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50-287 Oconee Nuclear Station, Unit 3, Duke Power Co. 05000287
AUTH.NAME AUTHOR AFFILIATION
TUCKER,H.B. Duke Power Co.
RECIP.NAME RECIPIENT AFFILIATION
DENTON,H.R. Office of Nuclear Reactor Regulation, Director
STOLZ,J.F. Operating Reactors Branch 4

SUBJECT: Forwards requests for relief from ASME Code Section XI inservice insp requirements for hydrostatic testing of valves to suppl 840913 & 1116 requests,Hydrostatic testing of valves impractical & uneconomcial.

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NOTES:AEOD/Ornstein:1cy. 05000269
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HAL B. TUCKER
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December 11, 1984

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Mr. John F. Stolz, Chief
Operating Reactors Branch No. 4

Subject: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Sir:

Pursuant to 10 CFR 50, 50.55a, please find attached two requests for relief from the requirements of Section XI of the ASME Boiler and Pressure Vessel Code (with Addenda through Winter 1980). These requests are submitted due to the impracticality of hydrostatically testing specific valves as required by the code following maintenance or modification.

By letter dated November 16, 1984, Duke Power submitted a general request for exemption from the requirements of hydrostatic testing following maintenance or modification for cases in which such testing is deemed impractical. The November 16th letter specifically identified alternate methods of Non Destructive Examination (NDE) to be performed for these cases. The general request, within context of the November 16th letter, is applicable to the two relief requests referred to in the preceeding paragraph; however, for remaining work associated with the recently concluded Unit 1, Cycle 9 refueling outage, Duke will submit any necessary relief requests on an individual basis.

In the future, for cases where hydrostatic testing following maintenance or modification is required, and Duke has been determined such testing to be impractical, Duke contends that the November 16th letter provides sufficient basis for such exemptions. Duke, therefore, will not submit any individual relief requests for such cases. Unless informed to the contrary, Duke considers this course of action is acceptable to the NRC.

This request is considered to supplement the request made by my letter of September 13, 1984, as supplemented by my November 16th letter. As such, no additional fees are required.

Very truly yours,

Hal B. Tucker

Hal B. Tucker

RFH:slb

A047
1/1

Attachment

cc: Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30323

Mr. J.C. Bryant
NRC Resident Inspector
Oconee Nuclear Station

Ms. Helen Nicolaras
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

DUKE POWER COMPANY
OCONEE NUCLEAR STATION
(WITH ADDENDA THROUGH WINTER 1980)
INSERVICE INSPECTION REQUIREMENTS

A. 1. Component for Which Exemption is Requested:

(a) Name and Number:

1SD43, 3/4" Gate Valve
Velan Model WH-254B-2TY
P. O. Drawing #122.A1

(b) Function:

Controls the flow in the Turbine Driven Emergency Feedwater Pump steam drain going to the trench.

(c) ASME Section III Code Class:

III

(d) Valve Category:

Manual Gate Valve

2. Reference Code Requirement That Has Been Determined To Be Impractical:

Inservice requirements per IWD-5000 of Section XI of ASME Code. Specifically, the requirement for hydrostatic testing following maintenance or modification.

3. Basis for Requesting Relief:

Hydrostatic testing of this valve is impractical and uneconomical; since the valve cannot be isolated, testing would require pressurization of both the auxiliary steam header and the Turbine Driven Emergency Feedwater Pump.

4. Alternate Examination:

Liquid penetrant testing will be performed on the welds, and a leak check will be performed at system temperature and pressure (495 psig and 310°F).

5. Implementation Schedule:

Valve 1SD43 was replaced 11/27/84. The liquid penetrant test was done after the installation, and leak check was performed at system temperature and pressure.

DUKE POWER COMPANY
OCONEE NUCLEAR STATION
REQUEST FOR RELIEF FROM ASME CODE SECTION XI
(WITH ADDENDA THROUGH WINTER 1980)
INSERVICE INSPECTION REQUIREMENTS

A. 1. Component for Which Exemption is Requested:

(a) Name and Number: 1 LP-46, 1½" check valve
P.O. Drawing #100A-1

(b) Function: Prevents backflow from pressurizer to auxiliary spray system.

