returned to within specification. The unit was returned to service at 1154.

Apparent Cause of Occurrence:

The excessive RCS leakage resulted from a packing leak on Pressurizer Spray Control Bypass Valve 2RC-2.

Analysis of Occurrence:

No Reactor Building purge was in operation at the time, so the leakage was entirely contained within the Reactor Building. The leak rate was well within the capacity of one high pressure injection pump. Therefore, this incident did not adversely affect public health and safety.

Corrective Action:

The valve was backseated, and the leak stopped. The valve will be repacked at the next cold shutdown of the unit. In addition, since a similar leakage occurred previously on Unit 1, it is intended that these valves will be replaced when replacement valves are available.

