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50-269 Oconee Nuclear Station, Unit 1, Duke Power Co.			05000269
50-270 Oconee Nuclear Station, Unit 2, Duke Power Co.			05000270
50-287 Oconee Nuclear Station, Unit 3, Duke Power Co.			05000287
50-369 William B. McGuire Nuclear Station, Unit 1, Duke Powe			05000369
50-370 William B. McGuire Nuclear Station, Unit 2, Duke Powe			05000370
50-413 Catawba Nuclear Station, Unit 1, Duke Power Co.			05000413
50-414 Catawba Nuclear Station, Unit 2, Duke Power Co.			05000414

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SUBJECT: Responds to Generic Ltr 89-21 re status of implementation of  
 USI requirements. S

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**DUKE POWER**

November 29, 1989

U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Attention: Document Control Desk

Subject: Oconee Nuclear Station  
Docket Nos. 50-269, 270, and 287  
McGuire Nuclear Station  
Docket Nos. 50-369 and 370  
Catawba Nuclear Station  
Docket Nos. 50-413 and 414  
Status of Implementation of  
Unresolved Safety Issue (USI) Requirements  
Response to Generic Letter 89-21

On October 19, 1989 the NRC issued Generic Letter 89-21 requesting I provide a review and report of the status of USI implementation at the Duke Power nuclear stations. This review has been completed and I am reporting the results on the attached Enclosure 1 sheets obtained from the generic letter.

The attached information provides a status summary for each USI with appropriate cross-references to other docketed transmittals containing further details. In that regard, this transmittal serves merely as an index and does not contain any new or revised commitments. The NRC staff should treat this response accordingly.

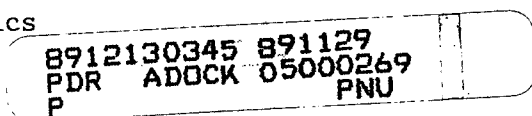
In regards to USI A-40, the staff does not require implementation of any specific action for the Oconee Nuclear Station. However, any future regulatory requirements concerning the USI A-40 resolution should consider integration of such requirements with those of USI A-46 for implementation as required by Generic Letter 87-02 to avoid duplication of efforts.

If there are questions or comments on this material, please direct these to the appropriate member of my Regulatory Compliance Staff.

Very truly yours,

Hal B. Tucker

JSW370/lcs



A012  
11

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ATTACHMENT

DUKE POWER COMPANY  
OCONEE NUCLEAR STATION  
McGUIRE NUCLEAR STATION  
CATAWBA NUCLEAR STATION

STATUS OF IMPLEMENTATION OF UNRESOLVED SAFETY  
ISSUE (USI) REQUIREMENTS

RESPONSE TO GENERIC LETTER 89-21

UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL TECHNICAL RESOLUTION HAS BEEN ACHIEVED

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-1	Water Hammer	SECY 84-119 NUREG-0927, Rev. 1 NUREG-0993, Rev. 1 NUREG-0737 Item I.A.2.3 SRP revisions	All	C/5-87	No Hardware Or Design Change. Included To Training Program. Submitted To NRC on 5/18/87 As Response To Item I.A.2.3.
A-2/ MPA D-10	Asymmetric Blowdown Loads on Reactor Primary Coolant Systems	NUREG-0609 GL 84-04, GDC-4	PWR	C	See SER Dated 5/20/84 Assessment of Asymmetric LOCA Loads Accepted. Duke Letter Dated 2/19/86 Informed NRC That Cavity Seal Ring Will Be Stored In The Reactor Cavity General Area During Normal Operation.
A-3	Westinghouse Steam Generator Tube Integrity	NUREG-0844 SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	W-PWR	NA	
A-4	CE Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	CE-PWR	NA	
A-5	B&W Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No Requirements)	B&W-PWR	I	See Duke Response To GL 85-02 Dated 7/17/85. Submitted A Change (5/31/88) To Tech Spec's To Specify SG Tube Leak Limits. Awaiting NRC Review And Approval
E A-6	Mark I Containment Short-Term Program	NUREG-0408	Mark I-BWR	NA	

