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 FACIL:50-269 Oconee Nuclear Station, Unit 1, Duke Power Co.  
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 RECIP.NAME RECIPIENT AFFILIATION  
 DENTON,H.R. Office of Nuclear Reactor Regulation, Director  
 STOLZ,J.F. Operating Reactors Branch 4

DOCKET #  
05000269

SUBJECT: Submits supplemental info in response to 840911 ltr re Tech Spec amend request to support facility operation at full-rated power during Cycle 9.Rev makes Tech Specs consistent w/transition cores containg Mark B & BZ fuel assemblies.

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November 1, 1984

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Attention: Mr. John F. Stolz, Chief  
Operating Reactors Branch No. 4

Subject: Oconee Nuclear Station, Unit 1  
Docket No. 50-269

Dear Sir:

This submittal provides supplemental information to a Duke Power letter of September 11, 1984 which transmitted the technical specification amendment request to support the operation of Oconee Unit 1 at full rated power during Cycle 9. Specifically, this submittal includes a minor revision to the text comprising "Bases-Unit 1" of Technical Specification 2.1. This revision makes the Technical Specifications consistent with transition cores containing both Mark B and Mark BZ type fuel assemblies.

Attached is page 2.1-3 of the Oconee Technical Specifications which contains the revision. This revised page (2.1-3) replaces its corresponding page provided in the September 11, 1984 submittal of the proposed Technical Specification changes for Oconee 1, Cycle 9.

As discussed in the September 11th submittal, the BAW-2 and the BWC CHF correlations are applicable to the Mark B and the Mark BZ type fuel assemblies, respectively. The present wording is applicable only to a full Mark B core using the BAW-2 correlation. Therefore, the change in wording reflects the use of both the BAW-2 (for Mark B) and the BWC (for Mark BZ) correlations in the transition Cycle 9 core, and thus merely provides the required generality for the description.

The omission of the change in the September 11, 1984 submittal constitutes an administrative oversight. This revision possesses no significance with respect to the safety analyses included in the original submittal. Since this submittal consists of a supplement to a previously submitted amendment request, as yet unapproved, Duke considers additional license fees to be unjustified.

Very truly yours,



Hal B. Tucker

RFH:slb

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Mr. Harold R. Denton, Director  
November 1, 1984  
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Attachment

cc: Mr. James P. O'Reilly, Regional Administrator  
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Ms. Helen Nicolaras  
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Mr. J. C. Bryant  
NRC Resident Inspector  
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Mr. Hayward Shealey, Chief  
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For each curve of Figure 2.1-3A a pressure-temperature point above and to the left of the curve would result in a DNBR greater than the CHF correlation limit or a local quality at the point of minimum DNBR less than the CHF correlation quality limit for that particular reactor coolant pump situation. The curve of Figure 2.1-1A is the most restrictive of all possible reactor coolant pump-maximum thermal power combinations shown in Figure 2.1-3A.

#### References

- (1) Correlation of Critical Heat Flux in a Bundle Cooled by Pressurized Water, BAW-10000, March, 1970.
- (2) Oconee 1, Cycle 4 - Reload Report - BAW-1447, March, 1977.
- (3) Correlation of 15x15 Geometry Zircaloy Grid Rod Bundle CHF Data with the BWC Correlation, BAW-10143P, Part 2, Babcock & Wilcox, Lynchburg, Virginia, August 1981.
- (4) Fuel Rod Bowing in Babcock & Wilcox Fuel Designs, BAW-10147P-A, Rev. 1, Babcock & Wilcox, May 1983.