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DUKE POWER

June 29, 1990

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Subject: Oconee Nuclear Station, Units 1, 2, and 3
Docket Nos. 50-269, -270, -287
McGuire Nuclear Station, Units 1 and 2
Docket Nos. 50-369, -370
Catawba Nuclear Station, Units 1 and 2
Docket Nos. 50-413, 414
Status of Licensee Implementation of Generic
Safety Issues Resolved With Imposition of
Requirements or Corrective Actions
Response to Generic Letter 90-04

Gentlemen:

On April 25, 1990 the NRC issued Generic Letter 90-04 requesting I review and provide documentation of the current implementation status of all generic safety issues (GSIs) identified within the generic letter as applicable Duke's nuclear stations. This review has been completed and I am reporting the results (Attachments) in a format similar to that in Enclosure 1 to the subject generic letter.

Attachments 1, 2 and 3 provide status summary for each GSI with appropriate cross-references to other docketed transmittals containing further details for Oconee Nuclear Station, McGuire Nuclear Station, and Catawba Nuclear Station, respectively. In that regard, this transmittal serves merely as an index and does not contain any new or revised commitments. The NRC staff should treat this response accordingly.

If there are questions or comments on this material, please direct these to the appropriate member of my Regulatory Compliance staff.

Very truly yours,

Hal B. Tucker

Hal B. Tucker

MAH/103/lcs

Attachments

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PDR ADDOCK 05000269
FDC

Alors
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Catawba Nuclear Station

Attachment 1

Duke Power Company
Oconee Nuclear Station

Response to Generic Letter 90-04

Status of Licensee Implementation of
Generic Safety Issues Resolved With
Imposition of Requirements or Corrective Actions
Response To Generic Letter 90-04

FACILITY NAME: Oconee Nuclear Station, Units 1,2, and 3
DOCKET NO.: 50-269, -270, -287
LICENSEE: Duke Power Company

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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	NA	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	NA	
43 (B107)	Reliability Of Air Systems	All Plants	I (01/31/91)	NRC GL 88-14, Bulletin 79-24. DPC Letters 10/30/79, 5/8/89 and 7/21/89. Modifications to Air System in process. Will notify NRC when system is operational and in service.
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I	NRC GL 89-13, GL 89-13, Supp. 1. DPC Letters 1/26/90, 5/31/90 describe actions taken and planned in response to NRC requirements. Supplements will be provided as committed.
67.3.3 (A017)	Improved Accident Monitoring	All Plants	C	DPC Letter 4/20/88, 5/15/88. NRC SERs 3/15/88, 12/19/88, one outstanding parameter (EQ aspect of CFT pressure and level instruments) pending NRC review and resolution of generic considerations.
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C	DPC Letter 11/4/83 NRC SER 5/15/85
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	C	DPC Letters 11/4/83, 2/27/86. NRC SER 9/11/86 DPC Letter 10/1/86 resolved the outstanding item contained in the SER.

Status of Licensee Implementation of
Generic Safety Issues Resolved With
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Response To Generic Letter 90-04

FACILITY NAME: Oconee Nuclear Station, Units 1,2, and 3
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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	C	DPC Letters 11/4/83, 8/9/85, 8/30/85, and 6/9/87. NRC SERs 9/28/87 and 11/11/87.
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C	DPC Letters 11/4/83, 1/17/84 and 6/9/87. NRC SER 11/4/87.
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	C	DPC Letter 11/4/83 pending NRC approval. Also, NRC GL 90-03 dated 3/20/90 under evaluation. DPC response to GL 90-03 due on 9/18/90.
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	All Plants	C	DPC Letters 11/4/83, 8/10/84, 2/21/85, 3/20/85, 3/29/85, 4/25/85 and 2/25/86. NRC SER 10/11/85.
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	C	DPC Letter 11/4/83. NRC SER 7/11/85.
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	C	DPC Letter 11/4/83. NRC SER 7/11/85.
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	DPC Letter 11/4/83. NRC SER 7/11/85.

