



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 17, 2016

LICENSEE: PSEG Nuclear LLC

FACILITY: Salem Nuclear Generating Station, Unit Nos. 1 and 2

SUBJECT: SUMMARY OF MARCH 30, 2016, PUBLIC PRE-APPLICATION MEETING WITH PSEG NUCLEAR LLC TO DISCUSS LICENSE AMENDMENT REQUEST REGARDING CONTAINMENT FAN COIL UNIT ALLOWED OUTAGE TIME AT THE SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 (CAC NOS. MF7445 AND MF7446)

On March 30, 2016, the U.S. Nuclear Regulatory Commission (NRC) staff conducted a Category 1 public meeting with PSEG Nuclear LLC (the licensee) at NRC Headquarters, One White Flint North, 11555 Rockville Pike, Rockville, Maryland. The purpose of the meeting was to discuss a potential risk-informed license amendment request (LAR) concerning proposed changes to the containment fan coil unit (CFCU) technical specifications (TSs) for the Salem Nuclear Generating Station (Salem), Unit Nos. 1 and 2. Specifically, the proposed TS changes would extend the allowed outage time for one or two inoperable CFCUs from 7 days to 14 days. A list of attendees is enclosed. The meeting notice and agenda, dated March 15, 2016, are available at Agencywide Documents Access and Management System (ADAMS) Accession No. ML16076A019.

The licensee's presentation slides are available at ADAMS Accession No. ML16089A070. The licensee provided an overview of the proposed changes; background information, including the current TS requirements; and the scope of the planned request, including the basis for the risk assessment. The licensee stated that it expected to submit the LAR by the third quarter of 2016 and will request a normal 1-year review. The licensee answered clarifying questions from the NRC staff. Listed below are the significant areas of discussion between the licensee and NRC staff:

- The licensee and NRC staff discussed the 2008 peer review of the Salem, Unit Nos. 1 and 2, internal events Probabilistic Risk Assessment (PRA), including a discussion of the guidance used to perform peer reviews (i.e., Nuclear Energy Institute (NEI) 00-02, "Probabilistic Risk Assessment Peer Review Process Guidance," and NEI 05-04, "Process for Performing Internal Events PRA Peer Reviews Using the ASME/ANS PRA Standard"). Following the meeting, the NRC staff informed the licensee of the following additional information regarding the PRA peer review process:
 - Appendix B of Regulatory Guide (RG) 1.200, "An Approach for Determining the Technical Adequacy of Probabilistic Risk Assessment Results for Risk-Informed Activities," Revision 2, provides the NRC's position on PRA peer reviews conducted in accordance with NEI 00-02, which includes conducting a supplemental self-assessment (i.e., beyond the peer review process) to satisfy the peer review requirements of the ASME/ANS-RA-Sb-2005 PRA Standard.

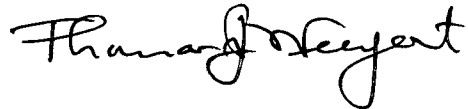
- Appendix C of RG 1.200, Revision 2, provides the NRC's position on PRA peer reviews conducted in accordance with NEI 05-04.
- The current version of the PRA standard is ASME/ANS RA-Sa-2009, "Standard for Level 1/Large Early Release Frequency Probabilistic Risk Assessment for Nuclear Power Plant Applications," as qualified by RG 1.200, Revision 2. The licensee indicated that the 2008 peer review of the Salem internal events PRA was performed to an earlier version of the PRA standard. In accordance with RG 1.200, Revision 2, it is expected that the differences between ASME/ANS RA-Sa-2009, as qualified by RG 1.200 Revision 2, and the earlier version of the standard used in the Salem PRA peer review be identified and addressed (i.e., perform a gap assessment). Following the meeting, the NRC staff informed the licensee of the following additional information pertaining to gap assessments:
 - Section 3.3, "Gap Assessment for PRAs Reviewed Against RG 1.200, Revision 1," of NEI 05-04, Revision 3, provides guidance on performing this gap assessment. The following are examples of gap assessments that are publically available: response to NRC RAI 3 in a letter dated May 2, 2014 (ADAMS Accession No. ML14122A364), and response to NRC RAI 1 in a letter dated November 4, 2014 (ADAMS Accession No. ML14308A196).
 - In accordance with Regulatory Position 4.2 of RG 1.200, Revision 2, the NRC staff expects licensees to submit a discussion of the resolution of the peer review (including supplemental self-assessment for peer reviews performed using the criteria in NEI 00-02 and any gap assessments) findings and observations that are applicable to the parts of the PRA required for the application.
- The licensee and NRC staff discussed details of the planned risk assessment. The licensee's approach seems to be consistent with guidance in RG 1.174, "An Approach for Using Probabilistic Risk Assessment in Risk-Informed Decisions on Plant-Specific Changes to the Licensing Basis," Revision 2, and RG 1.177, "An Approach for Plant-Specific, Risk-Informed Decisionmaking: Technical Specifications," Revision 1.

The public was invited to observe the meeting and was given an opportunity to communicate with the NRC staff after the business portion, but before the meeting was adjourned. No members of the public participated in the meeting or listened in on the telephone. No public meeting feedback forms were received. No regulatory decisions were made at the meeting.

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If you have any questions, please contact me at 301-415-4037 or Thomas.Wengert@nrc.gov.

Sincerely,

A handwritten signature in black ink that reads "Thomas J. Wengert". The signature is written in a cursive style with a large, prominent initial "T".

Thomas J. Wengert, Senior Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

Enclosure:
List of Attendees

cc w/enclosure: Distribution via Listserv

LIST OF ATTENDEES

MARCH 30, 2016, MEETING WITH PSEG NUCLEAR LLC

LICENSE AMENDMENT REQUEST PRE-APPLICATION

FOR THE SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

Name	Organization
Paul Duke	PSEG
Gary DeMoss	PSEG
Kevin King	PSEG
Margaret Chernoff	NRC/NRR/DSS
Larry Wheeler	NRC/NRR/DSS
Todd Hilsmeier	NRC/NRR/DRA
Adrienne Driver	NRC/NRR/DRA
Tom Wengert	NRC/NRR/DORL
Doug Broaddus*	NRC/NRR/DORL
Nageswara Karipineni	NRC/NRR/DSS

*Participated by telephone

Acronyms

DORL = Division of Operating Reactor Licensing
DRA = Division of Risk Assessment
DSS = Division of Safety Systems
NRC = U.S. Nuclear Regulatory Commission
NRR = Office of Nuclear Reactor Regulation
PSEG = PSEG Nuclear LLC

Enclosure

If you have any questions, please contact me at 301-415-4037 or Thomas.Wengert@nrc.gov.

Sincerely,

/RA/

Thomas J. Wengert, Senior Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

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List of Attendees

cc w/enclosure: Distribution via Listserv

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ADAMS Accession No.: ML16123A132 Mtg. Notice ML16076A019 Slides ML16089A070 *by e-mail

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NAME	TWengert	LRonewicz	SRosenberg (w/comments)	DBroaddus (AHon for)	TWengert
DATE	5/16/16	5/10/16	5/11/16	5/17/16	5/17/16

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