ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR:9011010115 DOC.DATE: 90/10/23 NOTARIZED: NO DOCKET #
FACIL:50-270 Oconee Nuclear Station, Unit 2, Duke Power Co. 05000270
AUTH.NAME AUTHOR AFFILIATION
TUCKER,H.B. Duke Power Co.
RECIP.NAME RECIPIENT AFFILIATION
Document Control Branch (Document Control Desk)

SUBJECT: Request for relief 90-05 from requirements of section XI of ASME boiler & pressure vessel code.

DISTRIBUTION CODE: A047D COPIES RECEIVED:LTR 1 ENCL 1 SIZE: 4
TITLE: OR Submittal: Inservice Inspection/Testing/Relief from ASME Code

NOTES:

	RECIPIENT ID CODE/NAME	COPIES LTTR	COPIES ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR	COPIES ENCL
	PD2-3 LA	1	0	PD2-3 PD	1	1
	WIENS,L	2	2			
INTERNAL:	ACRS	6	6	AEOD/DSP/TPAB	1	1
	NRR/DET/ECMB 9H	1	1	NRR/DET/EMEB 7E	1	1
	NUDOCS-ABSTRACT	1	1	OC/LFMB	1	0
	OGC/HDS2	1	0	REG FILE 01	1	1
	RES/DSIR/EIB	1	1			
EXTERNAL:	EG&G BROWN,B	1	1	EG&G RANSOME,C	1	1
	NRC PDR	1	1	NSIC	1	1

R I D S / A D D S

R I D S / A D D S

NOTE TO ALL "RIDS" RECIPIENTS:

PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK, ROOM P1-37 (EXT. 20079) TO ELIMINATE YOUR NAME FROM DISTRIBUTION LISTS FOR DOCUMENTS YOU DON'T NEED!

TOTAL NUMBER OF COPIES REQUIRED: LTTR 22 ENCL 19

Duke Power Company
P.O. Box 33198
Charlotte, N.C. 28242

HAL B. Tucker
Vice President
Nuclear Production
(704)373-4531

DUKE POWER

October 23, 1990

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Subject: Oconee Nuclear Station
Docket No. 50-270
Second Ten Year Interval
Request for Relief No. 90-05

Gentlemen:

Pursuant to 10CFR 50, 50.55a, please find attached request for relief number 90-05 from the requirements of Section XI of the ASME Boiler and Pressure Vessel Code (with Addenda through Winter 1980). This request is being submitted due to the impracticality of pressure testing specific welds as required by the Code following repair. The attached request concerns the inservice inspection at Oconee Unit 2 being performed during the second ten year interval.

Very truly yours,

Hal B. Tucker

LBJ/3/lbj

Attachment

cc: W/Diagram
Mr. L. A. Wiens
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
One White Flint North, Mail Stop 9H3
Washington, D.C. 20555

W/O Diagram
Mr. S. D. Ebneter
Regional Administrator, Region II
U. S. Nuclear Regulatory Commission
101 Marietta St., NW., Suite 2900
Atlanta, Georgia 30323

Mr. Heyward Shealy, Chief
Bureau of Radiological Health
S. C. Department of Health and
Environmental Control
2600 Bull Street
Columbia, S. C. 29201

Mr. P. H. Skinner
NRC Resident Inspector
Oconee Nuclear Station

Duke Power Company
Oconee Nuclear Station
Second Ten Year Interval
Request for Relief No. 90-05

I. Component for Which Relief is Requested:

(a) Name and Number:

Unit 2 Steam Drain (SD) System welds for installing valves 2SD-418, 2SD-419, 2SD-420, 2SD-421.

(b) Function:

Drains water which collects at the main steam stop valves.

(c) ISI Class/Duke Class:

ISI Class B/Duke Class F.

(d) IWV-2200 Valve Category:

N/A

(e) Design Pressure: 1050 psig
Design Temp: 630°F
Material: Schedule 80 - 1-inch carbon steel on 2SD-418, 2-inch carbon steel 2SD-419, 420, and 421

II. Reference Code Requirement that has been Determined to be Impractical:

ASME Boiler and Pressure Vessel Code Section XI, 1980 Edition (with Addenda through Winter 1980) paragraph IWA-4400(a), which states that after repairs by welding on the pressure retaining boundary, a system hydrostatic test shall be performed in accordance with IWA-5000.

III. Basis for Requesting Relief:

In order to perform a hydrostatic pressure test on the welds associated with Unit 2 Steam Drain System valves 2SD-418, -419, -420, -421, the piping incorporating these valves must be filled with water and pressurized. These valves are located in an area of the system that is connected to the condenser. Since these valves cannot be isolated from the condenser, a hydrostatic test would also be performed on the condenser. A hydrostatic test would overpressurize the condenser which could cause equipment damage and risk the safety of plant personnel.

IV. Alternate Examination:

A liquid penetrant examination will be performed and a VT-2 inspection will be performed at operating temperature and pressure.

V. Evaluation of Acceptability of Proposed Alternate Testing With Respect to the Level of Quality and Safety as Well as Public Health and Safety:

The specified method of hydrostatic testing verifies that there are no leaks at 1.25 times the design pressure. The alternate examination of the penetrant test assures that no significant flaws are evident in the welds. The VT-2 inspection indicates that no leaks are detectable when the system is at operating temperature and pressure. The alternate tests provide an equivalent method to indicate a leak at the higher stress level which is normally verified by the specified method of hydrostatic testing. As such, the proposed alternate examinations provide an acceptable level of quality and safety and will not endanger the health and safety of the public.

VI. Implementation Schedule:

Alternate examinations will be performed when welding is complete and during startup from the Unit 2 end of cycle 11 currently scheduled for October 26, 1990. Hydrostatic testing will be performed during the second ten year interval inservice inspection hydro of the main steam lines.