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ACCESSION NBR: 9204070113 DOC. DATE: 92/03/30 NOTARIZED: NO DOCKET #
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50-287 Oconee Nuclear Station, Unit 3, Duke Power Co. 05000287

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SUBJECT: Forwards Relief Request 92-06 from requirements of Section XI of ASME Boiler & Pressure Vessel Code in support of VT insp to be performed on body to bonnet joint of Valve 2CA-19 during next outage. W/one oversize diagram.

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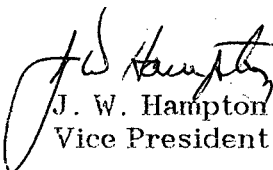
March 30, 1992

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Subject: Oconee Nuclear Station, Units 1, 2, and 3
Docket Nos. 50-269, -270, -287
Request for Relief No. 92-06

Pursuant to 10 CFR 50.55a, please find attached request for relief number 92-06 from the requirements of Section XI of the ASME Boiler and Pressure Vessel Code (with Addenda through Winter 1980). This request is being submitted due to the failure to properly perform a VT2 inspection during the last Unit 2 outage as explained in the attached request and the impracticality of inservice inspection under present plant conditions.

Very truly yours,


J. W. Hampton
Vice President

OCK/ock

Attachments

xc: (W/Attachments)

Mr. S. D. Ebnetter, Regional Administrator
US Nuclear Regulatory Commission, Region II

Mr. L. A. Wiens, Project Manager
Office of Nuclear Regulation

Mr. P. E. Harmon
NRC Senior Resident Inspector
Oconee Nuclear Station

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OCONEE NUCLEAR STATION

Second Ten Year Interval

Request # 92-06

1. Component for which relief is requested:

(a) Name and Number: 2CA - 19

(b) Function: Alternate flow path for Boric Acid addition to Letdown.

(c) ISI Class/Duke Class: Inlet NA G
 Outlet C C

(d) IWV-2000 Valve Category: NA

(e) Drawing: OFD-101A-2.2
 OM-246-008 Sh 2

2. Reference Code Requirement that has been determined to be impractical: IWA-5214(e) One body to bonnet bolt was replaced due to corrosion. This requires that a VT-2 inspection be performed on the body to bonnet joint.

3. Basis for requesting relief: This valve is in the Letdown Storage Tank room. Access to this room is restricted during plant operation due to high radiation levels around the tank. This is a 1 1/2" Velan gate valve, design pressure and temperature are 150# and 200 F. The valve is normally shut and serves as an alternate flow path from the Boric Acid Mix pump to the Letdown Filters. There is no flow meter in this line so it is not used except in an emergency.

During the past Unit 2 refueling outage (RFO) the discharge side of 2CA-19 was hydrostatic tested. 2CA-19 was used as a boundary valve. The decision to use 2CA-19 as a boundary valve was based on an initial review that the seat would allow enough water to pass into the bonnet area and pressurize the bonnet studs.

A later review (March 18, 1992) and more through review of the valve drawing (OM 246-008 sheet 1 & 2) could not support this conclusion. Based on this review it was decided that a proper VT-2 inspection had not been performed.

4. Alternate Examination: During the next outage of sufficient length for the radiation levels to decay off or the next refueling outage on Unit 2 this valve will be inspected to determine if it has leaked during this operating cycle.
5. Acceptability of proposed alternate testing with respect to the level of quality and safety as well as public health and safety: This valve is entirely contained within the Auxiliary Building. The Auxiliary Building is designed to contain any leakage, and it has the ability to monitor any gas release to ensure the health and safety of the general public is not endangered.

The correct material was used to make up this joint. The stud material was SA-193 B7, alloy steel, 1/2" X 2 1/2"-13 tpi UNC-2A, Duke Class B, ASME Section III subsection NC, QA tag # ON 063264, heat # LIB. The nut material was SA-194 Grade 7; 1/2"-13 UNC-2B heavy hex, Duke Class B, ASME Section III subsection NC, QA tag # ON 057531, heat # 6030315. The joint was torqued to 57 ft-lb in accordance with procedure MP/O/A/1200/002. The torquing was witness by W. L. Trout, of QA, on 2/12/92.

6. Implementation Schedule: During the next outage of sufficient length or during the next refueling outage in June 1993.

Requested By: Basil W. Carney Date: 3/25/92
Reviewed By: Ted K. Royal Date: 3/25/92
QA Reviewed: T. J. Coleman Date: 3-26-92
Approved By: Don E. Howard Date: 3-26-92