REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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FACIL:50-287 Oconee Nuclear Station, Unit 3, Duke Power Co. 05000287
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Region 2, Atlanta, Office of the Director

SUBJECT: LER 79-001/03L-0 on 790111:unidentified RCS leakage exceeded 1-GPM.Caused by package leakage from RCS valves 3RC-2 or 3RC-3 &/or pressurizer instrumentation valve.Caused by leaking valve,will be repaired at next shutdown.

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DUKE POWER COMPANY
OCONEE UNIT 3

Report Number: RO-287/79-1

Report Date: February 12, 1979

Occurrence Date: January 11, 1979

Facility: Oconee Unit 3, Seneca, South Carolina

Identification of Occurrence: RCS Leakage in Excess of 1.0 GPM

Conditions Prior to Occurrence:

At 0400 on January 11, 1979 it was determined that the Reactor Coolant System (RCS) had unidentified leakage in excess of 1.0 GPM, in violation of Oconee Nuclear Technical Specification 3.1.6.2. This determination was made as the result of an increase in the Reactor Building (RB) normal sump level, yielding an estimated leak rate of from .75 to 2.25 GPM over the period from 2330 January 10, 1979 to 0200 January 11, 1979. The increase was noted at 0254 on January 11, when the RB sump temperature alarm actuated and a significant increase in the RB particulate monitor (3RIA-47) activity was observed. At 0700 a safety evaluation was initiated, and the RB was entered to determine the orgin of the leakage. In addition, a video camera inspection of inaccessible areas of the RB was made. The leakage was discovered to be the result of steam discharge into the drain funnels on the pressurizer side of the primary shield wall. No other leakage was observed. This discharge had previously been identified as packing leakage from RCS valves 3RC-2, 3RC-3, and/or several pressurizer instrumentation valves. Therefore, the excess leakage was reclassified as identified leakage. The safety evaluation regarding the nature and location of the leak was completed by 1000 on January 11, and since the leak rate was well below the 10 GPM limit specified for identified leakage, the decision was made to continue operation. The leaking valve or valves will be repaired at the next shutdown, and a determination as to the cause of the packing failure will be made at that time.

Apparent Cause of Occurrence:

The leakage was identified to be the discharge of steam into the drain funnels as a result of a valve packing leak. The valve will be repaired and the cause of the packing failure will be determined at the next unit outage.

Analysis of Occurrence:

The RCS leakage is due to a packing leak from RCS valve 3RC-2 or 3RC-3 and/or a pressurizer instrumentation valve and does not constitute a violation of the RCS strength boundary. The leak rate is well within the 10 GPM limit allowed by Technical Specification 3.1.6.1. Therefore, the leakage does not endanger the public health and safety.

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Corrective Action:

The leakage was identified to be the result of a previously observed discharge of steam into the drain funnels. The source of the discharge is a valve packing leak, which will be identified and repaired at the next available unit outage.

NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

EXHIBIT A

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