



**NRC NEWS**  
**U. S. NUCLEAR REGULATORY COMMISSION**

Office of Public Affairs  
Washington, DC 20555-001

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No. 96-49

FOR IMMEDIATE RELEASE  
(Friday, March 8, 1996)

NOTE TO EDITORS:

The Nuclear Regulatory Commission staff has issued two letters to Northeast Utilities (NU) requesting certain information, under oath or affirmation, regarding operation of the Millstone 2 & 3 and Haddam Neck nuclear power plants in Connecticut.

Millstone 2 is required to submit to the NRC, seven days before restarting from its current refueling outage, actions it has taken to assure future operation will be conducted in compliance with its operating license, NRC regulations and final safety analysis report for the plant.

Millstone 3 and Haddam Neck, both of which are currently in operation, are required to provide the NRC similar information and assurance within 30 days of the date of receipt of the letters.

The NRC is taking this action after reviewing an internal NU document that found inaccuracies in Millstone 1's final safety analysis report. Millstone 1 is currently shut down.

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Attachments:

As stated

March 7, 1996

Mr. Robert E. Busch  
President - Energy Resources Group  
Northeast Utilities Service Company  
P.O. Box 128  
Waterford, CT 06385

Dear Mr. Busch:

On December 13, 1995, the NRC issued to Northeast Utilities (NU) a letter requesting NU pursuant to Section 182a of the Atomic Energy Act of 1954, as amended, and 10 CFR 50.54(f) to "describe actions taken to ensure that future operation of Millstone Unit 1 will be conducted in accordance with the terms and conditions of the Millstone Unit 1 operating license, the Commission's regulations, including 10 CFR 50.59, and the Millstone Unit 1 Updated Final Safety Analysis Report (UFSAR)." Since that time NRC has continued to conduct inspections and investigations at the Millstone

Station to determine the state of compliance of NU nuclear facilities with NRC requirements. As part of these efforts, NRC has obtained a copy of an internal NU document, "ACR 7007 - Event Response Team Report" (7007 Report), dated February 22, 1996.

The Executive Summary of the 7007 Report states that an Event Response Team was chartered to determine the causes for the inaccuracies in the Millstone Unit 1 UFSAR. The fundamental causes for these inaccuracies were found by this Team to include:

- The original 1986/1987 UFSAR contained errors and omissions;
- Administrative control programs such as Design Control, Corrective Action, and Commitment Tracking did not fully address regulatory requirements;
- NU did not fully implement the administrative programs. NU did not see the UFSAR as a document that was required to be accurate;
- Internal correspondence and events involving the design basis from 1985 through 1996 show a pattern of information communicated to NU management. This information consistently identified weaknesses and risks associated with the UFSAR and design bases. NU management made commitments, on the docket, to correct these deficiencies. The commitments to correct these deficiencies were ineffective, partially implemented, or not done;
- NU oversight did not identify this event pattern to management, its significance, or the effectiveness of corrective actions to prevent recurrence.

The 7007 Report further states that, due to the nature of the causes that the Team has identified, the potential exists for the presence of similar configuration management conditions at Millstone Unit 2. It notes that without a sample similar to the initiatives currently in progress for Millstone Unit 1 as a result of the 10 CFR 50.54(f) letter of December 13, 1995, the full implications for Millstone Unit 2 cannot be ascertained. The 7007 Report recognizes that those efforts may be underway. This Report also addresses Millstone Unit 3 and Haddam Neck which we are addressing by separate letter.

Current licensee reviews and NRC inspections of Millstone Unit 2 have identified a number of operability and design concerns. Millstone Unit 2 shutdown on February 20, 1996, when a potential design deficiency was identified that could block or reduce safety injection flow during the recirculation phase of an accident. During this shutdown, other design discrepancies were identified in which NU had not maintained the current design or licensing basis for Millstone Unit 2. For example, NU's inspection of the containment sump screen mesh revealed that debris larger than the design value could pass through with potential adverse consequences to the emergency core cooling systems. NU identified that the flood protection enclosure could not be installed on one of the service water pumps that has been relied on for ultimate heat sink operability. Further, the NRC identified that the post-accident containment hydrogen monitor design was flawed in that insufficient sample flow

would be available at low containment pressures when the monitor must be operable.

Consequently, there is a question as to whether Millstone Unit 2 conforms to the UFSAR, license conditions, and Commission regulations. Therefore, the NRC requires additional information to be submitted pursuant to Section 182a of the Atomic Energy Act of 1954, as amended, and 10 CFR 50.54(f) in writing, under oath or affirmation, to determine whether or not the license for Millstone Unit 2 should be suspended, modified, or revoked. The information is to be submitted no later than 7 days prior to Millstone Unit 2 restart (prior to criticality) from its current outage and is to describe actions taken to ensure that future operation of Millstone Unit 2 will be conducted in accordance with the terms and conditions of the Millstone Unit 2 operating license, the Commission's regulations, including 10 CFR 50.59, and the Millstone Unit 2 UFSAR.

The submittal should describe actions taken to assure that deficiencies identified at Millstone Unit 2 based on your ongoing review have been evaluated for operability, existence of unreviewed safety questions, and reportability. In particular, seriously degraded conditions must be reported pursuant to 10 CFR 50.72(b)(2)(i) and 50.73(a)(2)(ii).

