

ENCLOSURE 1

NOTICE OF VIOLATION

Duke Power Company
Oconee Units 1, 2 and 3

Docket Nos.: 50-269, 50-270, and 50-287
License Nos.: DPR-38, DPR-47, DPR-55

During an NRC inspection conducted on January 25 - March 5, 1993, a violation of NRC requirements was identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C, the violation is listed below:

10 CFR 50, Appendix B, Criterion II, "Quality Assurance Program," states in part "The applicant shall identify the structures, systems and components to be covered by the quality assurance program."

Duke Power Company Topical Report 1-A states through reference to ANSI 45.2.11 that all structures, systems and components necessary to mitigate the consequences of an accident are applicable to the quality assurance program, designated QA-1, and a controlled listing of those components are issued under the approval of the site Vice President.

The Administrative Policy Manual for Nuclear Stations, section 1.2.2.39, Safety Related Structures, Systems and Components, states "Safety-Related, i.e., QA Condition 1, structures, systems and components are those structures, systems and components at a station determined to be safety-related in accordance with the criteria and other information given in the applicable station "Quality Standards Manual."

Quality Standards Manual, section 4.1, states Table 1 provides QA Condition 1 items along with appropriate design drawings. Table 1 classifies 6.9 kV auxiliary AC power as not safety related and safety related is assigned to only safety related loads on the 4.16 kV TC, TD, and TE busses.

Contrary to the above, as of March 5, 1993, the licensee's identification of components covered by the quality assurance program was inadequate. The Quality Standards Manual did not reflect that the feeder circuit breakers from the startup transformers to the 6.9 kV TA and TB {Reactor Coolant pump (RCP)} busses or the 4.16 kV TC, TD, and TE circuit breakers feeding non-safety related loads were QA Condition 1 on all three units. These components are necessary to mitigate the consequence of an accident. Consequently, the licensee's quality assurance program was not applied to maintenance activities associated with circuit breaker 2TD8 on June 14, 1989, circuit breaker 2TC5 on June 22, 1989, and circuit breaker 2TA2 on February 10, 1988, when the work orders were designated not nuclear safety related.

This is a Severity Level IV violation (Supplement 1).

Pursuant to the provisions of 10 CFR 2.201, Duke Power is hereby required to submit a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D.C. 20555 with a copy

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to the Regional Administrator, Region II, and a copy to the NRC Resident Inspector at the Oconee facility that is the subject of this Notice, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. If an adequate reply is not received within the time specified in this Notice, an order or Demand for Information may be issued as to why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Dated at Atlanta, Georgia
this 07th day of May 1993