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DUKE POWER COMPANY  
- P. O. Box 33189  
CHARLOTTE, N. C. 28242

*Central File*  
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A. C. THIES  
SENIOR VICE PRESIDENT  
PRODUCTION AND TRANSMISSION

April 21, 1980

(704) 373-4249

APR 25 11:36

Mr. James P. O'Reilly, Director  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, Suite 3100  
Atlanta, Georgia 30303

Re: Oconee Nuclear Station  
Docket Nos. 50-269, -270, -287

Dear Sir:

Please find attached Duke Power Company's response to the 90-day phase of IE Bulletin 79-01B as required by Item 7(b). This documents completion of Items 4 and 5 of the bulletin, and updates our submittal of March 13, 1980.

Very truly yours,



A. C. Thies

ACT:scs

Attachment

cc: Director  
Office of Inspection & Enforcement  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555



OFFICIAL COPY


Mr. James P. O'Reilly, Director  
April 21, 1980  
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A. C. THIES, being duly sworn states that he is a Senior Vice President of Duke Power Company; that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this response to a bulletin requested under provisions of 10CFR 50.54 (f); and that all statements and matters set forth herein are true and correct to the best of his knowledge.



A. C. Thies, Senior Vice President

Subscribed and sworn to before me this 21st day of April, 1980.

  
Sue C. Herrill  
Notary Public

My Commission Expires:

September 20, 1984

OCONEE NUCLEAR STATION

RESPONSE TO IE BULLETIN 79-01B

In accordance with IE Bulletin 79-01B, Duke Power Company is providing as a part of the 90 day submittal revised copies of the master systems/components list and system component evaluation work sheets. These revised pages completely supercede our 45 day response and include the additional equipment associated with long-term core cooling following a LOCA.

It should be noted that our response to this bulletin has thus far not specifically included equipment required to achieve cold shutdown. However, based on the NRC's response to Generic Question 3 concerning this bulletin, we are presently determining the equipment required for cold shutdown. We expect to update the 90 day response to include the additional equipment by July 1, 1980.

As a result of the NRC's March 24 and 25, 1980 site audit of the 45 day response, the Staff has requested that we list and address the qualification of the following items:

- . Raychem Heat Shrink Tubing
- . Scotch Cast-9
- . Transmitter O-Ring Seals
- . Lubricating Grease

We expect to provide the qualification information for the above items by July 1, 1980.

In accordance with Action Item 4 of this bulletin, Duke Power Company is in the process of evaluating the qualification data presently available in our files against the guidelines of Enclosure 4. The following paragraphs generally discuss the guidelines of Enclosure 4 (Sections 3.0 through 8.0) as they relate to the qualification of safety-related equipment at Oconee:

### 3.0 Identification of Equipment

As stated in our 45 day submittal, the systems and components listed were identified based on the following criteria:

- a) the systems/components are required to function to mitigate the consequences of a postulate LOCA or high energy line break (HELB) inside the Reactor Building or a HELB outside the Reactor Building, and
- b) the systems/components are subjected to the environmental conditions resulting from the LOCA or HELB described in (a) above.

As stated above, the additional equipment identified in this 90 day submittal which was not included in the 45 day submittal is the equipment associated with long-term core cooling following a LOCA. The criteria for listing this electrical equipment is that it is subjected to a radiation dose above 1000 rads. Additionally, equipment required to achieve cold shutdown is presently being determined and will be included in the response to this bulletin by July 1, 1980.

#### 4.0 Service Conditions

The temperature and pressure conditions as a function of time following a LOCA for equipment located inside the Reactor Building are based on FSAR analyses (Ref. FSAR Chapter 14). The temperature and pressure conditions following a main steamline break are bounded by the LOCA Conditions. Each Ocone unit is provided with a redundant safety-related automatic spray system.

The radiation levels calculated for the Reactor Building were determined using the TID 14844 source term (100% noble gases, 50% iodines, and 1% fission products). For equipment in the Reactor Building, the radiation levels when shown on the component evaluation work sheets (spec value) are the 40 year normal dose plus one year accident dose.

Safety-related equipment located inside the Reactor Building which is submerged following a LOCA is identified on the component evaluation work sheets and has been analyzed in response to NRC questions (Ref. Duke letter to Rusche dated October 31, 1975 - Response to Question 2). The post-LOCA flood level in the Reactor Building is 785'-9 5/8".

The Reactor Building chemical spray environment is discussed in FSAR Supplement 6, Question 15 response and FSAR Supplement 7, Question 4 response.

The service conditions (i.e., temperature and pressure) for equipment located outside the Reactor Building which is exposed to a high energy line break (HELB) environment and which is required to mitigate the particular break is defined in the 1973 Oconee HELB Report (MDS Report No. 05-73.2).

The radiation analysis of areas outside the Reactor Building is done using a core inventory release of 100% noble gases, 50% iodines, and 1% fission products. This inventory is released into a water volume consisting of the Reactor Coolant System, Core Flood Tanks, and water injected by the HPI system. This method of determining the radiation levels is consistent with Regulatory Guide 1.7, TID 14844, and the October 31, 1979 clarifications to NUREG 0578.

#### 5.0 Qualification Methods

The method of qualification for the components listed in response to this bulletin is identified on the individual component evaluation work sheets.

It should be noted that the evaluation of the qualification programs for each generic type of equipment is in progress. Certain outstanding items (e.g., operating times and aging) must be resolved prior to completion of this evaluation. The schedule for resolving the outstanding items is as noted on the component evaluation work sheets.

#### 6.0 Margin

The margin between the service condition environment and the qualified environment for each component is as indicated on the individual component evaluation work sheets.

7.0 Aging

As mentioned in Paragraph 5.0 above, in the case of some equipments listed in the 90 day submittal, aging is an outstanding item that is still an ongoing effort as noted on the appropriate component evaluation work sheet for each specific equipment.

8.0 Documentation

Qualification documentation for the equipment identified on the systems/ components list will be available for audit at Ocone upon completion of the response to this bulletin.