

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8011040505 DOC. DATE: 80/10/31 NOTARIZED: NO
 FACIL: 50-269 Oconee Nuclear Station, Unit 1, Duke Power Co.
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 RECIP. NAME: REGION 2, Atlanta, Office of the Director

DOCKET # 05000269

SUBJECT: LER 80-030/03L-0: on 801002, valve LPSW-565 failed to close completely during testing causing reactor bldg cooler unit to be out of line in ES mode. Caused by torque switch setting being out of adjustment. Torque switches reset.

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 TITLE: Incident Reports

NOTES: M Cunningham: all amends to FSAR & changes to Tech Specs. AEOD, Ornstein: lcc. 05000269

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71 ENCL 71

DUKE POWER COMPANY
OCONEE NUCLEAR STATION, UNIT 1

Report Number: RO-269/80-30

Report Date: October 31, 1980

Occurrence Date: October 2, 1980

Facility: Oconee Unit 1, Seneca, South Carolina

Identification of Occurrence: Reactor Building Cooler Unit 1B Declared
Inoperable Due to Failure of Valve LPSW-565

Conditions Prior to Occurrence: Oconee 1 - 98% FP

Description of Occurrence:

At 1415 on October 2, 1980, valve LPSW-565 failed to close completely when tested per the Operational Valve Functional Test. Due to this failure, the Reactor Building Cooler Unit (RBCU) 1B could not be lined up in the E.S. mode. Therefore, RBCU 1B was declared inoperable. Operation with one RBCU inoperable constitutes a degraded mode permitted by Technical Specification 3.3.5.c(2) and is thus reportable pursuant to Technical Specification 6.6.2.1.b(2).

Apparent Cause of Occurrence:

It was determined that the failure was due to the torque switch settings being out of adjustment. The torque switches have been involved in several previous failures.

Analysis of Occurrence:

Upon receipt of a close signal from the RZ module, valve LPSW-565 closed until 600 gpm was indicated. Normal flow through this valve is 1000 to 1400 gpm. The valve was subsequently closed by bypassing the torque switch contacts in the breaker, thus indicating that the torque switch setting was probably the problem.

Valve LPSW-565 was installed to prevent cooling water from bypassing RBCU B following a LOCA and to isolate the auxiliary cooler header which is non-seismic. Failure of this valve does not cause nor increase the probability of any transient or accident which would endanger plant personnel, equipment, or the public.

If valve LPSW-565 failed to close following a LOCA, some flow would be diverted from RBCU B reducing its effectiveness. If an associated seismic event failed the auxiliary cooler header, with LPSW-565 open or partially closed, the leak would be detected by comparing inlet and outlet flows, and the cooler would be isolated by closing 1LPSW-19 and 1LPSW-21. This would remove RBCU 1B from service.

Pursuant to paragraph 6.3.2.7 of the FSAR, three cooling units or both trains of building spray or two cooling units with one train of building spary are adequate to cool the post-accident atmosphere. Since the other two RBCU's and both building spray trains were still available, adequate cooling in the event of an accident would have been available even with a postulated additional failure of one of the available building spray trains or RBCU's. Therefore, this incident was of no significance with respect to safe operation, and the health and safety of the public were not affected.

Corrective Action:

Valve LPSW-565 was closed from the breaker and LPSW-566 opened to return RBCU B to E.S. standby. The LPSW-565 torque switches were reset and the system returned to the normal operating mode.

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

SCNEEE 12000-000000-0003411114

REPORT SOURCE L0500026971002808103180

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES

On October 2, 1980, valve LPSW-565 failed to close completely when tested per the Operational Valve Functional Test. Due to this failure, the Reactor Building Cooler Unit could not be lined up in the ES mode, this RBCU 1B was declared inoperable. Two of the RB spray trains were operable and available for ES actuation to alleviate RB temperature and pressure in the event of an accident. Thus, this incident was of no significance with respect to safe operation, and health and safety of the public were not affected.

SYSTEM CODE SB CAUSE CODE E CAUSE SUBCODE B COMPONENT CODE VALVE EX COMP. SUBCODE E VALVE SUBCODE D LER/RO REPORT NUMBER 80 EVENT YEAR 80 SEQUENTIAL REPORT NO. 030 OCCURRENCE CODE 03 REPORT TYPE L REVISION NO. 0 ACTION TAKEN E FUTURE ACTION Z EFFECT ON PLANT Z SHUTDOWN METHOD Z HOURS 000 ATTACHMENT SUBMITTED Y NPRD-4 FORM SUB. Y PRIME COMP. SUPPLIER L COMPONENT MANUFACTURER X999

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS

It was determined that the failure was due to the torque switch settings being out of adjustment. Valve LPSW-565 was closed from the breaker and LPSW-566 opened to return RBCU B to ES standby. The LPSW-565 torque switches were reset and the system returned to the normal operating mode.

FACILITY STATUS E % POWER 098 OTHER STATUS NA METHOD OF DISCOVERY B DISCOVERY DESCRIPTION Operational Valve Functional Test ACTIVITY CONTENT Z AMOUNT OF ACTIVITY NA LOCATION OF RELEASE NA PERSONNEL EXPOSURES 000 Z DESCRIPTION NA PERSONNEL INJURIES 0000 DESCRIPTION NA LOSS OF OR DAMAGE TO FACILITY Z DESCRIPTION NA PUBLICITY ISSUED N DESCRIPTION NA

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