

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9601030063 DOC.DATE: 95/12/21 NOTARIZED: NO
FACIL:50-269 Oconee Nuclear Station, Unit 1, Duke Power Co.
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RECIP.NAME RECIPIENT AFFILIATION

DOCKET #
05000269

SUBJECT: LER 95-008-00:on 951129,containment isolation valve was inoperable.Caused by improper gear ratio.Corrected gear ratio during refueling outage.W/951221 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 2
TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES:

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AD4

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DUKE POWER

December 21, 1995

**U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555**

**Subject: Oconee Nuclear Station Unit
Docket Nos. 5-269, -270, -287
Licensee Event Report 269/95-08**

Gentlemen:

**Pursuant to 10 CFR 50.73 Sections (a) (1) and (d), attached is Licensee Event Report ,
269/95-08, concerning the inoperability of a containment isolation valve.**

**This report is being submitted in accordance with 10 CFR 50.73 (a) (2) (ii) (A). This
event is considered to be of no significance with respect to the health and safety of
the public.**

Very truly yours,

Joe M. Davis
for
J. W. Hampton

/fts

Attachment

**cc: Mr. S.D. Ebnetter
Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta St., NW, Suite 2900
Atlanta, GA 30323**

**INPO Records Center
700 Galleria Parkway
Atlanta, GA 30339-5957**

**Mr. L. A. Wiens
U.S. Nuclear Regulatory Commission
Office of Nuclear Reactor Regulation
Washington, D.C. 20555**

**Mr. P. E. Harmon
NRC Resident Inspector
Oconee Nuclear Station**

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ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Oconee Nuclear Station, Unit One	DOCKET NUMBER (2) 05000 269	PAGE (3) 1 of 1
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TITLE (4)
Containment Isolation Valve Inoperable

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER(S)
11	29	95	95	08	00	12	21	95		05000

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR (Check one or more of the following) (11)										
POWER LEVEL (10) 000	<input type="checkbox"/>	20.402(b)	<input type="checkbox"/>	20.405(c)	<input type="checkbox"/>	50.73(a)(2)(iv)	<input type="checkbox"/>	73.71(b)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
	<input type="checkbox"/>	20.405(a)(1)(i)	<input type="checkbox"/>	50.36(c)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	<input type="checkbox"/>	73.71(c)			
	<input type="checkbox"/>	20.405(a)(1)(ii)	<input type="checkbox"/>	50.36(c)(2)	<input type="checkbox"/>	50.73(a)(2)(vii)	<input type="checkbox"/>				
	<input type="checkbox"/>	20.405(a)(1)(iii)	<input type="checkbox"/>	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)	<input type="checkbox"/>				
	<input type="checkbox"/>	20.405(a)(1)(iv)	<input checked="" type="checkbox"/>	50.73(a)(2)(ii)(A)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)	<input type="checkbox"/>				
<input type="checkbox"/>	20.405(a)(1)(v)	<input type="checkbox"/>	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(x)	<input type="checkbox"/>					

LICENSEE CONTACT FOR THIS LER (12)								TELEPHONE NUMBER			
NAME Lanny V. Wilkie, Safety Review Manager								AREA CODE (864)		885-3518	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input checked="" type="checkbox"/>	YES (If yes, complete EXPECTED SUBMISSION DATE)			<input type="checkbox"/>	NO	02	19	96

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

DATE: November 29, 1995 Unit Status: Refueling Shutdown

During testing, Reactor Coolant System valve 1RC-6 was found to have a different stroke time than was expected. Valve 1RC-6 is the containment isolation valve for the Pressurizer (PZR) water space sampling.

Subsequent analysis determined that the actual gear ratio for this valve was different than what was assumed in the test setup calculation. Further analysis indicated that the stroke length had previously been assumed to be the same as the valve size. The actual stroke length was more than the valve size. When the corrected gear ratio was combined with the actual stroke length, the stroke time was calculated. Since the actual stroke time was different from the newly calculated time, an investigation was initiated.

On December 19, 1995, at 1530 hours, it was determined that valve 1RC-6 would not have been able to close against the required differential pressure during the time periods in which a PZR sample was being taken. This was due to the improper gear ratio. The gear ratio was corrected during the refueling outage and the valve stroke tested satisfactorily.

This report will be supplemented to address root cause and other LER content requirements after further investigation is complete.