

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

October 27, 1995

Mr. M. S. Tuckman
Senior Vice President
Nuclear Generation
Duke Power Company
P. O. Box 1006
Charlotte, North Carolina 28201-1006

SUBJECT:

SAFETY EVALUATION REGARDING TOPICAL REPORT DPC-NE-2007P. "FUEL

RECONSTITUTION ANALYSIS METHODOLOGY" (TAC NO. M88082)

Dear Mr. Tuckman:

By letter dated September 23, 1993, Duke Power Company (DPC) submitted Topical Report DPC-NE-2007P, "Fuel Reconstitution Analysis Methodology." This report presented DPC's methodology for performing analyses to support fuel assembly reconstitution, and was prepared in response to the guidance in Generic Letter 90-02, Supplement 1. The topical report is intended to be applicable to Oconee, Units 1, 2, and 3; McGuire, Units 1 and 2; and Catawba, Units 1 and 2. A supplemental response dated August 8, 1994, was submitted in response to our request for additional information dated April 26, 1994.

The NRC staff and its contractor, Pacific Northwest Laboratory (PNL), have completed their review of the topical report and the supplemental submittal. The staff's Safety Evaluation (SE) and PNL's Technical Evaluation Report are enclosed. The staff concluded that DPC-NE-2007P is acceptable for reload licensing applications for the use of solid replacement rods within the limits specified in the conclusion section of the SE. The staff does not approve of the use of vacancies in place of fuel rods because the fuel assemblies with vacancies do not conform to NRC-approved thermal-hydraulic, seismic, and loss-of-coolant accident (LOCA) analyses.

The staff does not intend to repeat its review of the matters described in the report and found acceptable when the report is referenced in a license application, except to ensure that the material presented is applicable to the specific plant involved. Staff acceptance applies only to the matters described in the report.

In accordance with procedures established in NUREG-0390, DPC must publish accepted proprietary and non-proprietary versions of this report. The accepted versions shall incorporate this letter and the enclosed evaluation between the title page and the abstract. The accepted versions shall include an "-A" (designating accepted) following the report identification symbol.

Should NRC criteria or regulations change so that staff conclusions regarding the acceptability of the report are invalidated, DPC will be expected to revise and resubmit their documentation, or to submit justification for continued effective applicability of the topical report without revision of their documentation.

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If you have questions regarding this matter, please contact Len Wiens at (301) 415-1495.

Sincerely,

Herbert N. Berkow, Director Project Directorate II-2

Division of Reactor Projects-I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270, 50-287, 50-369, 50-370, 50-413, and 50-414

Enclosure: Safety Evaluation w/attached Technical Evaluation Report

cc w/enclosure: See next page If you have questions regarding this matter, please contact Len Wiens at (301) 415-1495.

Sincerely,

Original signed by:

Herbert N. Berkow, Director Project Directorate II-2 Division of Reactor Projects-I/II Office of Nuclear Reactor Regulation

Docket Nos. 50-269, 50-270, 50-287, 50-369, 50-370, 50-413, and 50-414

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Sincerelý,

Original signed by:

Herbert N. Berkow, Director Project Directorate II-2 Division of Reactor Projects-I/II Office of Nuclear Reactor Regulation

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