Status of Licensee Implementation of
Generic Safety Issues Resolved With
Imposition of Requirements or Corrective Actions
Response To Generic Letter 90-04

FACILITY NAME: Oconee Nuclear Station, Units 1,2, and 3
DOCKET NO.: 50-269, -270, -287
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<u>GSII/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	C	DPC Letter 11/4/83. NRC SER 7/11/85.
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability-Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C	DPC Letters 11/4/83, 2/21/85, 3/29/85 and 5/24/85. NRC SER dated 9/6/85.
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C	DPC Letters 11/4/83, 12/30/83, 10/23/84, 3/20/85. NRC SER 2/11/85.
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	All W & B&W Plants	C	DPC Letters 11/4/83, 12/30/83, 10/23/84, 3/20/85, 9/17/85 and 3/25/86. NRC SER 2/11/85, NRC GL 85-10 dated 5/23/85, letter to B&WOG 12/6/85 and amendment Nos. 148, 148, 145 dated 8/20/86.
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	C	DPC Letters 11/4/83, 8/9/85 and 12/2/85. NRC SER dated 11/19/86.
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	C	DPC Letters 11/4/83, 8/10/84, 2/21/85, 3/20/85, 3/29/85 and 4/25/85. NRC SER 10/11/85.

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<u>GSII/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	C	DPC letter 11/4/83. NRC SERs dated 4/27/87 and 2/25/89 on Item 4.5.2.
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	NA	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	C	DPC Letter 5/25/88 in response to NRC GL 88-03 pending review by the NRC.
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	I	DPC Letters 1/3/89, 2/2/89 and 10/2/87 provide response to GL 88-17 and GL 87-12 and description of actions taken which are under review by the NRC. One outstanding (GL 88-17) modification will be installed during a RFO for each unit. Interlock already included in Oconee design and as with other B&W designed plants, Oconee is not sensitive to loss of decay heat removal problem.
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoun	NA	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	C	NRC Letters 10/20/80, 10/19/81 Amendment Nos. 101, 101, 98. DPC Letter 6/3/81.

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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	C	NRC Letters 10/20/80, 10/19/81 Amendment Nos. 101, 101, 98. DPC Letter 6/3/81.
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	NA	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	C/E	NRC SER 3/21/83 completed this issue. However, recent problems have been identified (LER 269/90-04 and LER 269/90-05). Corrective actions will be taken as committed by these LERs.
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	NA	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NA	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C	NRC Order 4/20/81 (Amendments OM4/OM4/OM4) Also implemented thru Tech Spec revisions and Oconee ISI program, Amendment Nos. 109, 109 and 106.

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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
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*NOTE

The following guidance was used to complete the status column.

- (1) Provide a separate entry for each licensed reactor unit. If the information is identical for multiple units, so state.
- (2) If a GSI is not applicable to a unit(s), enter "NA".
- (3) If a GSI is applicable but no changes were necessary to implement the resolution, enter "NC". If the GSI implementation was completed prior to issuance of the operating license, enter "NC", as no post-licensing changes were necessary.
- (4) If a GSI is applicable, submittal of information and/or changes were necessary and such submittals were made or changes are complete, enter "C". Also identify the licensee's document(s) to the NRC which certified completion, and the document date(s).
- (5) If a GSI is applicable and changes are necessary but such changes are not yet fully implemented, enter "I" and the projected month and year of completion. Provide a brief explanation of the outstanding work in the "Comments" column.
- (6) If implementation guidance for a resolved GSI was issued recently and the licensee is still evaluating the appropriate response, enter "E" and the projected response date.
- (7) The "Comments" column may be used to explain any entry in the "Status" column.

** NOTE

The 16 items listed for GSI 75 all relate to actions derived from the generic implications of Salem ATWS events. Item numbers correspond to Generic Letter 83-28 action item numbers.

Attachment 2

Duke Power Company
McGuire Nuclear Station

Response to Generic Letter 90-04

Status of Licensee Implementation of
Generic Safety Issues Resolved With
Imposition of Requirements or Corrective Actions
Response To Generic Letter 90-04

FACILITY NAME: McGuire Nuclear Station, Units 1 and 2
DOCKET NO.: 50-369, 370
LICENSEE: Duke Power Company

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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	NA	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	NA	
43 (B107)	Reliability Of Air Systems	All Plants	I-1/91	DPC Letter 5/8/89 response to G.L. 88-14, Attachment 5, Action Item 4. Remains to be completed.
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I	DPC Letters 1/26/90 and 5/31/90 responses to GL 89-13 describe actions taken and planned in response to NRC requirements. Supplements will be provided as committed.
67.3.3 (A017)	Improved Accident Monitoring	All Plants	C	NRC SER 3/23/87, two parameters remain outstanding pending NRC resolution of generic considerations.
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C	NRC SER 5/30/85
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	C	NRC SER 6/21/85 DPC Letter 10/1/86 resolved the outstanding item contained in the SER.
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	C	NRC SER's 7/16/86 and 12/29/86

