

50-269, 270, 287

50-369, 370, 413

50-414



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**

WASHINGTON, D.C. 20555-0001

October 15, 1997

Mr. M. S. Tuckman
Senior Vice President
Nuclear Generation
Duke Energy Corporation
P. O. Box 1006
Charlotte, NC 28201

**SUBJECT: EVALUATION OF 10-YEAR INTERVAL INSPECTION PROGRAM PLAN
ALTERNATIVES FOR OCONEE NUCLEAR STATION, UNITS 1, 2, AND 3;
MCGUIRE NUCLEAR STATION, UNITS 1 AND 2; AND CATAWBA NUCLEAR
STATION, UNITS 1 AND 2 (TAC NOS. M99075, M99076, M99087, M99088,
M99101, M99102, M99103)**

Dear Mr. Tuckman:

By letter dated June 20, 1997, Duke Energy Corporation (DEC) requested relief from the 1989 Edition of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (ASME Code), Section XI, paragraph IWA-5250(a)(2) requirements and approval of proposed alternatives to the ASME Code for the Oconee Nuclear Station, Units 1, 2, and 3, McGuire Nuclear Station, Units 1 and 2, and Catawba Nuclear Station, Units 1 and 2. The relief and alternatives are related to actions to be taken in the event leakage is detected at a bolted connection. As a result of an August 6, 1997, conference call, you submitted a revision to the proposed alternative in a letter dated August 7, 1997.

The staff, with technical assistance from its contractor, the Idaho National Engineering and Environmental Laboratory (INEEL), has reviewed the material provided and concluded that the proposed alternative to the requirements of the Code is a conservative and technically sound engineering approach and provides an acceptable level of quality and safety for leaking bolted connections. Therefore, your proposed alternative is authorized pursuant to 10 CFR 50.55a(a)(3)(i).

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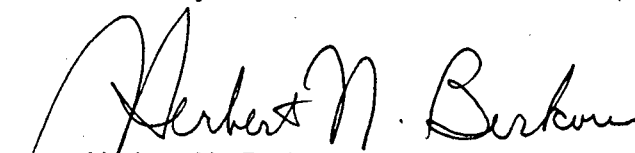


M. S. Tuckman

- 2 -

The staff's evaluation and conclusions are contained in Enclosure 1. Enclosure 2 is the INEEL Technical Letter Report.

Sincerely,



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, ~~50-270~~, ~~50-287~~,
50-369, ~~50-370~~, ~~50-413~~,
and ~~50-414~~

Enclosures:

1. NRR Safety Evaluation
2. INEEL Technical Letter Report

cc w/encl: See next page

The staff's evaluation and conclusions are contained in Enclosure 1. Enclosure 2 is the INEEL Technical Letter Report.

Sincerely,

ORIGINAL SIGNED BY:

Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270, 50-287,
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cc w/encl: See next page

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Catawba Nuclear Station
McGuire Nuclear Station
Oconee Nuclear Station

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