



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

May 27, 2016

Mr. Jerud Hanson  
Nuclear Energy Institute  
1201 F St., NW  
Suite 1100  
Washington, DC 20004-1218

SUBJECT: LICENSE RENEWAL INTERIM STAFF GUIDANCE, LR-ISG-2016-01,  
"CHANGES TO AGING MANAGEMENT GUIDANCE FOR VARIOUS STEAM  
GENERATOR COMPONENTS"

Dear Mr. Hanson:

I am writing to inform the Nuclear Energy Institute of the opportunity to comment on the draft License Renewal Interim Staff Guidance (LR-ISG), LR-ISG-2016-01, "Changes to Aging Management Guidance for Various Steam Generator Components." Enclosed is a copy of the *Federal Register* notice that announces the U.S. Nuclear Regulatory Commission's (NRC's) request for comments and provides instructions on how to submit comments.

Based on recent operating experience and staff's review of the analyses performed by the industry, the NRC staff has determined that the existing guidance in NUREG-1800, Revision 2, "Standard Review Plan for Review of License Renewal Applications for Nuclear Power Plants" (SRP-LR), and NUREG-1801, Revision 2, "Generic Aging Lessons Learned (GALL) Report," should be revised to provide updated recommendations for aging management of steam generator divider plate assemblies, tube-to-tubesheet welds, tubesheets, and heads (i.e., channel or lower/upper heads) within the scope of Title 10 of the *Code of Federal Regulations* Part 54.

Examples of the major changes are: (a) addition of visual inspections of steam generator head internal areas to GALL Report Section XI.M19, "Steam Generators," to detect signs of cracking or loss of material (e.g., detection of rust stains and gross cracking or distortion); (b) changes to SRP-LR Sections 3.1.2.2.11 and 3.1.3.2.11, "Cracking due to Primary Water Stress Corrosion Cracking" that describe further evaluation regarding primary water stress corrosion cracking (PWSCC) in divider plate assemblies and tube-to-tubesheet welds; and (c) changes to the aging management review items in the GALL Report and SRP-LR to provide updated guidance for managing cracking due to PWSCC and loss of material due to boric acid corrosion in steam generator components.

We also plan to consider the information in this LR-ISG and make corresponding changes when finalizing the aging management guidance for the subsequent license renewal period (i.e., up to 80 years of operation), which is documented in draft NUREG-2191, "Generic Aging Lessons Learned for Subsequent License Renewal Report" (GALL-SLR), and draft NUREG-2192, "Standard Review Plan for Review of Subsequent License Renewal for Nuclear Power Plants" (SRP-SLR), if it is practicable to do so in terms of the development schedule for subsequent license renewal guidance.

J. Hanson

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To inform other stakeholders of the opportunity to provide comments on the draft LR-ISG-2016-01, courtesy copies of this letter are being sent to subscribers of an NRC electronic mailing list on general license renewal topics.

If you have any questions, please contact Dr. Seung Min of my staff by telephone at 301-415-2045 or by e-mail at [Seung.Min@nrc.gov](mailto:Seung.Min@nrc.gov).

Sincerely,

*/RA/*

Dennis C. Morey, Acting Deputy Director  
Division of License Renewal  
Office of Nuclear Reactor Regulation

Enclosure:  
As stated

cc: Reactor License Renewal Stakeholder  
Group Listserv

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Dennis C. Morey, Acting Deputy Director  
Division of License Renewal  
Office of Nuclear Reactor Regulation

Enclosure:  
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cc: Reactor License Renewal Stakeholder  
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**ADAMS Accession Nos.:** ML16102A232 (Package), ML16102A308 (Letter), ML16102A268 (Draft LR-ISG), ML16099A259 (FRN)

**\*concurrence via email**

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Letter to J. Hanson from D. Morey dated May 27, 2016

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