- \* C - COMPLETE
- NC - NO CHANGES NECESSARY
- NA - NOT APPLICABLE
- I - INCOMPLETE
- E - EVALUATING ACTIONS REQUIRED

DUKE POWER COMPANY  
 OCONEE NUCLEAR STATION  
 UNITS 1, 2, AND 3  
 DOCKET NOS. 50-269, -270, -287

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-7/ D-01	Mark I Long-Term Program	NUREG-0661 NUREG-0661 Suppl. 1 GL 79-57	Mark I-BWR	NA	
A-8	Mark II Containment Pool Dynamic Loads	NUREG-0808 NUREG-0487, Suppl. 1/2 NUREG-0802 SRP 6.2.1.1C GDC 16	Mark II-BWR	NA	
A-9	Anticipated Transients Without Scram	NUREG-0460, Vol. 4 10 CFR 50.62	All	I/8-91	AMSAC/DSS System To Be Installed by 8/91. See Duke Letter Dated 8/4/89..
A-10/ MPA B-25	BWR Feedwater Nozzle Cracking	NUREG-0619 Letter from DG Eisenhut dated 11/13/80 GL 81-11	BWR	NA	
A-11	Reactor Vessel Material Toughness	NUREG-0744, Rev. 1 10 CFR 50.60/ 82-26	All	NC	No response required. B&WOG Approved Methodologies For RSVP Are Used..
A-12	Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports	NUREG-0577, Rev. 1 SRP Revision 5.3.4	PWP	NA	Applies Only To Plants With Construction Permits Issued After October 1983..
A-17	Systems Interactions	Ltr: DeYoung to licensees - 9/72 NUREG-1174, NUREG-1229, NUREG/CR-3922, NUREG/CR-4261, NUREG/CR-4470, GL 89-18 (No requirements)	All	NC	No Action Required.
A-24/ MPA B-60	Qualification of Class 1E Safety-Related Equipment.	NUREG-0588, Rev. 1 SRP 3.11 10 CFR 50.49 GL 82-09, GL 84-24 GL 85-15	All	C/1-85	See Duke Certification Letter Dated 1/28/85 And NRC SER Dated 9/22/86..

DUKE POWER COMPANY  
 OCONEE NUCLEAR STATION  
 UNITS 1, 2, AND 3  
 DOCKET NOS. 50-269, -270, -287

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-26/ MPA B-04	Reactor Vessel Pressure Transient Protection	DOP Letters to Licensees 8/76 NUREG-0224 NUREG-0371 SRP 5.2 GL 88-11	PWR	I/11-89	LTOP Implemented. See SER Dated 8/8/83. Response To GL 88-11 Provided On 1/31/89. LTOP & PT Tech Spec Sub- mitted By A Duke Letter Dated 11/15/89.
A-31	Residual Heat Removal Shutdown Requirements	NUREG-0606 RG 1.113, RG 1.139 SRP 5.4.7	All OLs After 01/79.	NA	For Plants With OL After 01/79. Oconee OL Prior To 1979.
A-36/ C-10, C-15	Control of Heavy Loads Near Spent Fuel	NUREG-0612 SRP 9.1.5 GL 81-07, GL 83-42, GL 85-11 Letter from DG Eisenhut dated 12/22/80	All	C/4-83	Duke Responded To Phase I See SER Dated 4/20/83. GL 85-11 Considered Phase II Completed. Duke Letter 10/2/87 Will Implement Phase II Commitments If Considered Needed. NRC Letter 11/2/87 - No Objection.
A-39	Determination of SRV Pool Dynamic Loads and Pressure Transients	NUREG-0802 NUREGs-0763,0783,0802 NUREG-0661 SRP 6.2.1.1.C	BWR	NA	
A-40	Seismic Design Criteria	SRP Revisions, NUREG/ CR-4776, NUREG/CR-0054, NUREG/CR-3480, NUREG/ CR-1582, NUREG/CR-1161, NUREG-1233, NUREG-4776 NUREG/CR-3805 NUREG/CR-5347 NUREG/CR-3509	All	NA	No Action Required For Operating Plants. Duke Requests That The NRC Integrate The Implementation Of The Actions For This Issue With The Actions Being Taken For USI A-46 Per Generic Letter 87-02.
A-42/ MPA B-05	Pipe Cracks in Boiling Water Reactors	NUREG-0313, Rev. 1 NUREG-0313, Rev. 2 GL 81-03, GL 88-01	BWR	NA	