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and your responses will be placed in the NRC Public Document Room (PDR) the Gelman Building, 2120 L Street, NW., Washington, DC, and in the local public document room located at the Learning Resources Center, Three Rivers Community-Technical College, 574 New London Turnpike, Norwich, CT 06360. The NRC also intends to place in the PDR a copy of the 7007 Report on March 15, 1996, unless you provide a sufficient basis to withhold this Report by March 12, 1996. Any request for withholding must be accompanied by a bracketed copy of the Report that identifies the information that you seek

to have protected and a redacted copy that deletes such information. You must provide for each portion of the document you seek to be withheld the bases for your claim of withholding.

Sincerely,

/s/

William T. Russell, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission

Docket No. 50-336

cc: See next page

March 7, 1996

Mr. Robert E. Busch  
President - Energy Resources Group  
Northeast Utilities Service Company  
P.O. Box 128

Dear Mr. Busch:

On December 13, 1995, the NRC issued to Northeast Utilities (NU) a letter requesting NU pursuant to Section 182a of the Atomic Energy Act of 1954, as amended, and 10 CFR 50.54(f) to "describe actions taken to ensure that future operation of Millstone Unit 1 will be conducted in accordance with the terms and conditions of the Millstone Unit 1 operating license, the Commission's regulations, including 10 CFR 50.59, and the Millstone Unit 1 Updated Final Safety Analysis Report (UFSAR)." Since that time NRC has continued to conduct inspections and investigations at the Millstone Station to determine the state of compliance of NU nuclear facilities with NRC requirements. As part of these efforts, NRC has obtained a copy of an internal NU document, "ACR 7007 - Event Response Team Report" (7007 Report), dated February 22, 1996.

The Executive Summary of the 7007 Report states that an Event Response Team was chartered to determine the causes for the inaccuracies in the Millstone Unit 1 UFSAR. The fundamental causes for these inaccuracies were found by this Team to include:

- The original 1986/1987 UFSAR contained errors and omissions;
- Administrative control programs such as Design Control, Corrective Action, and Commitment Tracking did not fully address regulatory requirements;
- NU did not fully implement the administrative programs. NU did not see the UFSAR as a document that was required to be accurate;
- Internal correspondence and events involving the design basis from 1985 through 1996 show a pattern of information communicated to NU management. This information consistently identified weaknesses and risks associated with the UFSAR and design bases. NU management made commitments, on the docket, to correct these deficiencies. The commitments to correct these deficiencies were ineffective, partially implemented, or not done;
- NU oversight did not identify this event pattern to management, its significance, or the effectiveness of corrective actions to prevent recurrence.

The 7007 Report further states that due to the nature of the causes that the Team has identified, the potential exists for the presence of similar configuration management conditions at the Haddam Neck Plant and Millstone Unit 3. It notes that without a sample similar to the initiatives (as a result of the 10 CFR 50.54(f) letter of December 13, 1995) currently in progress for Unit 1, the full implications for the Haddam Neck Plant and Millstone Unit 3 cannot be ascertained. The 7007 Report recognizes that those efforts may be underway. This Report also addresses Millstone Unit 2 which we are addressing by separate letter.

NRC does not have an inspection history at Millstone Unit 3

and Haddam Neck that reveals design deficiencies similar in number and nature to that of Millstone Units 1 and 2. Nevertheless in view of the 7007 Report, the NRC requires additional information to be submitted in writing, under oath or affirmation, pursuant to Section 182a of the Atomic Energy Act of 1954, as amended, and 10 CFR 50.54(f) to determine whether or not the licenses of the Millstone Unit 3 and Haddam Neck Plant should be suspended, modified, or revoked. The information required is as follows:

Millstone Unit 3

Within 30 days from the date of this letter, provide your actions taken to date and future plans to address the conclusions of the 7007 Report as it pertains to Millstone Unit 3. This should include your plans and schedule to ensure that future operation of Millstone Unit 3 will be conducted in accordance with the terms and conditions of the Millstone Unit 3 operating license, the Commission's regulations, including 10 CFR 50.59, and the Millstone Unit 3 UFSAR.

Haddam Neck Plant

Within 30 days from the date of this letter, provide your actions taken to date and future plans to address the conclusions of the 7007 Report as it pertains to Haddam Neck. This should include your plans and schedule to ensure that future operation of Haddam Neck will be conducted in accordance with the terms and conditions of the Haddam Neck operating license, the Commission's regulations, including 10 CFR 50.59, and the Haddam Neck UFSAR.

Each submittal should describe actions taken to assure that deficiencies identified at the respective facilities based on your ongoing reviews have been evaluated for operability, existence of unreviewed safety questions, and reportability pursuant to 10 CFR 50.72(b)(2)(i) and 50.73(a)(2)(ii).

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and your responses will be placed in the NRC Public Document Room (PDR) the Gelman Building, 2120 L Street, NW., Washington, DC, and in the local public document room located at the Learning Resources Center, Three Rivers Community-Technical College, 574 New London Turnpike, Norwich, CT 06360. The NRC also intends to place in the PDR a copy of the 7007 Report by March 15, 1996, unless you provide a sufficient basis to withhold this Report by March 12, 1996. Any request for withholding must be accompanied by a bracketed copy of the Report that identifies the information that you seek to have protected and a redacted copy that deletes such information. You must provide for each portion of the document you seek to be withheld the bases for your claim of withholding.

Sincerely,

/s/

William T. Russell, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Docket Nos. 50-213 and 50-423

cc: See next page