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SUBJECT: Discusses 900319 ltr from NRC to BW re results of recent analysis of LOCA limits for 177 fuel assembly LL plants.

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July 18, 1990

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Subject: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287
10 CFR 50.46(a)(3)

Reference: J. H. Taylor (B&W) to Dr. T. E. Murley (NRC)
Letter Dated March 19, 1990 (JHT/90-43)

Dear Sir:

By the above referenced letter, the NRC was advised by Babcock and Wilcox (B&W) of The Results of a Recent Analysis of the Loss of Coolant Accident (LOCA) Limits for the 177 Fuel Assembly (FA) Lowered Loop (LL) Plants. The analysis performed by B&W determined a revised Peak Cladding Temperature (PCT) of 2194°F for the 6 foot core elevation, which is greater than 50°F from the previously calculated PCT. As required by 10 CFR 50.46, the NRC was notified of this "Significant" change in the application of the LOCA Evaluation Model (EM) by the above referenced letter (attached).

The purpose of this letter is to document the above referenced letter on the Dockets for Oconee Units 1, 2, and 3. Accordingly, the above referenced letter is provided. As specified in Mr. Taylor's March 19, 1990 letter, the results of the 4 foot evaluation will be provided to the NRC when they become available.

Very truly yours,

Hal B. Tucker

PFG/110/lcs

cc: W/Attachment
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