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-43	Containment Emergency Sump Performance	NUREG-0510, NUREG-0869, Rev. 1 NUREG-0897, R.G.1.8? (Rev. 0), SRP 6.2.2 GL 85-22 No Requirements	All	NC	Not Required For Operating Plants.
A-44	Station Blackout	RG 1.155 NUREG-1032 NUREG-1109 10 CFR 50.63	All	I	Submitted Evaluation On 4/17/89. Awaiting NRC Review.
A-45	Shutdown Decay Heat Removal Requirements	SECY 88-260 NUREG-1289 NUREG/CR-5230 SECY 88-260 (No requirements)	All	E-12/90	Will Subsume This Issue Into The IPE Program. H. B. Tucker Letter Dated 10/30/89.
A-46	Seismic Qualification of Equipment in Operating Plants	NUREG-1030 NUREG-1211/ GL 87-02, GL 87-03	All	E	Duke Will Adopt SQUG Program. Implementation Based On Final NRC SER On SQUG GIP. See Duke Letter 10/07/88.
A-47	Safety Implication of Control Systems	NUREG-1217, NUREG-1218 GL 89-19	All	E	Response Due On 3/19/90.
A-48	Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment	10 CFR 50.44 SECY 89-122	All, except PWRs with large dry containments	NA	No Ice Condensers At ONS.
A-49	Pressurized Thermal Shock	RGs 1.154, 1.99 SECY 82-465 SECY 83-288 SECY 81-687 10 CFR 50.61/ GL 88-11	PWR	C/3-87	See NRC SER Dated 3/27/87. See Duke Letter 1/31/89 In Response To GL 88-11. All Units Meet PTS Screening Criteria.



UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL TECHNICAL RESOLUTION HAS BEEN ACHIEVED

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-1	Water Hammer	SECY 84-119 NUREG-0927, Rev. 1 NUREG-0993, Rev. 1 NUREG-0737 Item I.A.2.3 SRP revisions	All	NC	H. B. Tucker Letters Dated 5/18/87 & 9/11/87 Provided Certification of Training Programs
A-2/ MPA D-10	Asymmetric Blowdown Loads on Reactor Primary Coolant Systems	NUREG-0609 GL 84-04, GDC-4	PWR	NC	MNS SER, Supplement No. 3 Dated May 1980
A-3	Westinghouse Steam Generator Tube Integrity	NUREG-0844 SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	W-PWR	C-7/85	H. B. Tucker Letter Dated 7/17/85 Provided Response to G.L. 85-02
A-4	CE Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	CE-PWR	NA	
A-5	B&W Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No Requirements)	B&W-PWR	NA	
E A-6	Mark I Containment Short-Term Program	NUREG-0408	Mark I-BWR	NA	

- \* C - COMPLETE  
 NC - NO CHANGES NECESSARY  
 NA - NOT APPLICABLE  
 I - INCOMPLETE  
 E - EVALUATING ACTIONS REQUIRED

DUKE POWER COMPANY  
 McGUIRE NUCLEAR STATION  
 UNITS 1 AND 2  
 DOCKET NOS. 369 AND 370

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-7/ D-01	Mark I Long-Term Program	NUREG-0661 NUREG-0661 Suppl. 1 GL 79-57	Mark I-BWR	NA	
A-8	Mark II Containment Pool Dynamic Loads	NUREG-0808 NUREG-0487, Suppl. 1/2 NUREG-0802 SPP 6.2.1.1C GDC 16	Mark II-BWR	NA	
A-9	Anticipated Transients Without Scram	NUREG-0460, Vol. 4 10 CFR 50.62	All	C-12/88	NRC SER Dated 11/6/87
A-10/ MPA B-25	BWR Feedwater Nozzle Cracking	NUREG-0619 Letter from DG Eisenhut dated 11/13/80 GL 81-11	BWR	NA	
A-11	Reactor Vessel Material Toughness	NUREG-0744, Rev. 1 10 CFR 50.60/ 82-26	All	NC	NRC Letter Dated 10/21/86
A-12	Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports	NUREG-0577, Rev. 1 SRP Revision 5.3.4	PWP	NA	Only Applicable to CP's Issued After 10/83
A-17	Systems Interactions	Ltr: DeYoung to Licensees - 9/72 NUREG-1174, NUREG-1229, NUREG/CR-3922, NUREG/CR-4261, NUREG/CR-4470, GL 89-18 (No requirements)	All	NC	No Actions Required
A-24/ MPA B-60	Qualification of Class 1E Safety-Related Equipment	NUREG-0588, Rev. 1 SRP 3.11 10 CFR 50.49 GL 82-09, GL 84-24 GL 85-15	All	C-1/85	H. B. Tucker Letter Dated 1/28/85 Provided Certification of Compliance With 10CFR 50.49