Status of Licensee Implementation of
Generic Safety Issues Resolved With
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FACILITY NAME: McGuire Nuclear Station, Units 1 and 2
DOCKET NO.: 50-369, 370
LICENSEE: Duke Power Company

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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C	NRC SER 10/14/87
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	C	DPC Letter 12/3/85 pending NRC approval. Also, NRC GL 90-03 dated 3/20/90 under evaluation. DPC response to GL 90-03 due on 9/18/90.
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	All Plants	C	NRC SER 10/31/85
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	C	NRC SER 10/31/85
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	C	NRC SER 10/31/85
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	NRC SER 10/31/85
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	C	NRC SER 10/31/85

Status of Licensee Implementation of
Generic Safety Issues Resolved With
Imposition of Requirements or Corrective Actions
Response To Generic Letter 90-04

FACILITY NAME: McGuire Nuclear Station, Units 1 and 2
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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability-Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C	NRC SER 10/20/87 (MNS FSAR Section 1.9, Reference No. 23)
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C	Modifications completed 7/85.
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	All W & B&W Plants	C	License Amendment No. 74/55, NRC Letter 9/2/87, Approved Tech Spec changes in accordance with GL 83-28, Section 4.3.
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	NA	
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	C	NRC SER 10/31/85
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	C	NRC SER 1/28/87 NRC SER 6/29/89

Status of Licensee Implementation of
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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	NA	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	C	DPC Letter 5/26/88. NRC Letter 6/8/88
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	I-11/91	DPC Letters 10/2/87, 1/3/89 and 2/2/89 provided response to GL 87-12 and GL 88-17. Tech Spec submittals are scheduled for 7/90. Modifications will be installed during a future RFO for each unit.
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoun	NA	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	C	License Amendment No. 86/67, NRC Letter 6/6/88 approved Technical Specification changes in accordance with G.L. 84-13.
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	C	License Amendment No. 86/67, NRC Letter 6/6/88 approved Technical Specification changes in accordance with G.L. 84-13.
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	NA	

Status of Licensee Implementation of
Generic Safety Issues Resolved With
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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	NC	MNS SER dated March 1978 MNS SER, Supplement 1, dated May 1978
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	NA	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NA	MNS OL Application 6/7/74 Compliance with Regulatory Guide 1.52 is discussed in the MNS FSAR Sections 1.7 and 6.2.3.2.
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	NA	Order did not apply to plants under construction. Present Tech Specs include low pressure systems connected to the Reactor Coolant System Pressure Boundary.

*NOTE

The following guidance was used to complete the status column.

- (1) Provide a separate entry for each licensed reactor unit. If the information is identical for multiple units, so state.
- (2) If a GSI is not applicable to a unit(s), enter "NA".
- (3) If a GSI is applicable but no changes were necessary to implement the resolution, enter "NC". If the GSI implementation was completed prior to issuance of the operating license, enter "NC", as no post-licensing changes were necessary.

Status of Licensee Implementation of
Generic Safety Issues Resolved With
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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
(4)	If a GSI is applicable, submittal of information and/or changes were necessary and such submittals were made or changes are complete, enter "C". Also identify the licensee's document(s) to the NRC which certified completion, and the document date(s).			
(5)	If a GSI is applicable and changes are necessary but such changes are not yet fully implemented, enter "I" and the projected month and year of completion. Provide a brief explanation of the outstanding work in the "Comments" column.			
(6)	If implementation guidance for a resolved GSI was issued recently and the licensee is still evaluating the appropriate response, enter "E" and the projected response date.			
(7)	The "Comments" column may be used to explain any entry in the "Status" column.			

** NOTE

The 16 items listed for GSI 75 all relate to actions derived from the generic implications of Salem ATWS events. Item numbers correspond to Generic Letter 83-28 action item numbers.

