

ENCLOSURE 1

NOTICE OF VIOLATION

Duke Power Company
Oconee 1, 2 and 3

Docket Nos. 50-269, 270 and 287
License Nos. DPR-38, 47, and 55

During an NRC inspection conducted on August 28 - September 24, 1994, three violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C, the violations are listed below:

- A. Technical Specification 6.4.1 requires that the Station be operated and maintained in accordance with approved procedures.

Modification procedure TN/1/A/2881/0/DL1, Unit 1 Power Battery Charger 1PA, 1PB, and 1PS Replacement, step 8.8.10 required that the internal wiring of breaker 1X0-F1AT be verified by performing a continuity test between the conductors.

Contrary to the above, step 8.8.10 of procedure TN/1/A/2881/0/DL1 was not performed prior to installing breaker 1X0-F1AT in motor control center 1X0. The breaker leads had been inadvertently reversed during the modification process and this resulted in the loss of motor control center 1X0 when the breaker compartment stabs contacted the bus bars due to a phase to phase short. Loss of 1X0 resulted in the loss of the majority of the Unit 1 radiation monitors, as well as pressurizer spray control.

This is a Severity Level IV violation and applies to Unit 1 (Supplement I)

- B. Technical Specification 6.4.1.c requires that approved written procedures with appropriate instructions shall be provided for actions taken to correct specific and foreseen potential malfunctions of systems or components involving nuclear safety.

Babcock & Wilcox Generic Emergency Operating Guidelines Bases, Section IV.FF - Forced Flow Cooldown, Part 5.2 (cooling down with a dry steam generator), lists the compressive limit for a B&W Once Through Steam Generator (OTSG) as 60 degrees fahrenheit - tubes hotter than shell.

Contrary to the above, from a time assumed to begin at initial operation until August 10, 1994, there was no procedural guidance at Oconee addressing the compressive limit for the OTSGs, components which are important to nuclear safety. This lack of procedural guidance resulted in the 3B OTSG exceeding the compressive limit by 22 degrees fahrenheit on August 10, 1994.

This is a Severity Level IV violation (Supplement I).

- C. 10 CFR Part 50, Appendix B, Criterion XVI, "Corrective Action," and the licensee's Quality Assurance Program (Duke-1-A, Section 17.3.2.13) require that measures be established to assure that conditions adverse to quality are promptly identified and corrected.

Contrary to the above, in a June 17, 1986 response to Babcock & Wilcox Owners Group Recommendation TR-032-ICS and in Licensee Event Report 270/93-01, the licensee identified a condition adverse to quality without taking prompt/appropriate corrective action. This adverse condition involved the repositioning of Turbine Bypass Valves (TBV) following a restoration of power to the Integrated Control System (ICS). The lack of prompt/appropriate corrective action resulted in the dry out of the 3B Once Through Steam Generator on August 10, 1994, when the Unit 3 TBVs randomly repositioned following ICS power restoration.

This is a Severity Level IV violation (Supplement I).

Pursuant to the provisions of 10 CFR 2.201, Duke Power Company is hereby required to submit a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D.C. 20555 with a copy to the Regional Administrator, Region II, and a copy to the NRC Resident Inspector at the Oconee Nuclear Plant, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. If an adequate reply is not received within the time specified in this Notice, an order or Demand for Information may be issued as to why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Dated at Atlanta, Georgia
this 20th day of October 1994