(c) ASME Section III Code Class: I

(d) Valve Category: N/A

2. Reference Code Requirement That Has Been Determined To Be Impractical: Inservice requirements per IWB-5000 of Section XI of ASME Code. Specifically, the requirement for hydrostatic testing following maintenance or modification.

3. Basis for Requesting Relief: Hydrostatic testing is impractical and uneconomical because the entire pressurizer would have to be pressurized to perform the test.

4. Alternate Examination: Liquid penetrant test the stainless steel socket welds that replaced valve 1LP46. This was performed and accepted. Additionally, visual examination for leakage at system operating temperature and pressure will be performed.

5. Implementation Schedule: Valve 1LP46 was replaced 11/10/84, Visual inspection of welds will be performed at hot shutdown.

LAST WELD: 7-34

Ellenburg

System # 53A(9) UNIT #1 R.B. CLASS B&C CRES/304H SHEET 7

Size	WALL THICKNESS	CLASS	MAT.	WELD NUMBER	Weld Procedure	N.DT Code
1½"	.281"	B	304H	2-4, 7,8, 28,29, 8AA, 8BA, 22, 23, 30, 31,	P7/8-8	6
				10-12, 13A, 14, 15B, 16A, 16B, 16-19, 20A, 21A	L-231	6
1"	.250	C	304H	34	L-204	0
1"	.250"	C	304H	24	P-7	0
1"	.250	C	316H	32, 33	L-201	0
						6
Attachments				4A, 8A, 8B, 11A, 15A	P-7	