DUKE POWER COMPANY  
 McGUIRE NUCLEAR STATION  
 UNITS 1 AND 2  
 DOCKET NOS. 369 AND 370

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-26/ MPA B-04	Reactor Vessel Pressure Transient Protection	DOP Letters to Licensees 8/76 NUREG-0224 NUREG-0371 SRP 5.2 GL 88-11	PWR	C-7/89	NRC Letter Dated 7/18/89 Issued T.S. Change
A-31	Residual Heat Removal Shutdown Requirements	NUREG-0606 RG 1.113, RG 1.139 SRP 5.4.7	All Ols After 01/79.	C-3/78	MNS SER Section 5.3.3 Dated March 1978 and MNS SER Supplement No. 3 Dated May 1980
A-36/ C-10, C-15	Control of Heavy Loads Near Spent Fuel	NUREG-0612 SRP 9.1.5 GL 81-07, GL 83-42, GL 85-11 Letter from DG Eisenhut dated 12/22/80	All	C-3/85	NRC Letter Dated 3/12/85
A-39	Determination of SRV Pool Dynamic Loads and Pressure Transients	NUREG-0802 NUREGs-0763,0783,0802 NUREG-0661 SRP 6.2.1.1.C	BWR	NA	
A-40	Seismic Design Criteria	SRP Revisions, NUREG/ CR-4776, NUREG/CR-0054, NUREG/CR-3480, NUREG/ CR-1582, NUREG/CR-1161, NUREG-1233, NUREG-4776 NUREG/CR-3805 NUREG/CR-5347 NUREG/CR-3509	All	NC	NRC Approval of Seismic Design, MNS SER Dated March 1978, USI A-40 Discussed in MNS SER, Supplement No. 3 Dated May 1980
A-42/ MPA B-05	Pipe Cracks in Boiling Water Reactors	NUREG-0313, Rev. 1 NUREG-0313, Rev. 2 GL 81-03, GL 88-01	BWR	NA	

DUKE POWER COMPANY  
 McGUIRE NUCLEAR STATION  
 UNITS 1 AND 2  
 DOCKET NOS. 369 AND 370

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-43	Containment Emergency Sump Performance	NUREG-0510, NUREG-0869, Rev. 1 NUREG-0897, R.G.1.8? (Rev. 0), SRP 6.2.2 GL 85-22 No Requirements	All	NC	No Licensee Actions Required. Only Affects Applications Filed After 1985. MNS SER, Supplement No. 3 May 1980
A-44	Station Blackout	RG 1.155 NUREG-1032 NUREG-1109 10 CFR 50.63	All	I	H. B. Tucker Letter Dated 4/17/89, Awaiting NRC Approval
A-45	Shutdown Decay Heat Removal Requirements	SECY 88-260 NUREG-1289 NUREG/CR-5230 SECY 88-260 (No requirements)	All	E-12/91	H. B. Tucker Letter Dated 11/1/89, Response to G.L. 88-20, IPE
A-46	Seismic Qualification of Equipment in Operating Plants	NUREG-1030 NUREG-1211/ GL 87-02, GL 87-03	All	NA	NRC Approval of Seismic Design, MNS SER Dated March 1978, Only Applicable to Older Plants.
A-47	Safety Implication of Control Systems	NUREG-1217, NUREG-1218 GL 89-19	All	E-3/90	Response to G.L. 89-19 Required by 3/19/90
A-48	Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment	10 CFR 50.44 SECY 89-122	All, except PWRs with large dry containments	I	H. B. Tucker Letter Dated 1/4/89 Awaiting NRC Approval, Containment Response Analysis to Follow
A-49	Pressurized Thermal Shock	RGs 1.154, 1.99 SECY 82-465 SECY 83-288 SECY 81-687 10 CFR 50.61/ GL 88-11	PWR	C-1/86	H. B. Tucker Letter Dated 1/15/86. NRC Letter Dated 10/21/86

