

## 6.9 CORE OPERATING LIMITS REPORT

### Specification

6.9.1 Core operating limits shall be established prior to each reload cycle, or prior to any remaining part of a reload cycle, for the following:

- (1) Axial Power Imbalance Protective Limits and Variable Low RCS Pressure Protective Limits for Specification 2.1.
- (2) Reactor Protective System Trip Setting Limits for the Flux/Flow/Imbalance and Variable Low Reactor Coolant System Pressure trip functions in Specification 2.3.
- (3) Power Dependent Rod Insertion Limits for Specifications 3.1.3.5, 3.1.11, 3.5.2.1.b, 3.5.2.2.d.2.c, 3.5.2.3, and 3.5.2.5.c.
- (4) Concentrated Boric Acid Storage Tank volume and boron concentration for Specification 3.2.2.
- (5) Core Flood Tank boron concentration for Specification 3.3.3.
- (6) Borated Water Storage Tank boron concentration for Specification 3.3.4.
- (7) Spent Fuel Pool boron concentration for Specification 3.8.15.
- (8) Quadrant Power Tilt Limits for Specification 3.5.2.4.a, 3.5.2.4.b, 3.5.2.4.d, 3.5.2.4.e, and 3.5.2.4.f.
- (9) Power Imbalance Limits for Specification 3.5.2.6

and shall be documented in the CORE OPERATING LIMITS REPORTS.

6.9.2 The approved methods used to determine the core operating limits given in Technical Specification 6.9.1 are specified in the CORE OPERATING LIMITS REPORT. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically:

- (1) DPC-NE-1002A, Reload Design Methodology II, October 1985.
- (2) NFS-1001A, Reload Design Methodology, April 1984.
- (3) DPC-NE-2003A, Oconee Nuclear Station Core Thermal Hydraulic Methodology Using VIPRE-01, July 1989.
- (4) DPC-NE-1004A, Nuclear Design Methodology Using CASMO-3/SIMULATE-3P, November 1992.
- (5) BAW-10162P-A, TACO3 Fuel Pin Thermal Analysis Computer Code, B&W Fuel Company, November, 1989.
- (6) BAW-10183P, Fuel Rod Gas Pressure Criterion, B&W Fuel Company, as approved by SER dated February, 1994.

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8. DPC-NE-3002A, "FSAR Chapter 15 System Transient Analysis Methodology," November 1991.  
(Methodology used in the system thermal-hydraulic analyses which determine the core operating limits)
9. DPC-NE-3000P-A, "Thermal-Hydraulic Transient Analysis Methodology," August 1994.  
(Modeling used in the system thermal-hydraulic analyses)
10. DPC-NE-1004A, "Nuclear Design Methodology Using CASMO-3/SIMULATE-3P," November 1992.  
(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient.)
11. DPC-NE-2004P-A, "Duke Power Company McGuire and Catawba Nuclear Stations Core Thermal-Hydraulic Methodology using VIPRE-01," December 1991 (DPC Proprietary).  
(Methodology for Specifications 2.2.1 - Reactor Trip System Instrumentation Setpoints, 3.2.1 - Axial Flux Difference (AFD), and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor  $F_{\Delta H}(X,Y)$ .)
12. DPC-NE-2001P-A, Rev. 1, "Fuel Mechanical Reload Analysis Methodology for Mark-BW fuel," October 1990 (DPC Proprietary).  
(Methodology for Specification 2.2.1 - Reactor Trip System Instrumentation Setpoints.)
13. DPC-NE-2005P-A, "Thermal Hydraulic Statistical Core Design Methodology," February 1995 (DPC Proprietary).  
(Methodology for Specification 2.2.1 - Reactor Trip System Instrumentation Setpoints, Specification 3.2.1 - Axial Flux Difference, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).
14. BAW-10162P-A, TAC03 Fuel Pin Thermal Analysis Computer Code, B&W Fuel Company, November 1989.  
(Methodology used for Specification 2.2.1 - Reactor Trip System Instrumentation setpoints).
15. BAW-10183P, Fuel Rod Gas Pressure Criterion, B&W Fuel Company, as approved by SER dated February, 1994.  
(Used for Specification 2.2.1, Reactor Trip System Instrumentation Setpoints).