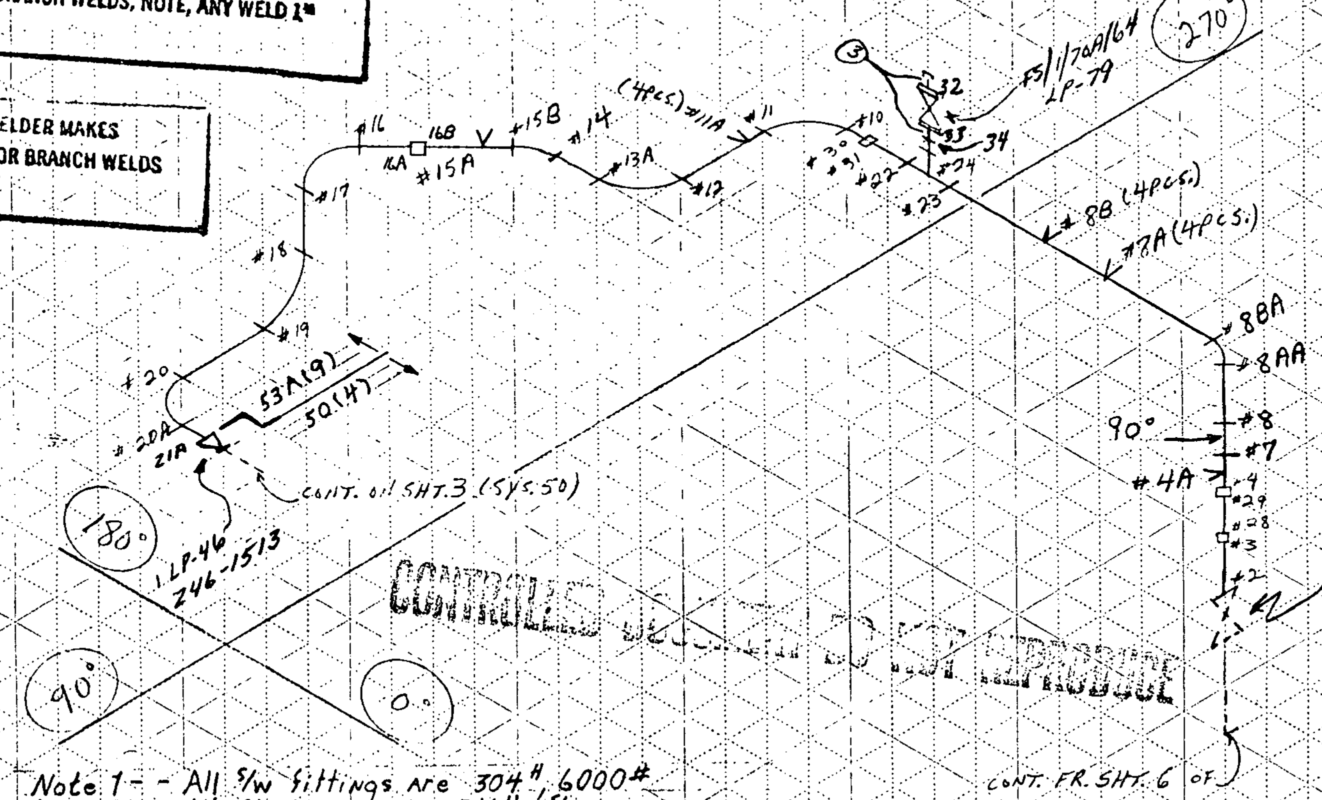
B31.7
P.S. 1501.2
SECT III - CL. 1

OMIT WELDS	
9, 7, 25	
5, 6, 14	5
15, 13	8
26, 27	10
21	11

ADDED WELDS	
[REPUNCH: 3, 4, 28, 29]	
[REPUNCH # 24 to 1"]	
30, 31	
8A, 8B, 11A, 15A,	4
4A, 8AA, 8BA	5
14	6
Relocate #4A	7
16A, 16B, 15B, 13A	8
Correct Size	9
Note 1 & 2	9
Repunch 1½" to B	9
Repunch All to 304H	

CLASS B - 100% RT ALL BUTT WELDS AND BRANCH WELDS OVER 4" NPS. 100% MT/PT ALL FILLET, SOCKET, SEAL, ATTACHMENT, AND BRANCH WELDS. NOTE, ANY WELD 1" AND LESS IN DIAMETER IS CLASS C OR E.

CLASS C - RT 1 WELD FOR EVERY 10 BUTT WELDS A WELDER MAKES OVER 4" MT/PT 1 WELD FOR EVERY 10 FILLET, SOCKET, OR BRANCH WELDS A WELDER MAKES OVER 4" DIAMETER.



ADDED WELDS	
32-34	10
ADD W.P. 204	11
21A	12

Note 1 - - All S/W fittings are 304H, 6000#
Note 2 - - All flanges are 316H, S/S
NOTE 3 - - 316H MAT'L