UNRESOLVED SAFETY ISSUES FOR WHICH A FINAL TECHNICAL RESOLUTION HAS BEEN ACHIEVED

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-1	Water Hammer	SECY 84-119 NUREG-0927, Rev. 1 NUREG-0993, Rev. 1 NUREG-0737 Item I.A.2.3 SRP revisions	All	NC	H. B. Tucker Letter's Dated 5/18/87 & 9/11/87 Provided Certification Of Training Programs.
A-2/ MPA D-10	Asymmetric Blowdown Loads on Reactor Primary Coolant Systems	NUREG-0609 GL 84-04, GDC-4	PWR	NC	CNS SER Dated February 1983.
A-3	Westinghouse Steam Generator Tube Integrity	NUREG-0844 SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	W-PWR	C-7/85	H. B. Tucker Letter Dated 7/17/85 Provided Response to G. L. 85-02
A-4	CE Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No requirements)	CE-PWR	NA	---
A-5	B&W Steam Generator Tube Integrity	NUREG-0844, SECY 86-97 SECY 88-272 GL 85-02 (No Requirements)	B&W-PWR	NA	---
E A-6	Mark I Containment Short-Term Program	NUREG-0408	Mark I-BWR	NA	---

- \* C - COMPLETE  
 NC - NO CHANGES NECESSARY  
 NA - NOT APPLICABLE  
 I - INCOMPLETE  
 E - EVALUATING ACTIONS REQUIRED

DUKE POWER COMPANY  
 CATAWBA NUCLEAR STATION  
 UNITS 1 AND 2  
 DOCKET NOS. 50-413 AND 414

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-7/ D-01	Mark I Long-Term Program	NUREG-0661 NUREG-0661 Suppl. 1 GL 79-57	Mark I-BWR	NA	---
A-8	Mark II Containment Pool Dynamic Loads	NUREG-0808 NUREG-0487, Suppl. 1/2 NUREG-0802 SRP 6.2.1.1C GDC 16	Mark II-BWR	NA	---
A-9	Anticipated Transients Without Scram	NUREG-0460, Vol. 4 10 CFR 50.62	All	C-9/89	NRC SER Dated 11/6/87
A-10/ MPA B-25	BWR Feedwater Nozzle Cracking	NUREG-0619 Letter from DG Eisenhut dated 11/13/80 GL 81-11	BWR	NA	---
A-11	Reactor Vessel Material Toughness	NUREG-0744, Rev. 1 10 CFR 50.60/ 82-26	All	NC-7/84	CNS SER Dated February 1983 Concluded That The Catawba Units Meet 10 CFR 50.60 Requirements.
A-12	Fracture Toughness of Steam Generator and Reactor Coolant Pump Supports	NUREG-0577, Rev. 1 SRP Revision 5.3.4	PWR	NC	CNS SER Dated February 1983 Concluded That The Catawba Units Are Adequately Addressing This Generic Safety Issue
A-17	Systems Interactions	Ltr: DeYoung to licensees - 9/77 NUREG-1174, NUREG-1229, NUREG/CR-3922, NUREG/CR-4261, NUREG/CR-4470, GL 89-18 (No requirements)	All	NC	No Actions Required. GL 89-18 Provides Resolution
A-24/ MPA B-60	Qualification of Class 1E Safety-Related Equipment	NUREG-0588, Rev. 1 SRP 3.11 10 CFR 50.49 GL 82-09, GL 84-24 GL 85-15	All	C-5/86	CNS SSER #6 Dated May 1986 Approved Unit 2 Differences. Unit 1 and 2 Identical Equipment Approved In SSERs # 3, 4, & 5.