Attachment 3

Duke Power Company
Catawba Nuclear Station

Response to Generic Letter 90-04

Status of Licensee Implementation of
Generic Safety Issues Resolved With
Imposition of Requirements or Corrective Actions
Response To Generic Letter 90-04

FACILITY NAME: Catawba Nuclear Station, Units 1 and 2
DOCKET NO.: 50-413 and 50-414
LICENSEE: Duke Power Company

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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In The BWR Scram System	All BWRs	NA	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	NA	
43 (B107)	Reliability Of Air Systems	All Plants	I - 1/91	DPC Letter 5/8/89 response to GL 88-14. Attachment 5, Action Item 11 remains to be completed.
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I	DPC Letters 1/26/90 and 5/31/90 responses to GL 89-13 describe actions taken and planned in response to NRC requirements. Supplements will be provided as committed.
67.3.3 (A017)	Improved Accident Monitoring	All Plants	C	NUREG 0954, Supp. 5. Two parameters remain outstanding pending NRC resolution of generic considerations.
75** (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C	NRC SER 6/21/85
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	C	NRC SER 6/21/85 DPC Letter 10/1/86 resolved the outstanding item contained in the SER.
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	C	NRC SERs 7/16/86 and 6/10/87

Status of Licensee Implementation of
Generic Safety Issues Resolved With
Imposition of Requirements or Corrective Actions
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FACILITY NAME: Catawba Nuclear Station, Units 1 and 2
DOCKET NO.: 50-413 and 50-414
LICENSEE: Duke Power Company

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C	NRC SER 10/26/87
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	C	NRC SER 10/26/87. Also, NRC GL 90-03 dated 3/20/90 under evaluation. DPC response to GL 90-03 due on 9/18/90.
75 (B078)	Items 3.1.1 & 3.1.2 - Post - Maintenance Testing (Reactor Trip System Components)	All Plants	C	NRC SER 7/29/87
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	C	NUREG 0954, Supp. 5
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	C	NRC SER 7/29/87
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	NUREG 0954, Supp. 5
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	C	NRC SER 10/26/87

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Generic Safety Issues Resolved With
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<u>GSII/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability-Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C	NRC SER 10/26/87
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C	NUREG 0954, Supp. 4
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment For Westinghouse and B&W Plants)	All W & B&W Plants	C	NRC Letter 5/9/89
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	NA	
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	C	NRC SER 7/29/87
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	C	NRC SERs 1/27/87 and 7/21/89

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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	NA	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	C	DPC Letters 5/26/88, 11/29/88. NRC Letter 12/2/88
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	I - 12/91	DPC Letters 10/2/87, 1/3/89, and 2/2/89 provided response to GL 87-12 and GL 88-17. Tech Spec changes will be submitted for NRC approval.
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft. Calhoun	NA	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	NC	Completed during licensing process.
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	NC	Completed during licensing process.
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	NA	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	NC	NUREG 0954. Topic addressed during licensing process.

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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	NA	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NC	Completed during licensing process. Compliance with Regulatory Guides 1.52 and 1.140 is discussed in CNS FSAR Section 1.8.
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	NA	Order did not apply to plants under construction. Present Tech Specs include low pressure systems connected to the Reactor System pressure boundary.

*NOTE

The following guidance was used to complete the status column.

- (1) Provide a separate entry for each licensed reactor unit. If the information is identical for multiple units, so state.
- (2) If a GSI is not applicable to a unit(s), enter "NA".
- (3) If a GSI is applicable but no changes were necessary to implement the resolution, enter "NC". If the GSI implementation was completed prior to issuance of the operating license, enter "NC", as no post-licensing changes were necessary.

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<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
(4)	If a GSI is applicable, submittal of information and/or changes were necessary and such submittals were made or changes are complete, enter "C". Also identify the licensee's document(s) to the NRC which certified completion, and the document date(s).			
(5)	If a GSI is applicable and changes are necessary but such changes are not yet fully implemented, enter "I" and the projected month and year of completion. Provide a brief explanation of the outstanding work in the "Comments" column.			
(6)	If implementation guidance for a resolved GSI was issued recently and the licensee is still evaluating the appropriate response, enter "E" and the projected response date.			
(7)	The "Comments" column may be used to explain any entry in the "Status" column.			

** NOTE

The 16 items listed for GSI 75 all relate to actions derived from the generic implications of Salem ATWS events. Item numbers correspond to Generic Letter 83-28 action item numbers.