REF. DRWGS. 481A

WR. 96986B

REV. 12 11-7-84

✓ by K.W. McGeorge 11 Dec 72

FORM QR 27 | REVISION 1

Ellenburg

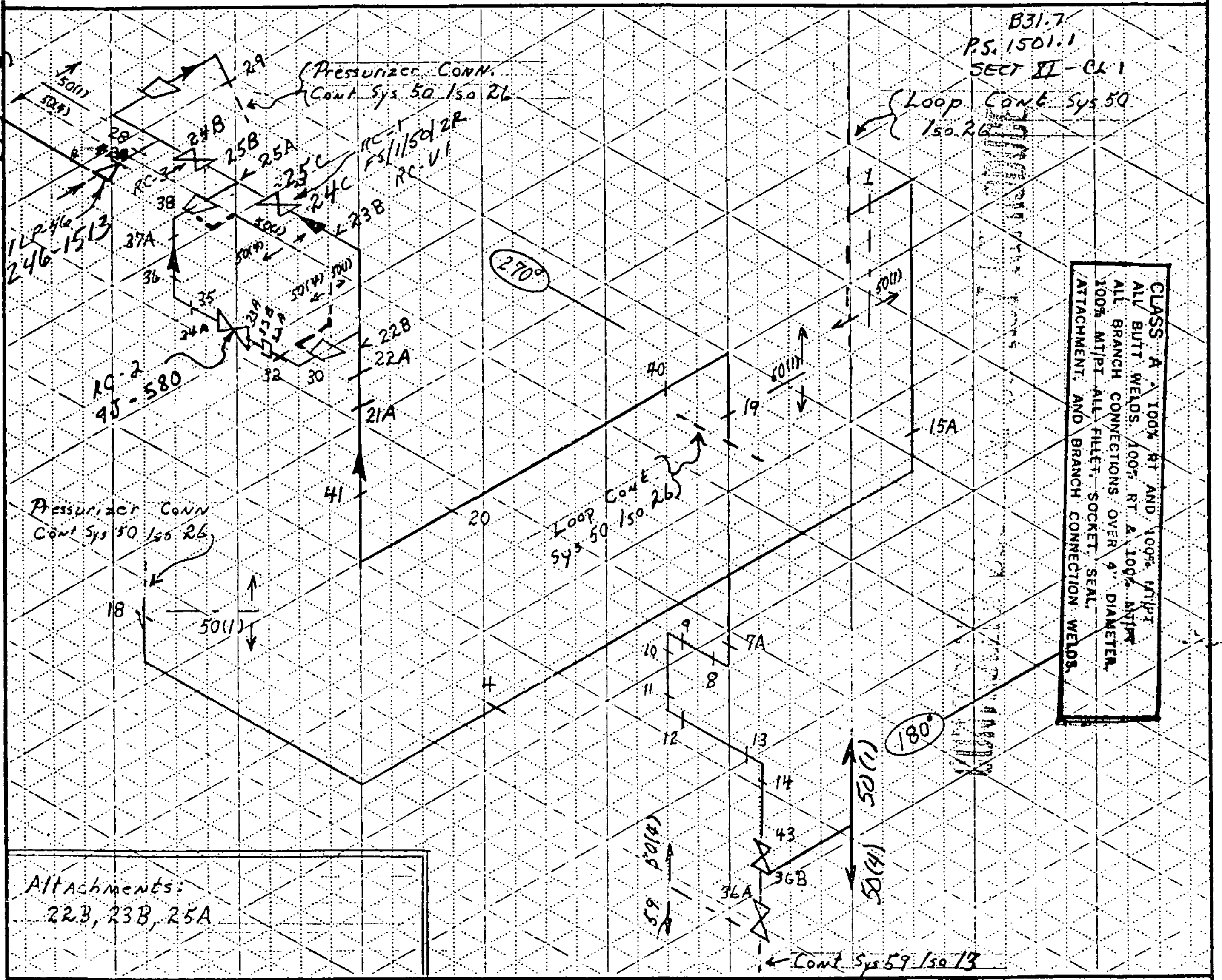
DUKE POWER COMPANY
CONSTRUCTION DEPARTMENT

ISOMETRIC SKETCH Part 1 of 3

PROJECT OCONEE SYSTEM 50 SUB SYSTEMS (1) (4) UNIT 1 ISO. NO.* 3 REV. NO. 18

L-231 L-201 L-204 welds 1129 - P-11

CLASS A + C MATERIAL CRES/316 + Inconel WELDING PROCEDURE P-7 or P-8 LAST WELD NO.* 43 DATE 11-7-84



REF. DWG. NOS.		SIZE × WALL THICKNESS	WELD NUMBERS			NDT CODE	ISO. REV. NO.	CHANGES		ISO. REV. NO.	CHANGES	
DWG.	REV.			CLASS	MAT.			±	WELD NOS.		±	WELD NOS.
0-479A	N/A	10"ø x 1.000"	1	A	INC 316	1	8		Weld Transferred	13	-	21, 22
0-479E		10"ø x 1.000"	15A, 4, 18	A	316	1	8		by W.T.R-30	13 DM	+	21A, 22A
0-478A		4"ø x .531"	29	A	INC 316	1	8	+	7B	14		RE-ADD 24, 25
0-478B		2½"ø x .375"	19-20, 24C, 25C,	A	316	1	8		Repunch 14, 15A, 13	15	-	24, 25
0-481A		21A, 22A	40, 41, 24B, 25B				9		Draftsman error		+	24C, 25C
PD-100A-1		1½"ø x .281"	28	A	316	1	10	-	7B	16		L-201
		1"ø x .250"	7A, 8-14, 43, 36A, 36B	A	316	1	11	-	24, 25	17	+	32A, 32B, 33A, 34A
		1½" x .281"	42A	A	316	2	11	+	24A, 25B		-	34, 33
WR 96986B		½"ø x .199"	30, 32, 35, 36, 37A, 38	A	316	1	12	-	26, 27, 24A	14	-	42
NSM-0765		½"ø x .188"	32A, 32B, 33A, 34A	C	316	01	12		24B		+	42A

*ALL WELD NUMBERS SHOWN ABOVE ARE PRECEDED BY THE ISO. NO.

RLM

NSM-992

✓ K.W. [illegible] 11 Dec 7?