DUKE POWER COMPANY  
 CATAWBA NUCLEAR STATION  
 UNITS 1 AND 2  
 DOCKET NOS. 50-413 AND 414

USI/MPA NUMBER	TITLE	REF. DOCUMENT	APPLICABILITY	STATUS/DATE*	REMARKS
A-26/ MPA B-04	Reactor Vessel Pressure Transient Protection	DOP Letters to Licensees 8/76 NUREG-0224 NUREG-0371 SRP 5.2 GL 88-11	PWR	I	Tech Spec Change Requested Per H. B. Tucker Letter Dated 4/19/89. Awaiting NRC Action, Date Unknown
A-31	Residual Heat Removal Shutdown Requirements	NUREG-0606 RG 1.113, RG 1.139 SRP 5.4.7	All OLs After 01/79.	C-11/86	Pressurizer PORVs Upgraded To Safety Grade. U1 First Refueling Outage; U2 To Initial Criticality.
A-36/ C-10, C-15	Control of Heavy Loads Near Spent Fuel	NUREG-0612 SRP 9.1.5 GL 81-07, GL 83-42, GL 85-11 Letter from DG Eisenhut dated 12/22/80	All	C-6/85	CNS SSER #4 Dated December 1984 Approved Licensee's Submittals As Satisfying NUREG-0612 Guidelines. GL 85-11 Dated 6/85 With- Drew Phase II Of NUREG-0612
A-39	Determination of SRV Pool Dynamic Loads and Pressure Transients	NUREG-0802 NUREGs-0763, 0783, 0802 NUREG-0661 SRP 6.2.1.1.C	BWR	NA	
A-40	Seismic Design Criteria	SRP Revisions, NUREG/ CR-4776, NUREG/CR-0054, NUREG/CR-3480, NUREG/ CR-1582, NUREG/CR-1161, NUREG-1233, NUREG-4776 NUREG/CR-3805 NUREG/CR-5347 NUREG/CR-3509	All	NC	CNS SER, Dated February 1983 Stated Seismic Design Acceptable
A-42/ MPA B-05	Pipe Cracks in Boiling Water Reactors	NUREG-0313, Rev. 1 NUREG-0313, Rev. 2 GL 81-03, GL 88-01	BWR	NA	---

DUKE POWER COMPANY  
 CATAWBA NUCLEAR STATION  
 UNITS 1 AND 2  
 DOCKET NOS. 50-413 AND 414

<u>USI/MPA NUMBER</u>	<u>TITLE</u>	<u>REF. DOCUMENT</u>	<u>APPLICABILITY</u>	<u>STATUS/DATE*</u>	<u>REMARKS</u>
A-43	Containment Emergency Sump Performance	NUREG-0510, NUREG-0869, Rev. 1 NUREG-0897, R.G.1.8? (Rev. 0), SRP 6.2.2 GL 85-22 No Requirements	All	NC	NRC Letter Dated 8/29/86
A-44	Station Blackout	RG 1.155 NUREG-1032 NUREG-1109 10 CFR 50.63	All	I	H. B. Tucker Letter Dated 4/17/89, Awaiting NRC Approval, Date Unknown
A-45	Shutdown Decay Heat Removal Requirements	SECY 88-260 NUREG-1289 NUREG/CR-5230 SECY 88-260 (No requirements)	All	E-12/91	H. B. Tucker Letter Dated 11/1/89, Response To G.L. 88-20 (IPE).
A-46	Seismic Qualification of Equipment in Operating Plants	NUREG-1030 NUREG-1211/ GL 87-02, GL 87-03	All	NA	Seismic Qualification Of Equipment Has Been Reviewed And Found Acceptable. CNS SER Dated February 1983.
A-47	Safety Implication of Control Systems	NUREG-1217, NUREG-1218 GL 89-19	All	E-3/90	Response To G.L. 89-19 Required By 3/19/90
A-48	Hydrogen Control Measures and Effects of Hydrogen Burns on Safety Equipment	10 CFR 50.44 SECY 89-122	All, except PWRs with large dry containments	I	H. B. Tucker Letter Dated 1/4/89. Awaiting NRC Approval Of Hydrogen Generation Analysis. Containment Response Analysis To Follow
A-49	Pressurized Thermal Shock	RGs 1.154, 1.99 SECY 82-465 SECY 83-288 SECY 81-687 10 CFR 50.61/ GL 88-11	PWR	C-9/87	NRC Letter Dated 6/28/88